



UNITED STATES  
 NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 5, 1993

Docket No. 50-461

Mr. Richard F. Phares  
 Director - Licensing  
 Clinton Power Station  
 P. O. Box 678  
 Mail Code V920  
 Clinton, Illinois 61727

Dear Mr. Phares:

SUBJECT: ISSUANCE OF AMENDMENT (TAC NO. M82642)

The U. S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 82 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. The amendment is in response to your application dated December 27, 1991 (U-601916).

The amendment modifies Clinton Power Station Technical Specification (TS) 3/4.7.2, "Control Room Ventilation System," by revising Action Statement b.1 to provide an alternative where the operable Control Room Ventilation System does not have to be operated in the high radiation mode. Also, Surveillance Requirement 4.7.2.e.5 is revised to reference the control room pressure relative to the adjacent areas as opposed to the outside atmosphere as previously identified. In addition, as discussed with your staff, a number of editorial changes were made to TS 3/4.7.2 so that it would more closely resemble the staff's Improved Standard Technical Specifications for BWR/6 facilities as found in NUREG-1434.

The following pages from recently issued amendments are being reissued due to typographical and formatting errors. Page 3/4 3-67 from Amendment No. 79 is being reissued to correctly refer to turbine first stage pressure "values" as opposed to "valves." Page 3/4 1-8 from Amendment No. 81 is being reissued with revised numbering of the surveillance requirements so that the Technical Specifications will no longer have two surveillance requirements numbered 4.1.3.3. Page 3/4 5-5 from Amendment No. 81 is being reissued due to a formatting error. This last correction will result in the issuance of a new page 3/4 5-4a and the reissuance of pages 3/4 5-5 and 3/4 5-6 to their previous versions.

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Mr. Richard F. Phares

- 2 -

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Original Signed By:

Douglas V. Pickett, Project Manager  
Project Directorate III-2  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 82 to NPF-62
2. Safety Evaluation

cc w/enclosures:  
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DOCUMENT NAME: G:CLI82642.AMD

Docket No. 50-461

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Director - Licensing  
Clinton Power Station  
P. O. Box 678  
Mail Code V920  
Clinton, Illinois 61727

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The amendment modifies Clinton Power Station Technical Specification (TS) 3/4.7.2, "Control Room Ventilation System," by revising Action Statement b.1 to provide an alternative where the operable Control Room Ventilation System does not have to be operated in the high radiation mode. Also, Surveillance Requirement 4.7.2.e.5 is revised to reference the control room pressure relative to the adjacent areas as opposed to the outside atmosphere as previously identified. In addition, as discussed with your staff, a number of editorial changes were made to TS 3/4.7.2 so that it would more closely resemble the staff's Improved Standard Technical Specifications for BWR/6 facilities as found in NUREG-1434.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

Douglas V. Pickett, Project Manager  
Project Directorate III-2  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. to NPF-62
- 2. Safety Evaluation

cc w/enclosures:  
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IA:RD32:DRPW  
CMoore  
7/19/93

*DVR*  
PE:PD32:DRPW  
RLauffer  
7/15/93

*DVR*  
PM:PD32:DRPW  
DPickett  
7/15/93

*CG*  
BC:OTSB  
CGrimes  
7/19/93

D:PD32:DRPW  
JDyer  
8/2/93

*RB*  
OGC-OWFN  
R Bachmann  
7/22/93

DOCUMENT NAME: G:CL182642.AMD

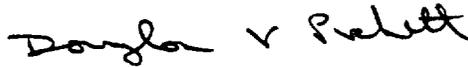
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Mr. Richard F. Phares

- 2 -

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,



Douglas V. Pickett, Project Manager  
Project Directorate III-2  
Division of Reactor Projects III/IV/V  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 82 to NPF-62
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. Richard F. Phares  
Illinois Power Company

Clinton Power Station  
Unit No. 1

cc:

Mr. J. S. Perry  
Senior Vice President  
Clinton Power Station  
Post Office Box 678  
Clinton, Illinois 61727

Illinois Department  
of Nuclear Safety  
Office of Nuclear Facility Safety  
1035 Outer Park Drive  
Springfield, Illinois 62704

