

February 17, 1983

Docket No. 50-265

Mr. L. DelGeorge  
Director Nuclear Licensing  
Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

Dear Mr. DelGeorge:

The Commission has issued the enclosed Amendment No. 78 to Facility Operating License DPR-30 for the Quad Cities Nuclear Power Station, Unit 2. The Amendment consists of changes to the Technical Specifications in response to your application dated May 12, 1981.

The amendment changes provisions of the Technical Specifications pertaining to the reactor protection system delay and response times, and the average power range monitor gain adjustment.

Copies of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original Signed By:

Roby B. Bevan, Project Manager  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

- 1. Amendment No. 78 to DPR-30
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures  
See next page

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*Consent and FRN only*

Mr. L. DelGeorge  
Commonwealth Edison Company

cc:

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The Honorable Tom Corcoran  
United States House of Representatives  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY  
AND  
IOWA ILLINOIS GAS AND ELECTRIC COMPANY  
  
DOCKET NO. 50-265  
  
QUAD CITIES NUCLEAR POWER STATION, UNIT 2  
  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 78  
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated May 12, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 78, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 17, 1983

ATTACHMENT TO LICENSE AMENDMENT NO. 78

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix "A" Technical Specifications as follows:

Remove

- 1.1/2.1-2
- 1.1/2.1-2a
- 1.1/2.1-8
- 1.1/2.1-9
- 3.1/4.1-1

Replace

- 1.1/2.1-2
- 1.1/2.1-2a
- 1.1/2.1-8
- 1.1/2.1-9
- 3.1/4.1-1

QUAD-CITIES  
DPR-30

D. Reactor Water Level (Shutdown Condition)

Whenever the reactor is in the shutdown condition with irradiated fuel in the reactor vessel, the water level shall not be less than that corresponding to 12 inches above the top of the active fuel\* when it is seated in the core.

\*Top of active fuel is defined to be 360 inches above vessel zero (See Bases J.2).

Where:

FRP = fraction of rated thermal power (2511 MWt)

MFLPD = maximum fraction of limiting power density where the limiting power density for each bundle is the design linear heat generation rate for that bundle.

The ratio of FRP/MFLPD shall be set equal to 1.0 unless the actual operating value is less than 1.0 in which case the actual operating value will be used.

This adjustment may also be performed by increasing the APRM gain by the inverse ratio, MFLPD/FRP, which accomplishes the same degree of protection as reducing the trip setting by FRP/MFLPD.

2. APRM Flux Scram Trip Setting (Refueling or Startup and Hot Standby Mode)

When the reactor mode switch is in the Refuel or Startup Hot Standby position, the APRM scram shall be set at less than or equal to 15% of rated neutron flux.

3. IRM Flux Scram Trip Setting

The IRM flux scram setting shall be set at less than or equal to 120/125 of full scale.

4. When the reactor mode switch is in the startup or run position, the reactor shall not be operated in the natural circulation flow mode.

B. APRM Rod Block Setting

The APRM rod block setting shall be as shown in Figure 2.1-1 and shall be:

$$S \leq (0.58W_D + 50)$$

1.1/2.1-2

QUAD-CITIES  
DPR-30

The definitions used above for the APRM scram trip apply. In the event of operation with a maximum fraction limiting power density (MFLPD) greater than the fraction of rated power (FRP), the setting shall be modified as follows:

$$S \leq (0.58W_D + 50) \frac{FRP}{MFLPD}$$

The definitions used above for the APRM scram trip apply.

The ratio of FRP to MFLPD shall be set equal to 1.0 unless the actual operating value is less than 1.0, in which case the actual operating value will be used.

This adjustment may also be performed by increasing the APRM gain by the inverse ratio, MFLPD/FRP, which accomplishes the same degree of protection as reducing the trip setting by FRP/MFLPD.

- C. Reactor low water level scram setting shall be 144 inches above the top of the active fuel\* at normal operating conditions.
- D. Reactor low water level ECCS initiation shall be 84 inches (+4 inches /-0 inch) above the top of the active fuel\* at normal operating conditions.
- E. Turbine stop valve scram shall be  $\leq$  10% valve closure from full open.
- F. Turbine control valve fast closure scram shall initiate upon actuation of the fast closure solenoid valves which trip the turbine control valves.
- G. Main steamline isolation valve closure scram shall be  $\leq$  10% valve closure from full open.
- H. Main steamline low-pressure initiation of main steamline isolation valve closure shall be  $\geq$  825 psig.

