

December 21, 1988

Docket No.: 50-461

Mr. Dale L. Holtzscher
Acting Manager - Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

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Dear Mr. Holtzscher:

SUBJECT: TECHNICAL SPECIFICATION CHANGE REQUEST TO CORRECT TYPOGRAPHICAL
ERRORS AND CLARIFY EXISTING REQUIREMENTS (TAC NO. 66553)

Re: Clinton Power Station, Unit No. 1

The Commission has issued the enclosed Amendment No. 14 to the Facility
Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This
amendment consists of changes to the Technical Specifications (TSs) in
response to your application dated October 30, 1987.

This amendment revises Technical Specification Table 3.6.4-1 in order to
correct typographical errors and clarify existing requirements regarding Note
(a).

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be
included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Janice A. Stevens, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 14 to
License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

PDIII-2 JStevens/km
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
December 21, 1988

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Acting Manager - Licensing and Safety
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Sincerely,

A handwritten signature in cursive script that reads "Janice A. Stevens".

Janice A. Stevens, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

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License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Dale L. Holtzscher
Illinois Power Company

Clinton Power Station
Unit 1

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Illinois Power Company* (IP), Soyland Power Cooperative, Inc., and Western Illinois Power Cooperative, Inc. (the licensees) dated October 30, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and Western Illinois Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

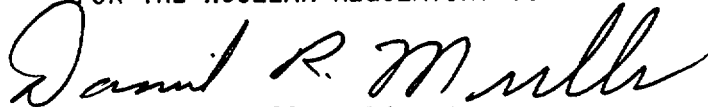
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Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 14 , are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "Daniel R. Muller", is written over the typed name.

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 21, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 14

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

3/4 6-42
3/4 6-43
3/4 6-44
3/4 6-45
3/4 6-46
3/4 6-47
3/4 6-48
3/4 6-49
3/4 6-50
3/4 6-51
3/4 6-61

Insert

3/4 6-42
3/4 6-43
3/4 6-44
3/4 6-45
3/4 6-46
3/4 6-47
3/4 6-48
3/4 6-49
3/4 6-50
3/4 6-51
3/4 6-61

TABLE 3.6.4-1 (Continued)
CONTAINMENT ISOLATION VALVES

	<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
	3. <u>Test Connections, Vents, and Drains</u> ^(a)						
	1) Main Steam Line C 1B21-F025C 1E32-F327C 1E32-F330A	5	NA	1, 2, 3	NA	No	9.0
	2) Main Steam Line A 1B21-F025A 1E32-F327A 1E32-F329A	6	NA	1, 2, 3	NA	No	9.0
	3) Main Steam Line D 1B21-F025D 1E32-F327D 1E32-F330C	7	NA	1, 2, 3	NA	No	9.0
	4) Main Steam Line B 1B21-F025B 1E32-F327B 1E32-F329C	8	NA	1, 2, 3	NA	No	9.0
	5) Feedwater/RHR Line A 1B21-F063A 1B21-F030A 1E12-F058A 1E12-F349A	9	NA	1, 2, 3, 1, 2, 3 1, 2, 3 1, 2, 3	NA	Yes No No No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1

3/4 6-43

Amendment No. 14

<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
<u>Test Connections, Vents, and Drains</u> ^(a) (Continued)						
6) Feedwater/RHR Line B 1B21-F063B 1B21-F030B 1E12-F058B 1E12-F349B 1G33-F057	10	NA	1, 2, 3, # 1, 2, 3 1, 2, 3 1, 2, 3 1, 2, 3	NA	Yes No No No No	9.0
7) RHR A Suction 1E12-F334A 1E12-F335A	11	NA	1, 2, 3	NA	No	9.9
8) RHR B Suction 1E12-F334B 1E12-F335B	12	NA	1, 2, 3	NA	No	9.9
9) RHR C Suction 1E12-F334C 1E12-F335C	13	NA	1, 2, 3	NA	No	9.9
10) RHR Shutdown Cooling 1E12-F001	14	NA	1, 2, 3	NA	No	9.0
11) RHR/LPCI A Injection 1E12-F107A 1E12-F331A 1E12-F329A	15	NA	1, 2, 3	NA	No	9.0
12) RHR/LPCI B Injection 1E12-F107B 1E12-F331B 1E12-F329B	16	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)
CONTAINMENT ISOLATION VALVES