Mr. J. A. Miller  
Manager Nuclear Station  
Engineering Department  
Clinton Power Station  
Post Office Box 678  
Clinton, Illinois 61727

Mr. J. W. Blattner  
Project Manager  
Sargent & Lundy Engineers  
55 East Monroe Street  
Chicago, Illinois 60603

Sheldon Zabel, Esquire  
Schiff, Hardin & Waite  
7200 Sears Tower  
233 Wacker Drive  
Chicago, Illinois 60606

Resident Inspector  
U.S. Nuclear Regulatory Commission  
RR#3, Box 229 A  
Clinton, Illinois 61727

Mr. R. T. Hill  
Licensing Services Manager  
General Electric Company  
175 Curtner Avenue, M/C 481  
San Jose, California 95125

Regional Administrator, Region III  
799 Roosevelt Road, Building 4  
Glen Ellyn, Illinois 60137

Chairman of DeWitt County  
c/o County Clerk's Office  
DeWitt County Courthouse  
Clinton, Illinois 61727

Mr. Robert Neumann  
Office of Public Counsel  
State of Illinois Center  
100 W. Randolph, Suite 11-300  
Chicago, Illinois 60601



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 82  
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Illinois Power Company\* (IP), and Soyland Power Cooperative, Inc. (the licensees) dated December 27, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

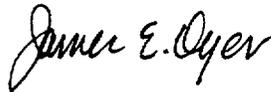
\*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 82 , are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director  
Project Directorate III-2  
Division of Reactor Projects - III/IV/V  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: August 5, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 82

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages, as indicated by asterisks, are provided to maintain document completeness.

Remove Pages

3/4 7-3  
3/4 7-4\*  
3/4 7-5\*  
3/4 7-6

Insert Pages

3/4 7-3  
3/4 7-4\*  
3/4 7-5\*  
3/4 7-6

The following pages are corrected pages from Amendment No. 79 or Amendment No. 81.

3/4 1-8  
3/4 3-67  
--  
3/4 5-5  
3/4 5-6

3/4 1-8  
3/4 3-67  
3/4 5-4a  
3/4 5-5  
3/4 5-6

## PLANT SYSTEMS

### 3/4.7.2 CONTROL ROOM VENTILATION SYSTEM

#### LIMITING CONDITION FOR OPERATION

3.7.2 Two independent Control Room Ventilation Systems shall be OPERABLE.†

APPLICABILITY: OPERATIONAL CONDITIONS 1, 2, 3, and \*.

#### ACTION:

- a. In OPERATIONAL CONDITION 1, 2 or 3 with one Control Room Ventilation System inoperable, restore the inoperable system to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- b. In OPERATIONAL CONDITION \*:
  1. With one Control Room Ventilation System inoperable, restore the inoperable system to OPERABLE status within 7 days. Otherwise, immediately initiate and maintain operation of the OPERABLE system in the high radiation mode of operation or suspend CORE ALTERATIONS, handling of irradiated fuel in the secondary containment and operations with a potential for draining the reactor vessel. The provisions of Specification 3.0.4 are not applicable for entry into OPERATIONAL CONDITION 4 or 5.
  2. With both Control Room Ventilation Systems inoperable, immediately suspend CORE ALTERATIONS, handling of irradiated fuel in the secondary containment and operations with a potential for draining the reactor vessel.
  3. The provisions of Specification 3.0.3 are not applicable.

#### SURVEILLANCE REQUIREMENTS

4.7.2 Each Control Room Ventilation System shall be demonstrated OPERABLE:†

- a. At least once per 12 hours by verifying that the control room air temperature is less than or equal to 86°F.
- b. At least once per 31 days on a STAGGERED TEST BASIS by initiating, from the control room, flow through the HEPA filters and charcoal adsorbers and verifying that the makeup filter system operates continuously for at least 10 hours with the heaters operating; and with flow through the recirculation charcoal adsorber for at least 15 minutes.

\*During movement of irradiated fuel assemblies in the primary or secondary containment, during CORE ALTERATIONS, or during operations with a potential for draining the reactor vessel.

†Automatic transfer to the chlorine mode is not required when chlorine containers having a capacity of 150 pounds or less are stored 100 meters or more from the control room or its fresh air inlets.