\*Top of active fuel is defined to be 360 inches above vessel zero (See Bases 3.2)

1.1/2.1-2a

An increase in the APRM scram trip setting would decrease the margin present before the fuel cladding integrity safety limit is reached. The APRM scram trip setting was determined by an analysis of margins required to provide a reasonable range for maneuvering during operation. Reducing this operating margin would increase the frequency of spurious scrams, which have an adverse effect on reactor safety because of the resulting thermal stresses. Thus, the APRM scram trip setting was selected because it provides adequate margin for the fuel cladding integrity safety limit yet allows operating margin that reduces the possibility of unnecessary scrams.

The scram trip setting must be adjusted to ensure that the LWR transient peak is not increased for any combination of maximum fraction of limiting power density (MFLPD) and reactor core thermal power. The scram setting is adjusted in accordance with the formula in Specification 2.1.A.1, when the MFLPD is greater than the fraction of rated power (FRP).

The adjustment may be accomplished by increasing the APRM gain by the reciprocal of FRP/MFLPD. This provides the same degree of protection as reducing the trip setting by FRP/MFLPD by raising the initial APRM readings closer to the trip settings such that a scram would be received at the same point in a transient as if the trip settings had been reduced by  $\frac{FRP}{MFLPD}$ .

## 2. APRM Flux Scram Trip Setting (Refuel or Startup/Hot Standby Mode)

For operation in the Startup mode while the reactor is at low pressure, the APRM scram setting of 15% of rated power provides adequate thermal margin between the setpoint and the safety limit, 25% of rated. The margin is adequate to accommodate anticipated maneuvers associated with power plant startup. Effects of increasing pressure at zero or low void content are minor, cold water from sources available during startup is not much colder than that already in the system, temperature coefficients are small, and control rod patterns are constrained to be uniform by operating procedures backed up by the rod worth minimizer. Of all possible sources of reactivity input, uniform control rod withdrawal is the most probable cause of significant power rise. Because the flux distribution associated with uniform rod withdrawals does not involve high local peaks, and because several rods must be moved to change power by a significant percentage of rated power, the rate of power rise is very slow. Generally, the heat flux is in near equilibrium with the fission rate. In an assumed uniform rod withdrawal approach to the scram level, the rate of power rise is no more than 5% of rated power per minute, and the APRM system would be more than adequate to assure a scram before the power could exceed the safety limit. The 15% APRM scram remains active until the mode switch is placed in the Run position. This switch occurs when reactor pressure is greater than 825 psig.

## 3. IRM Flux Scram Trip Setting

The IRM system consists of eight channels, four in each of the reactor protection system logic channels. The IRM is a 5-decade instrument which covers the range of power level between that covered by the SRM and the APRM. The 5 decades are broken down into 10 ranges, each being one-half a decade in size.

The IRM scram trip setting of 120 divisions is active in each range of the IRM. For example, if the instrument were on Range 1, the scram setting would be 120 divisions for that range; likewise, if the instrument were on Range 5, the scram would be 120 divisions on that range. Thus, as the IRM is ranged up to accommodate the increase in power level, the scram trip setting is also ranged up.

The most significant sources of reactivity change during the power increase are due to control rod withdrawal. In order to ensure that the IRM provides adequate protection against the single rod withdrawal error, a range of rod withdrawal accidents was analyzed. This analysis included starting the accident at various power levels. The most severe case involves an initial condition in which the reactor is just subcritical and the IRM system is not yet on scale.

Additional conservatism was taken in this analysis by assuming that the IRM channel closest to the withdrawn rod is bypassed. The results of this analysis show that the reactor is scrammed and peak power limited to 1X of rated power, thus maintaining MCFR above the fuel cladding integrity safety limit. Based on the above analysis, the IRM provides protection against local control rod withdrawal errors and continuous withdrawal of control rods in sequence and provides backup protection for the APRM.



