June 25, 1990

Docket No. 50-461

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Mr. Frank A. Spangenberg
Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

Dear Mr. Spangenberg:

SUBJECT: AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-62
CLINTON POWER STATION, UNIT NO. 1 (TAC NO. 73811)

The Commission has issued the enclosed Amendment No. 37 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment is in response to your application dated June 30, 1989.

This amendment to Table 3.6.4-1 of the Clinton Power Station Technical Specifications adds certain valves associated with containment penetrations 17, 35, 36, and 42 to the list of containment isolation valves. This change is in response to the determination by the NRC staff that four testable check valves must be considered containment isolation valves.

A copy of the Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next biweekly Federal Register notice.

Sincerely,

/S/

John B. Hickman, Project Manager
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 37 to License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

DOCUMENT NAME: AMENDMENT 73811

Office:	LA/PDIII-2	PM/PDIII-	D/PDIII-3	OGC
Surname:	CMoore	JHickman:ta	JHannon	E Holler
Date:	6/18/90	6/15/90	6/25/90	6/18/90

DF01
1/1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

June 25, 1990

Docket No. 50-461

Mr. Frank A. Spangenberg
Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

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Sincerely,

John B. Hickman, Project Manager
Project Directorate III-3
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Enclosures:
1. Amendment No. 37 to
License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Frank A. Spangenberg
Illinois Power Company

Clinton Power Station
Unit No. 1

cc: Mr. J. S. Perry
Vice President
Clinton Power Station
P. O. Box 678
Clinton, Illinois 61727

Mr. R. D. Freeman
Manager-Nuclear Station
Engineering Department
Clinton Power Station
P. O. Box 678
Clinton, Illinois 61727

Sheldon Zabel, Esquire
Schiff, Hardin & Waite
7200 Sears Tower
233 Wacker Drive
Chicago, Illinois 60606

Resident Inspector
U. S. Nuclear Regulatory Commission
RR#3, Box 229 A
Clinton, Illinois 61727

Mr. L. Larson
Project Manager
General Electric Company
175 Curtner Avenue, N/C 395
San Jose, California 95125

Regional Administrator, Region III
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137

Chairman of DeWitt County
c/o County Clerk's Office
DeWitt County Courthouse
Clinton, Illinois 61727

Illinois Department
of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Mr. Donald Schopfer
Project Manager
Sargent & Lundy Engineers
55 East Monroe Street
Chicago, Illinois 60603

Robert Neumann
Office of Public Counsel
State of Illinois Center
100 W. Randolph
Suite 11-300
Chicago, Illinois 60601

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 37
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Illinois Power Company* (IP), and Soyland Power Cooperative, Inc. (the licensees) dated June 30, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No.37 , are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Thomas V. Wambach
John N. Hannon, Director
Project Directorate III-3
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of issuance: June 25, 1990

ATTACHMENT TO LICENSE AMENDMENT NO.37

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Corresponding overleaf pages are provided to maintain document completeness.

Remove	Insert
3/4 6-44	3/4 6-44
3/4 6-46	3/4 6-46
3/4 6-53	3/4 6-53
3/4 6-55	3/4 6-55

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

	VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Test Connections, Vents, and Drains[a] (Continued)							
6)	Feedwater/RHR Line B	10	NA		NA		9.0
	1B21-F063B			1, 2, 3,#		Yes	
	1B21-F030B			1, 2, 3		No	
	1E12-F058B			1, 2, 3		No	
	1E12-F349B			1, 2, 3		No	
	1G33-F057			1, 2, 3		No	
7)	RHR A Suction	11	NA	1, 2, 3	NA	No	9.9
	1E12-F334A						
	1E12-F335A						
8)	RHR B Suction	12	NA	1, 2, 3	NA	No	9.9
	1E12-F334B						
	1E12-F335B						
9)	RHR C Suction	13	NA	1, 2, 3	NA	No	9.9
	1E12-F334C						
	1E12-F335C						
10)	RHR Shutdown Cooling	14	NA	1, 2, 3	NA	No	9.0
	1E12-F001						
11)	RHR/LPCI A Injection	15	NA	1, 2, 3	NA	No	9.0
	1E12-F107A						
	1E12-F331A						
	1E12-F329A						
12)	RHR/LPCI B Injection	16	NA	1, 2, 3	NA	No	9.0
	1E12-F107B						
	1E12-F331B						
	1E12-F329B						