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
<u>Test Connections, Vents and Drains</u> ^(a) (Continued)						
13) RHR/LPCI C Injection 1E12-F056C 1E12-F057C	17	NA	1, 2, 3	NA	No	9.0
14) RHR A Test Line 1E12-F365A 1E12-F366A 1E21-F346 1E21-F347 1E12-F414 1E12-F415 1E12-F418 1E12-F419 1E12-F420 1E12-F421	18	NA	1, 2, 3	NA	No	9.0
15) RHR C Test Line 1E12-F353 1E12-F354 1E12-F428 1E12-F429	19	NA	1, 2, 3	NA	No	9.0
16) RHR B Test Line 1E12-F365B 1E12-F366B 1E12-F426 1E12-F427	20	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1

3/4 6-45

Amendment No. 14

	<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
	<u>Test Connections, Vents and Drains</u>		(a)	(Continued)			
17)	RHR HX 1E12-F432A 1E12-F433A	24	NA	1, 2, 3	NA	No	9.0
18)	RHR HX 1E12-F432B 1E12-F433B	26	NA	1, 2, 3	NA	No	9.0
19)	RCIC Pump Suction 1E51-F336 1E51-F337	28	NA	1, 2, 3	NA	No	9.9
20)	RCIC Suction Release Discharge 1E12-F436 1E12-F437	31	NA	1, 2, 3	NA	No	9.9
21)	LPCS Pump Suction 1E21-F331 1E21-F344	32	NA	1, 2, 3	NA	No	9.9
22)	HPCS Test To Supp. Pool 1E22-F376	33	NA	1, 2, 3	NA	No	9.9
23)	Supp. Pool Cleanup Pump Suction 1SF034	34	NA	1, 2, 3	NA	No	9.9

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1

3/4 6-46

Amendment No. 14

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
<u>Test Connections, Vents and Drains</u> ^(a) (Continued)						
24) HPCS Pump Discharge 1E22-F021 1E22-F022	35	NA	1, 2, 3	NA	No	9.0
25) LPCS Pump Discharge 1E21-F013 1E21-F014	36	NA	1, 2, 3	NA	No	9.0
26) RCIC 1E51-F041	41	NA	1, 2, 3	NA	No	9.0
27) Head Spray 1E51-F034 1E51-F035 1E51-F390 1E51-F391 1E12-F061 1E12-F062	42	NA	1, 2, 3	NA	No	9.0
28) RCIC Turb Steam Supply 1E51-F399 1E51-F072 1E51-F401	43	NA	1, 2, 3	NA	No	9.0
29) RCIC Turb Vacuum Breaker 1E51-F080 1E51-F082 1E51-F375 1E51-F376 1E51-F083	44	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1

3/4 6-47

Amendment No. 14

<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
<u>Test Connections, Vents and Drains</u> ^(a) (Continued)						
30) Main Steam Drain Line 1B21-F017	45	NA	1, 2, 3	NA	No	9.0
31) CCW Supply 1CC164 1CC266	46	NA	1, 2, 3	NA	No Yes	9.0
32) CCW Return 1CC165	47	NA	1, 2, 3	NA	No	9.0
33) Makeup Condensate 1MC011	50	NA	1, 2, 3	NA	No	9.0
34) Fuel Pool Cool/Cleanup Supply 1FC092	52	NA	1, 2, 3	NA	No	9.0
35) Fuel Pool Cool/Cleanup Return 1FC093	53	NA	1, 2, 3	NA	No	9.0
36) Fire Protection 1FP127	56	NA	1, 2, 3	NA	No	9.0
37) Instrument Air 1IA039	57	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1