## PLANT SYSTEMS

### CONTROL ROOM VENTILATION SYSTEM

#### SURVEILLANCE REQUIREMENTS (Continued)

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##### 4.7.2 (Continued)

- c. At least once per 18 months or (1) after any structural maintenance on the makeup or recirculation HEPA filters or charcoal adsorber housings, or (2) following painting, fire or chemical release in any ventilation zone communicating with the makeup or recirculation filter system by:
1. Verifying that the makeup filter system satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% and uses the test procedure guidance in Regulatory Positions C.5.a, C.5.c, and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978\*, and the system flow rate is 3000 cfm  $\pm$  10%.
  2. Verifying that the recirculation filter system satisfies bypass leakage testing acceptance criteria of less than 2% total bypass and uses test procedure guidance in Regulatory Positions C.5.a and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978\*, and the system flow rate is 64,000 cfm  $\pm$  10%.
  3. Verifying, within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978\*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978\*, for a methyl iodide penetration of less than 0.175% for makeup filter system carbon adsorber and 6% for recirculation filter system carbon adsorber when tested; in accordance with ASTM D3803-79 methods, with the following parameters:

##### Make Up Filter System

- |                      |            |
|----------------------|------------|
| a) Bed Depth         | - 4 inches |
| b) Velocity          | - 40 fpm   |
| c) Temperature       | - 30°C     |
| d) Relative Humidity | - 70%      |

##### Recirculation Filter System

- |                      |            |
|----------------------|------------|
| a) Bed Depth         | - 2 inches |
| b) Velocity          | - 80 fpm   |
| c) Temperature       | - 30°C     |
| d) Relative Humidity | - 70%      |

4. Verifying flow rate of 3000 cfm  $\pm$  10% for the makeup filter system and 64,000 cfm  $\pm$  10% for the recirculation filter system during operation when tested in accordance with ANSI N510-1980.

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\*ANSI N510-1980 shall be used in place of ANSI N510-1975 as referenced in Regulatory Guide 1.52, Revision 2, March 1978.

## PLANT SYSTEMS

### CONTROL ROOM VENTILATION SYSTEM

#### SURVEILLANCE REQUIREMENTS (Continued)

##### 4.7.2 (Continued)

- d. After every 720 hours of charcoal adsorber operation, by verifying within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978\*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978\*, for a methyl iodide penetration of less than 0.175% for the makeup filter system carbon adsorber and 6% for the recirculation filter system carbon adsorber when tested; in accordance with ASTM D3803-79 methods, with the following parameters:

##### Make Up Filter System

- a) Bed Depth - 4 inches
- b) Velocity - 40 fpm
- c) Temperature - 30°C
- d) Relative Humidity - 70%

##### Recirculation Filter System

- a) Bed Depth - 2 inches
- b) Velocity - 80 fpm
- c) Temperature - 30°C
- d) Relative Humidity - 70%

- e. At least once per 18 months by:

1. Verifying that the pressure drop across the combined HEPA filters and charcoal adsorber banks is less than 6 inches Water Gauge while operating the makeup filter system at a flow rate of 3000 cfm  $\pm$  10%.
2. Verifying that on a high chlorine actuation\*\* and a manual initiation test signal, the system automatically\*\* switches to the chlorine mode of operation and the dampers close within 2 seconds.\*\*\*
3. Verifying that the control room leak rate is limited to < 4000 cfm  $\pm$  10% at  $\geq$  1/8-inch Water Gauge (W.G.) with respect to adjacent areas.
4. Verifying that on a smoke mode actuation test signal, the system automatically switches to the smoke mode of operation at a flow rate less than or equal to 64,000 cfm  $\pm$  10%.
5. Verifying that on a high radiation actuation test signal, the system automatically switches to the high radiation mode of operation and

\*ANSI N510-1980 shall be used in place of ANSI N510-1975 as referenced in Regulatory Guide 1.52, Revision 2, March 1978.

\*\*Automatic transfer to the chlorine mode is not required when chlorine containers having a capacity of 150 pounds or less are stored 100 meters from the control room or its fresh air inlets.

\*\*\*This specification is not applicable after all chlorine containers having a capacity of 100 pounds or greater are removed from the site including the chlorine containers located at the site sewage treatment plant.