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DPR-30

B. APRM Rod Block Trip Setting

Reactor power level may be varied by moving control rods or by varying the recirculation flow rate. The APRM system provides a control rod block to prevent gross rod withdrawal at constant recirculation flow rate to protect against grossly exceeding the MCPR Fuel Cladding Integrity Safety Limit. This rod block trip setting, which is automatically varied with recirculation loop flow rate, prevents an increase in the reactor power level to excessive values due to control rod withdrawal. The flow variable trip setting provides substantial margin from fuel damage, assuming a steady-state operation at the trip setting, over the entire recirculation flow range. The margin to the safety limit increases as the flow decreases for the specified trip setting versus flow relationship; therefore the worst-case MCPR which could occur during steady-state operation is at 100% of rated thermal power because of the APRM rod block trip setting. The actual power distribution in the core is established by specified control rod sequences and is monitored continuously by the incore LPRM system. As with APRM scram trip setting, the APRM rod block trip setting is adjusted downward if the maximum fraction of limiting power density exceeds the fraction of rated power, thus preserving the APRM rod block safety margin. As with the scram setting, this may be accomplished by adjusting the APRM gains.

C. Reactor Low Water Level Scram

The reactor low water level scram is set at a point which will assure that the water level used in the bases for the safety limit is maintained. The scram setpoint is based on normal operating temperature and pressure conditions because the level instrumentation is density compensated.

D. Reactor Low Low Water Level ECCS Initiation Trip Point

The emergency core cooling subsystems are designed to provide sufficient cooling to the core to dissipate the energy associated with the loss-of-coolant accident and to limit fuel cladding temperature to well below the cladding melting temperature to assure that core geometry remains intact and to limit any cladding metal-water reaction to less than 1%. To accomplish their intended function, the capacity of each emergency core cooling system component was established based on the reactor low water level scram setpoint. To lower the setpoint of the low water level scram would increase the capacity requirement for each of the ECCS components. Thus, the reactor vessel low water level scram was set low enough to permit margin for operation, yet will not be set lower because of ECCS capacity requirements.

The design of the ECCS components to meet the above criteria was dependent on three previously set parameters: the maximum break size, the low water level scram setpoint, and the ECCS initiation setpoint. To lower the setpoint for initiation of the ECCS could lead to a loss of effective core cooling. To raise the ECCS initiation setpoint would be in a safe direction, but it would reduce the margin established to prevent actuation of the ECCS during normal operation or during normally expected transients.

E. Turbine Stop Valve Scram

The turbine stop valve closure scram trip anticipates the pressure, neutron flux, and heat flux increase that could result from rapid closure of the turbine stop valves. With a scram trip setting of 10% of valve closure from full open, the resultant increase in surface heat flux is limited such that MCPR remains above the MCPR fuel cladding integrity safety limit even during the worst-case transient that assumes the turbine bypass is closed.

F. Turbine Control Valve Fast Closure Scram

The turbine control valve fast closure scram is provided to anticipate the rapid increase in pressure and neutron flux resulting from fast closure of the turbine control valves due to a load rejection and subsequent failure of the bypass, i.e., it prevents MCPR from becoming less than the MCPR fuel cladding integrity safety limit for this transient. For the load rejection without bypass transient from 100% power, the peak heat flux (and therefore LHGR) increases on the order of 15% which provides wide margin to the value corresponding to 1% plastic strain of the cladding.

1.1/2.1-9

### 3.1/4.1 REACTOR PROTECTION SYSTEM

#### LIMITING CONDITIONS FOR OPERATION

**Applicability:**

Applies to the instrumentation and associated devices which initiate a reactor scram.

**Objective:**

To assure the operability of the reactor protection system.

#### SURVEILLANCE REQUIREMENTS

**Applicability:**

Applies to the surveillance of the instrumentation and associated devices which initiate reactor scram.

**Objective:**

To specify the type and frequency of surveillance to be applied to the protection instrumentation.

