

February 23, 1989

Docket No. 50-461

Mr. Dale L. Holtzscher
Acting Manager - Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

Dear Mr. Holtzscher:

By letter dated January 31, 1989, the Commission issued Amendment No. 18 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment revised the Technical Specifications to support the first refueling of the reactor and subsequent reactor operation in the maximum extended operating domain and with reduced feedwater temperatures. Revised page 5 of the License and revised Technical Specification page 3/4 6-17 were inadvertently omitted from the amendment. These revised pages are enclosed for incorporation into your License and Technical Specifications.

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John B. Hickman, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
As stated

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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A handwritten signature in cursive script, reading "John B. Hickman".

John B. Hickman, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
As stated

cc w/enclosure:
See next page

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Clinton Power Station
Unit 1

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DeWitt County Courthouse
Clinton, Illinois 61727

CONTAINMENT SYSTEMS

DRYWELL AIR LOCKS

SURVEILLANCE REQUIREMENTS

4.6.2.3 Each drywell air lock shall be demonstrated OPERABLE:

- a. Within 72 hours following each closing, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying seal leakage rate less than or equal to 2 scf per hour when the gap between the door seals is pressurized to 3.0 psig.
- b. By conducting an overall air lock leakage test at 3.0 psig and verifying that the overall air lock leakage rate is within its limit:
 - 1. Each cold shutdown if not performed within the previous 6 months[#].
 - 2. Prior to establishing DRYWELL INTEGRITY when maintenance has been performed on the air lock that could affect the air lock sealing capability.
- c. By verifying, prior to drywell entry if not performed within the past 18 months, that only one door in the air lock can be opened at a time.

[#]At least once per 18 months, the air lock will be pressurized to 19.7 psid prior to conducting the overall air lock test.

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(8) Post-Fuel Loading Initial Test Program (Section 14, SER, SSER 5 and SSER 6)

Any changes to the initial test program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(9) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737, Section 7.5.3.1, SSER 5 and SSER 8, and Section 18, SER, SSER 5 and Safety Evaluation Dated April 17, 1987)

- a. IP in accordance with the commitment contained in a letter dated December 11, 1986, shall install and have operational separate power sources for each of the fuel zone level channels as provided for in Regulatory Guide 1.97 prior to startup following the first refueling outage.
- b. IP shall submit a detailed control room design final supplemental summary report within 90 days of issuance of the full power license that completes all the remaining items identified in Section 18.3 of the Safety Evaluation dated April 17, 1987.

- D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. These include: (a) an exemption from the requirements of 10 CFR 70.24 for the criticality alarm monitors around the fuel storage area; (b) an exemption from the requirements of Appendix A to 10 CFR Part 50, General Design Criterion 61 to permit a schedular deferral of completion of preoperational testing of a portion of the Fuel Handling System until prior to offloading fuel from the reactor vessel (Section 14, SSER 8); (c) an exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, substituting the seal leakage test at Pa of paragraph III.D.2(b)(iii) for the entire airlock test at Pa of paragraph III.D.2(b)(ii) of Appendix J when no maintenance has been performed in the airlock that could affect its sealing capability (Section 6.2.6 of SSER 6); and (d) an exemption from the requirement of paragraph III.C.3 of Appendix J, exempting the measured leakage rates from the main steam isolation valves from inclusion in the combined leak rate for the local leak rate tests (Section 6.2.6 of SSER 6). The special circumstances regarding each exemption, except for Item (a) above, are identified in the referenced section of the safety evaluation report and the supplements thereto.