## PLANT SYSTEMS

### CONTROL ROOM VENTILATION SYSTEM

#### SURVEILLANCE REQUIREMENTS (Continued)

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##### 4.7.2 (Continued)

the control room is maintained at a positive pressure of at least 1/8-inch W.G. with respect to adjacent areas during system operation at a flow rate less than or equal to 3000 cfm.

6. Verifying that the makeup filters heaters dissipate at least 14.4 kW when tested in accordance with ANSI N510-1980.
- f. After each complete or partial replacement of a HEPA filter bank in the makeup filter system, by verifying that the HEPA filter bank satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% in accordance with ANSI N510-1980 while operating the system at a flow rate of 3000 cfm  $\pm$  10%.
- g. After each complete or partial replacement of a charcoal adsorber bank in the makeup or recirculation filter systems, by verifying that the charcoal adsorber bank satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% for the makeup filter system and 2% total bypass leakage for the recirculation filter system in accordance with ANSI N510-1980 for a halogenated hydrocarbon refrigerant test gas while operating the makeup system at a flow rate of 3000 cfm  $\pm$  10% and the recirculation filter system at a flow rate of 64,000  $\pm$  10%.
- h. At least once per 18 months by verifying that the air leakage rate into the negative pressure portions of the Main Control Room Ventilation System ductwork located outside the Main Control Room habitability boundary between fan OVC04CA(B) and isolation dampers OVC03YA(B) inclusive, and fire dampers OVC042YA(E), OVC042YB(F), OVC042YC(G) and OVC042YD(H) to be  $\leq$  650 cfm when tested in accordance with an NRC-approved test method. In addition, visually verify the integrity of the Recirculation Filter Housing flexible connection to fan OVC03CA(B).

## REACTIVITY CONTROL SYSTEMS

### CONTROL ROD MAXIMUM SCRAM INSERTION TIMES

#### SURVEILLANCE REQUIREMENTS

4.1.3.2.1 The maximum scram insertion time of the control rods shall be demonstrated through measurement with reactor coolant pressure greater than or equal to 950 psig and, during single control rod scram time tests, the control rod drive pumps isolated from the accumulators:

- a. For all control rods prior to THERMAL POWER exceeding 40% of RATED THERMAL POWER following CORE ALTERATIONS or after a reactor shutdown that is greater than 120 days,
- b. For at least 10% of the control rods, on a rotating basis, at least once per 120 days of POWER OPERATION.

4.1.3.2.2 The maximum scram insertion time for specifically affected individual control rods following maintenance on or modification to the control rod or control rod drive system which could affect the scram insertion time of those specific control rods shall be demonstrated through measurement with reactor coolant pressure greater than or equal to 950 psig. Alternatively, those specific control rods may be determined OPERABLE with reactor coolant pressure less than 950 psig by demonstrating an acceptable scram insertion time to notch position 13. The scram time acceptance criteria for this alternate test shall be determined by linear interpolation between 0.95 seconds at a reactor coolant pressure of 0 psig and 1.40 seconds at 950 psig. If this alternate test is utilized, the individual scram time shall also be measured with reactor coolant pressure greater than or equal to 950 psig prior to exceeding 40% of RATED THERMAL POWER. For each of the above single control rod scram time tests, the control rod drive pumps shall be isolated from the accumulators.

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\*The provisions of Specification 4.0.4 are not applicable for entry into OPERATIONAL CONDITION 2 provided this surveillance requirement is completed prior to entry into OPERATIONAL CONDITION 1.

TABLE 3.3.6-2 (Continued)

CONTROL ROD BLOCK INSTRUMENTATION SETPOINTS

TABLE NOTATIONS

- \* These are turbine first stage pressure values.
- \*\* The Average Power Range Monitor rod block function is varied as a function of recirculation loop flow (W). The trip setting of this function must be maintained in accordance with note (a) of Table 2.2.1-1.
- # Instrument zero is 758' 5" msl.
- ## Instrument zero is 758' 4 1/2" msl.

EMERGENCY CORE COOLING SYSTEMS

ECCS - OPERATING

SURVEILLANCE REQUIREMENTS (Continued)

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4.5.1 (Continued)

- b. There is a corresponding change in the measured steam flow, or
  - c. The acoustic tail-pipe monitor alarms.
4. At least once per 18 months, performing a CHANNEL CALIBRATION of the accumulator low pressure alarm system and verifying an alarm setpoint of  $\geq 140$  psig on decreasing pressure.