#### SPECIFICATIONS

- A. The setpoints, minimum number of trip systems, and minimum number of instrument channels that must be operable for each position of the reactor mode switch shall be as given in Tables 3.1-1 through 3.1-4. The system response times from the opening of the sensor contact up to and including the opening of the trip actuator contacts shall not exceed 50 milliseconds.
- B. If, during operation, the maximum fraction of limiting power density exceeds the fraction of rated power when operating above 25% rated thermal power, either:
  1. the APRM scram and rod block settings shall be reduced to the values given by the equations in Specifications 2.1.A.1 and 2.1.B. This may also be accomplished by increasing the APRM gain as described therein.
  2. the power distribution shall be changed such that the maximum fraction of limiting power density no longer exceeds the fraction of rated power.
- A. Instrumentation systems shall be functionally tested and calibrated as indicated in Tables 4.1-1 and 4.1-2 respectively.
- B. Daily during reactor power operation, the core power distribution shall be checked for maximum fraction of limiting power density (MFLPD) and compared with the fraction of rated power (FRP) when operating above 25% rated thermal power.
- C. When it is determined that a channel is failed in the unsafe condition and Column 1 of Tables 3.1-1 through 3.1-3 cannot be met, that trip system must be put in the tripped condition immediately. All other RPS channels that monitor the same variable shall be functionally tested within 8 hours. The trip system with the failed channel may be untripped for a period of time not to exceed 1 hour to conduct this testing. As long as the trip system with the failed channel contains at least one operable channel monitoring that same variable, that trip system may be placed in the untripped position for short periods of time to allow functional testing of all RPS instrument channels as specified by Table 4.1-1. The trip system may be in the untripped position for no more than 8 hours per functional test period for this testing.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 78 TO FACILITY LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

DOCKET NO. 50-265

1. Introduction

By letter dated May 12, 1981, Commonwealth Edison Company (CECo, the licensee) proposed that changes be made to the technical specifications (TS) for Quad Cities Unit 2. These changes would (1) correct the reactor protection system (RPS) delay and response times as given in the TS to the more restrictive values that were used in the licensing basis, and (2) provide for use of the average power range monitor (APRM) gain adjustment for operation at higher-than-normal power peaking. These changes are identical to those reviewed by the staff and approved for Unit 1, by letter dated December 5, 1980.

2. Discussion and Evaluation

RPS Delay and Response Times

The RPS delay time is the time from opening of the sensor contact to opening of the trip actuation contacts. The licensee proposes to change the RPS delay time now given in the TS (100 msec) to the more restrictive value (50 msec) used by the General Electric Company (GE) in the licensing analysis.

Since the RPS delay time given in the TS should be consistent with that used in the licensing analysis, the change of the RPS delay time from 100 msec to 50 msec is acceptable.

Similarly, the RPS response time is the time lapse between reaching the neutron sensor setpoint and the start of control rod motion. The licensee proposes to change the RPS response time of 390 msec now given in the TS to the more restrictive value of 290 msec used in the licensing analysis. Since the TS value should be consistent with that used in the licensing analysis, the change of the RPS response time from 390 msec to 290 msec is acceptable.

APRM Gain Adjustment

The licensee has proposed TS changes that would allow use of APRM gain adjustments rather than reducing APRM trip setpoints when operating at higher than normal power peaking; i.e., whenever the maximum fraction of limiting power density (MFLPD) exceeds the fraction of rated thermal power (FRP). This establishes an initial APRM signal closer to the flow-biased setpoints, and so has the same effect as

reducing the actual scram and rod block setpoints. The proposed changes also require that the ratio FRP/MFLPD multiplier shall apply only above 25% rated thermal power, in order to be consistent with the LHGR surveillance requirement, and in agreement with Standard Technical Specifications.

Operation in the manner described above, i.e., use of APRM gain adjustments rather than reducing APRM setpoints for operation when MFLPD exceeds FRP, has previously been reviewed for Quad Cities Unit 1, and approved by letter dated December 5, 1980. In our evaluation of the present application we have determined that the same considerations apply for Unit 2. On the basis of the foregoing, proposed operation in the manner described is acceptable.

### 3. Environmental Considerations

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §1.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

### 4. Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant reduction in a margin of safety, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Roby Bevan

Dated: February 17, 1983

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-265

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY  
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 78 to Facility Operating License No. DPR-30 issued to Commonwealth Edison Company and Iowa-Illinois Gas and Electric Company, which revised the Technical Specifications for operation of the Quad Cities Nuclear Power Station, Unit 2, located in Rock Island County, Illinois. The amendment is effective as of the date of issuance.

The amendment changes provisions of the Technical Specifications pertaining to the reactor protection system delay and response times, and the average power range monitor gain adjustment.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of the amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of the amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of the amendment.