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Test Connections, Vents and Drains[a] (Continued)						
13) RHR/LPCI C Injection 1E12-F056C 1E12-F351 1E12-F456B	17	NA	1, 2, 3	NA	No	9.0
14) RHR A Test Line 1E12-F365A 1E12-F366A 1E21-F346 1E21-F347 1E12-F414 1E12-F415 1E12-F418 1E12-F419 1E12-F420 1E12-F421	18	NA	1, 2, 3	NA	No	9.0
15) RHR C Test Line 1E12-F353 1E12-F354 1E12-F428 1E12-F429	19	NA	1, 2, 3	NA	No	9.0
16) RHR B Test Line 1E12-F365B 1E12-F366B 1E12-F426 1E12-F427	20	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

	VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Test Connections, Vents and Drains[a]		(Continued)					
17)	RHR HX 1E12-F432A 1E12-F433A	24	NA	1, 2, 3	NA	No	9.0
18)	RHR HX 1E12-F432B 1E12-F433B	26	NA	1, 2, 3	NA	No	9.0
19)	RCIC Pump Suction 1E51-F336 1E51-F337	28	NA	1, 2, 3	NA	No	9.9
20)	RCIC Suction Release Discharge 1E12-F436 1E12-F437	31	NA	1, 2, 3	NA	No	9.9
21)	LPCS Pump Suction 1E21-F331 1E21-F344	32	NA	1, 2, 3	NA	No	9.9
22)	HPCS Test To Supp. Pool 1E22-F376	33	NA	1, 2, 3	NA	No	9.9
23)	Supp. Pool Cleanup Pump Suction 1SF034	34	NA	1, 2, 3	NA	No	9.9

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Test Connections, Vents and Drains[a]	(Continued)					
24) HPCS Pump Discharge 1E22-F021 1E22-F366B	35	NA	1, 2, 3	NA	No	9.0
25) LPCS Pump Discharge 1E21-F013 1E21-F358 1E21-F356A	36	NA	1, 2, 3	NA	No	9.0
26) RCIC 1E51-F041	41	NA	1, 2, 3	NA	No	9.0
27) Head Spray 1E51-F034 1E51-F391 1E12-F061 1E51-F367	42	NA	1, 2, 3	NA	No	9.0
28) RCIC Turb Steam Supply 1E51-F399 1E51-F072 1E51-F401	43	NA	1, 2, 3	NA	No	9.0
29) RCIC Turb Vacuum Breaker 1E51-F080 1E51-F082 1E51-F375 1E51-F376 1E51-F083	44	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

	VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Other Isolation Valves (Continued)							
9)	RHR C Suction Line 1E12-F105	13	NA	1, 2, 3	NA	No	9.9
10)	RHR A Shutdown Cooling 1E12-F475	14	NA	1, 2, 3	NA	No	9.0
11)	RHR/LPCI A Injection 1E12-F027A 1E12-F042A 1E12-F028A	15	NA	1, 2, 3	NA	No	9.0
12)	RHR/LPCI B Injection 1E12-F027B 1E12-F042B 1E12-F028B	16	NA	1, 2, 3	NA	No	9.0
13)	RHR/LPCI C Injection 1E12-F042C 1E12-F041C 1E12-F301C	17	NA	1, 2, 3	NA	No	9.0
14)	RHR A/LPCS Test Line 1E21-F011 1E12-F064A	18	NA	1, 2, 3	NA	No	9.9
15)	RHR C Test Line 1E12-F064C	19	NA	1, 2, 3	NA	No	9.9
16)	RHR B Test Line 1E12-F064B	20	NA	1, 2, 3	NA	No	9.9

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Other Isolation Valves (Continued)						
17) RHR A Suction Relief 1E12-F017A	21	NA	1, 2, 3	NA	No	9.9
18) RHR Shutdown Cool Relief 1E12-F005	23	NA	1, 2, 3	NA	No	9.9
19) RHR A HX Relief Line 1E12-F055A	24	NA	1, 2, 3	NA	No	9.9
20) RHR B Suction Relief 1E12-F017B	25	NA	1, 2, 3	NA	No	9.9
21) RHR B HX Relief Line 1E12-F055B	26	NA	1, 2, 3	NA	No	9.9
22) RHR/LPCI B Inj. Relief 1E12-F025B	27	NA	1, 2, 3	NA	No	9.0
23) RHR C Suction Relief 1E12-F101	29	NA	1, 2, 3	NA	No	9.9
24) RHR/LPCI C Inj. Relief 1E12-F025C	30	NA	1, 2, 3	NA	No	9.9
25) RHR To RCIC Suction Relief 1E12-F036	31	NA	1, 2, 3	NA	No	9.9
26) LPCS Suction Line 1E21-F001	32	NA	1, 2, 3	NA	No	9.9