## EMERGENCY CORE COOLING SYSTEMS

### 3/4.5.2 ECCS - SHUTDOWN

#### LIMITING CONDITION FOR OPERATION

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3.5.2 At least two of the following shall be OPERABLE and capable of being powered from a diesel generator of Specification 3.8.1.2.b.

- a. The low pressure core spray (LPCS) system with a flow path capable of taking suction from the suppression pool and transferring the water through the spray sparger to the reactor vessel.
- b. Low pressure coolant injection (LPCI) subsystem "A" of the RHR system with a flow path capable of taking suction from the suppression pool and transferring the water to the reactor vessel.
- c. Low pressure coolant injection (LPCI) subsystem "B" of the RHR system with a flow path capable of taking suction from the suppression pool and transferring the water to the reactor vessel.
- d. Low pressure coolant injection (LPCI) subsystem "C" of the RHR system with a flow path capable of taking suction from the suppression pool and transferring the water to the reactor vessel.
- e. The high pressure core spray (HPCS) system with a flow path capable of taking suction from one of the following water sources and transferring the water through the spray sparger to the reactor vessel:
  1. From the suppression pool, or
  2. When the suppression pool level is less than the limit or is drained, from the RCIC storage tank containing at least 125,000 available gallons of water.

APPLICABILITY: OPERATIONAL CONDITIONS 4 and 5\*.

#### ACTION:

- a. With one of the above required subsystems/systems inoperable, restore at least two subsystems/systems to OPERABLE status within 4 hours or suspend all operations that have a potential for draining the reactor vessel. The provisions of Specification 3.0.4 are not applicable.
- b. With both of the above required subsystems/systems inoperable, suspend CORE ALTERATIONS and all operations that have a potential for draining the reactor vessel. Restore at least one subsystem/system to OPERABLE status within 4 hours or establish PRIMARY CONTAINMENT INTEGRITY within the next 8 hours.

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\* The ECCS is not required to be OPERABLE provided that the reactor vessel head is removed, the cavity is flooded, the reactor cavity to steam dryer pool gate is open and water level in these upper containment pools is maintained within the limits of Specification 3.9.8 and 3.9.9.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 82 TO FACILITY OPERATING LICENSE NO. NPF-62  
ILLINOIS POWER COMPANY, ET AL.  
CLINTON POWER STATION, UNIT NO. 1  
DOCKET NO. 50-461

1.0 INTRODUCTION

By letter dated December 27, 1991, the Illinois Power Company (IP, the licensee) requested an amendment to Facility Operating License No. NPF-62 for the Clinton Power Station (CPS). The proposed amendment would modify Clinton Power Station Technical Specification (TS) 3/4.7.2, "Control Room Ventilation System," by revising Action Statement b.1 to provide an alternative where the operable Control Room Ventilation System does not have to be operated in the high radiation mode. Also, Surveillance Requirement (SR) 4.7.2.e.5 would be revised to reference the control room pressure relative to the adjacent areas as opposed to the outside atmosphere as currently identified.

2.0 EVALUATION

Technical Specification 3.7.2.b.1

During Operational Conditions 4, 5, or \* (when irradiated fuel is being handled in the secondary containment), TS 3.7.2.b.1 currently requires, in part, that "With one Control Room Ventilation System inoperable, restore the inoperable system to OPERABLE status within 7 days or initiate and maintain operation of the OPERABLE system in the high radiation mode of operation." This requirement ensures that the control room will remain habitable for operations personnel during and following all design basis accident conditions. In addition, requiring the operable system to be in the high radiation mode of operation ensures compliance with General Design Criterion 19 of Appendix A to 10 CFR 50.