• For further details with respect to this action, see (1) the application for amendment dated May 12, 1981 (2) Amendment No. 78 to License No. DPR-30, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Moline Public Library, 504 - 17th Street, Moline, Illinois. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 17th day of February 1983.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief  
Operating Reactors Branch #2  
Division of Licensing

**PDR**

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~~Docket File 27-48~~

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FEB 23 1983

Docket No. 27-48

MEMORANDUM FOR: Joseph M. Felton, Director  
Division of Rules and Records

FROM: Edward F. Hawkins, Acting Branch Chief  
Low-Level Waste Licensing Branch  
Division of Waste Management

SUBJECT: FEDERAL REGISTER NOTICE

Please have the attached Federal Register Notice published in the Federal Register as soon as possible. The original and 12 copies are provided in accordance with SECY procedures.

Original Signed By  
Edward F. Hawkins

Edward F. Hawkins, Acting Branch Chief  
Low-Level Waste Licensing Branch  
Division of Waste Management

Enclosure:  
FR Notice re Issuance of Special  
Nuclear Material Disposal  
License Amendment

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RF  
1/24/83*

OFFICE	WMLL <i>ras</i>	<del>WMLL</del>	ELD <i>From RLF</i>			
SURNAME	JAShaffner: <i>ajf</i>	EFHawkins	RLFonner			
DATE	83/02/23	83/02/23	83/02/24			

NUCLEAR REGULATORY COMMISSION

DOCKET 27-48

US ECOLOGY, INC.

AGENCY: U.S. Nuclear Regulatory Commission

ACTION: Notice of Issuance of Special Nuclear Material Disposal  
License Amendment

SUMMARY: On January 28, 1983 the U.S. Nuclear Regulatory Commission (NRC) issued Amendment 3 to NRC License No. 16-19204-01 to US Ecology, Inc. (previously Nuclear Engineering Co.) The license allows for the receipt and disposal of special nuclear material (SNM) at US Ecology's low level waste disposal facility located on the the Hanford Reservation near Richland, Washington. The amendment was developed primarily to provide increased compatibility with the State of Washington license for disposal of by-product and source material, to clarify licensee responsibilities with regard to site closure and decommissioning, and to define procedures for license termination following cessation of burial operations.



NRC has determined that the amended license will not result in any significant impact on the environment and therefore does not require preparation of an environmental impact statement. An environmental impact appraisal was prepared upon issuance of the license in 1979 and is available for public inspection.

ADDRESSES: The amended license and environmental impact appraisal are available for public inspection at the Commission's Public Document Room located at 1717 H Street, N.W., Washington, D.C. 20555, and at the Richland Public Library Reference Department, Swift and Northgate Streets, Richland, Washington 99352

FOR FURTHER INFORMATION CONTACT: Mr. Edward F. Hawkins, Acting Chief, Low-Level Waste Licensing Branch, Division of Waste Management, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. Telephone: (301) 427-4433.

## SUPPLEMENTARY INFORMATION

The Nuclear Regulatory Commission, on January 28, 1983, issued to US Ecology, Inc., (formerly Nuclear Engineering Co.) an amendment to its license to receive and dispose by burial, special nuclear material at its commercial low level waste disposal facility located in the Hanford Reservation near Richland, Washington. (US Ecology's corporate headquarters are located at 9200 Shelbyville Road, Louisville, Kentucky 40207). The recently issued amendment amends License No. 16-19204-01 in its entirety. All information concerning this license can be located in NRC Docket No. 27-48.

NRC has determined that issuance of the amended license will result in no significant environmental impact and therefore does not require the preparation of an environmental impact statement (EIS). The SNM license, as amended, authorizes disposal of waste containing small amounts of uranium-233 and uranium-235. Wastes containing plutonium in concentrations greater than 100 nanocuries per gram are prohibited from the site.

Disposal of by-product and source material at the site is licensed by the State of Washington Department of Social and Health Services. That

agency is also the site landlord. Preparation of this amendment has been coordinated with Washington State officials and is consistent with conditions of the state by-product and source material license, which was renewed October 11, 1982.

Conditions of this amendment clearly specify license obligations with regard to closure and decommissioning of the Hanford low level waste disposal facility and further specify licensee obligations beyond the license expiration date pending formal NRC licensing action.

The Commission finds that issuance of the amendment complies with the requirements of the Atomic Energy Act of 1954, as amended, and the requirements of Title 10, Chapter 1, Code of Federal Regulations.

Dated at Washington, D.C. this 23rd day of February 1983.

For the Nuclear Regulatory Commission



Edward F. Hawkins, Acting Branch Chief  
Low-Level Waste Licensing Branch  
Division of Waste Management