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

	VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Other Isolation Valves (Continued)							
27)	HPCS Test Line & Relief 1E22-F014 1E22-F035 1E22-F039 1E22-F012	33	NA	1, 2, 3	NA	No	9.9
28)	HPCS Injection Line 1E22-F004 1E22-F005 1E22-F304	35	NA	1, 2, 3	NA	No	9.0
29)	LPCS Injection Line 1E21-F005 1E21-F006 1E21-F340	36	NA	1, 2, 3	NA	No	9.0
30)	HPCS Suction Line 1E22-F015	37	NA	1, 2, 3	NA	No	9.9
31)	LPCS Pump Relief Line 1E21-F018 1E21-F031	38	NA	1, 2, 3	NA	No	9.9
32)	RCIC Min. Flow Relief 1E51-F090 1E51-F019	40	NA	1, 2, 3	NA	No	9.9
33)	RCIC Turbine Exhaust 1E51-F068 1E51-F040	41	NA	1, 2, 3	NA	No	9.9
34)	RCIC Head Spray 1E51-F013 1E51-F066 1E51-F316	42	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
Other Isolation Valves (Continued)						
Primary Containment (Continued)						
35) SX To Containment Cooler 1SX089A 1SX088A	48	NA	1, 2, 3	NA	No	9.0
36) Instrument Air 1IA175	57	NA	1, 2, 3	NA	No	9.0
37) Instrument Air Bottles 1IA042B 1IA012A	58	NA	1, 2, 3,#	NA	Yes	9.0
38) CRD 1C11-F122 1C11-F083	63	NA	1, 2, 3,#	NA	Yes	9.0
39) RHR Flush Line 1E12-F030	76	NA	1, 2, 3	NA	No	9.9
40) RHR/LPCI A Injec. Relief 1E12-F025A	87	NA	1, 2, 3	NA	No	9.0
41) RHR HX A Vent 1E12-F074A	89	NA	1, 2, 3	NA	No	9.9

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 37 TO FACILITY OPERATING LICENSE NO. NPF-62

ILLINOIS POWER COMPANY, ET AL.

CLINTON POWER STATION, UNIT NO. 1

DOCKET NO. 50-461

1.0 INTRODUCTION

In March 1987, an allegation was made in which it was noted that certain pressure isolation testable check valves inside the drywell at Clinton Power Station were not being considered containment isolation valves and appropriately tested as required by 10 CFR Part 50, Appendix J. This allegation was subsequently investigated by NRC Region III and then reviewed by NRR. The NRC staff concluded that the following four testable check valves must be considered containment isolation valves:

(1) 1E12-F041C associated with Loop "C" of the Residual Heat Removal (RHR) System Low Pressure Coolant Injection (LPCI) Mode;

(2) 1E22-F005 associated with the High Pressure Core Spray (HPCS) System;

(3) 1E21-F006 associated with the Low Pressure Core Spray (LPCS) System; and

(4) 1E51-F066 associated with the Reactor Core Isolation Cooling (RCIC) System.

In response to NRC's determination that Illinois Power should consider the above-noted testable check valves to be containment isolation valves, the Illinois Power Company (IP), et al., requested, by letter dated June 30, 1989, an amendment to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. That amendment request reflected a re-establishment of the containment test boundary to include the applicable testable check valves as containment isolation valves. In the course of establishing the new test boundary, some test connection/vent/drain valves were added as containment isolation valves and some were deleted. The amendment reflects these changes as well.

2.0 EVALUATION

The addition of the four testable check valves to Table 3.6.4-1 as containment isolation valves is a result of a previous NRC determination that these valves should be considered containment isolation valves and is, therefore, acceptable to the staff. The addition of the associated three-fourth-inch test connection/vent/drain or three-fourth-inch bypass equalization valves to the

list of containment isolation valves was previously determined by the NRC not to be necessary as these valves would not require any special testing. However, the addition of the test, vent, drain, and bypass valves to Table 3.6.4-1 is a conservative action and therefore, is acceptable to the staff. The deletion of certain vent and drain valves from Table 3.6.4-1 is based on the re-establishment of the containment test boundary to include the four testable check valves. With the addition of the testable check valves as containment isolation valves the noted vent and drain valves are no longer necessary as containment isolation valves, therefore, their deletion from the table is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or a change to a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Prinicipal Contributor: John B. Hickman

Dated: June 25, 1990