Activities performed during Operational Conditions 4, 5, or \* that could lead to high radiation conditions in the control room include core alterations, handling of irradiated fuel within the secondary containment, and activities which could potentially drain the reactor vessel. However, as stated by the licensee, by precluding these activities, the need for the control room ventilation system to operate in the high radiation mode to maintain compliance with the design criterion is eliminated. Therefore, the licensee has proposed an option to TS 3.7.2.b.1. With one Control Room Ventilation System inoperable in Operational Conditions 4, 5, or \*, the licensee has proposed the option of either operating the OPERABLE system in the high

radiation mode, or suspending core alterations, handling of irradiated fuel in the secondary containment, and operations with the potential for draining the reactor vessel.

This proposed TS change will provide the licensee an alternative action which will increase the useable life of the system filters, and reduce the costs associated with both testing the sample canisters and replacing the adsorber charcoal due to unnecessary accumulated hours of operation. The proposed change will also reduce the potential radiation exposure to test and maintenance personnel incurred during these activities. Finally, the proposed TS change will provide the licensee more flexibility in scheduling work during refueling outages.

The licensee's proposal is consistent with the actions currently specified for the situation when both trains in the system are inoperable. Eliminating the requirement to run the OPERABLE train in the high radiation mode does not affect the operability of that train.

The staff concurs with the licensee's justification to allow the option of either running the OPERABLE control room ventilation system in the high radiation mode or suspending operations that could lead to high radiation conditions in the control room. The staff agrees that while in OPERATIONAL CONDITIONS 4, 5, or \*, by suspending core alterations, the handling of irradiated fuel within primary or secondary containment, and activities which could potentially drain the reactor vessel, the need to operate the control room ventilation system in the high radiation mode has been eliminated. Therefore, the staff finds the proposed changes acceptable.

While the staff finds the proposed changes acceptable, the staff approached the licensee regarding the incorporation of some minor, editorial changes that would more closely resemble the staff's Improved Standard Technical Specifications (ISTS) for BWR/6 facilities found in NUREG-1434. The licensee concurred in the staff's suggestions and they have been incorporated as follows:

- 1) The APPLICABILITY of specification 3.7.2 has been revised from "All OPERATIONAL CONDITIONS and \*" to "OPERATIONAL CONDITIONS 1, 2, 3, and \*." While this reflects the APPLICABILITY statement of the ISTS, it also envelopes those conditions when the control room atmosphere needs to be protected against high radiation conditions (i.e., OPERATIONAL CONDITIONS 1, 2, and 3 envelopes loss-of-coolant accidents while OPERATIONAL CONDITION \* envelopes fuel handling accidents).
- 2) ACTION 3.7.2.b has been revised from "In OPERATIONAL CONDITION 4, 5, or \*" to "In OPERATIONAL CONDITION \*." ACTION 3.7.2.b is only applicable for fuel handling accidents and OPERATIONAL CONDITION \* envelopes these conditions.

- 3) The staff suggested insertion of the word "immediately" in ACTIONS 3.7.2.b.1 and 3.7.2.b.2 for clarification as the existing technical specifications do not provide time constraints.
- 4) Previous ACTION 3.7.2.c was revised to become ACTION 3.7.2.b.3. This change is intended to clarify that the reference to Specification 3.0.3 is only applicable to fuel handling accidents under ACTION 3.7.2.b.
- 5) The definition of \* was modified to more closely represent the definition found in the ISTS.

#### Technical Specification 4.7.2.e.5

Surveillance Requirement 4.7.2.e.5 currently states that at least once every 18 months each Control Room Ventilation System shall be demonstrated operable by verifying the system automatically switches to the high radiation mode of operation following a high radiation actuation test signal and the control room is maintained at a positive pressure of at least 1/8-inch water gauge (W.G.) relative to the outside atmosphere. The licensee's proposal changes the wording of SR 4.7.2.e.5 to reference the control room pressure "with respect to the adjacent areas" as opposed to the outside atmosphere as currently identified.

As currently worded, SR 4.7.2.e.5 has generated confusion since it is not clear what is meant by "outside atmosphere". The proposed change will clarify this SR and make it consistent with SR 4.7.2.e.3 which references control room leakage "with respect to adjacent areas."

The staff concurs with the licensee's justification in that the proposed change will provide clarification of the intent of the surveillance requirement. Therefore, the staff finds the proposed change acceptable. The proposed change is also similar to that found in the staff's ISTS.

#### 3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendment. The State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes surveillance requirements. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding

(57 FR 40213). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

#### 5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Laufer  
D. Pickett

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