3/4 6-48

Amendment No. 14

<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
<u>Test Connections, Vents and Drains^(a)</u> (Continued)						
38) Service Air Line 1SA046	59	NA	1, 2, 3	NA	No	9.0
39) RWCU Pump Suction 1G33-F002	60	NA	1, 2, 3	NA	No	9.0
40) RWCU Return 1G33-F061	61	NA	1, 2, 3	NA	No	9.0
41) Hydrogen Recombiner 1HG019	62	NA	1, 2, 3	NA	No	9.0
42) CRD Pump Discharge 1C11-F128	63	NA	1, 2, 3	NA	No	9.0
43) RWCU Return 1G33-F055	64	NA	1, 2, 3	NA	No	9.0
44) Containment Pressurization (test penet.) 1SA129	67	NA	1, 2, 3	NA	No	9.0
45) Hydrogen Recombiner 1HG016 1HG020	71	NA	1, 2, 3	NA	No	9.0
46) Hydrogen Recombiner 1HG017 1HG021	72	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1

3/4 6-49

Amendment No. 14

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
<u>Test Connections, Vents and Drains</u> ^(a) (Continued)						
47) SX To Recir. Pump 1CC170	78	NA	1, 2, 3	NA	No	9.0
48) Supp. Pool Cleanup Return 1SF023	79	NA	1, 2, 3	NA	No	9.0
49) Fire Protection 1FP129	81	NA	1, 2, 3	NA	No	9.0
50) Fire Protection 1FP128	82	NA	1, 2, 3	NA	No	9.0
51) Cycle Condensate 1CY019	85	NA	1, 2, 3	NA	No	9.0
52) RWCU Letdown 1G33-F070	86	NA	1, 2, 3	NA	No	9.0
53) SX From Recir. Pump 1CC171	88	NA	1, 2, 3	NA	No	9.0
54) Containment HVAC Supply 1VR003	101	NA	1, 2, 3	NA	No	9.0
55) Containment HVAC Return 1VQ007	102	NA	1, 2, 3	NA	No	9.0
56) Containment HVAC 1VR011	106	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1

3/4 6-50

Amendment No. 14

VALVE NUMBER	PENETRATION NUMBER	ISOLATION SIGNAL†	APPLICABLE OPERATIONAL CONDITIONS	MAXIMUM ISOLATION TIME (Seconds)	SECONDARY CONTAINMENT BYPASS PATH (YES/NO)	TEST PRESSURE (psig)*
<u>Test Connections, Vents and Drains</u> ^(a) (Continued)						
57) Drywell Chilled Water 1VP044B 1VP077D	107	NA	1, 2, 3	NA	No	9.0
58) Drywell Chilled Water 1VP047B 1VP077B	108	NA	1, 2, 3	NA	No	9.0
59) Drywell Chilled Water 1VP044A 1VP077C	109	NA	1, 2, 3	NA	No	9.0
60) Drywell Chilled Water 1VP047A 1VP077A	110	NA	1, 2, 3	NA	No	NA
61) Containment HVAC 1VR012	113	NA	1, 2, 3	NA	No	9.0
62) Standby Liquid Control 1C41-F340B 1C41-F341B	116	NA	1, 2, 3	NA	No	9.0
63) Hydrogen Recombiner 1HG018	166	NA	1, 2, 3	NA	No	9.0

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

CLINTON - UNIT 1	VALVE	PENETRATION	ISOLATION	APPLICABLE	MAXIMUM	SECONDARY	TEST
	NUMBER	NUMBER	SIGNAL†	OPERATIONAL	ISOLATION	CONTAINMENT	PRESSURE
				CONDITIONS	TIME	BYPASS PATH	(psig)*
					(Seconds)	(YES/NO)	
3/4 6-51	<u>Test Connections, Vents and Drains</u> ^(a) (Continued)						
	64) Drywell Pressure		NA	1, 2, 3	NA	No	9.0
	1CM076	151					
	1CM077	203					
	65) Reactor Pressure	151	NA	1, 2, 3	NA	No	9.0
	1CM072						
	1CM073						
	66) Reactor Pressure	160	NA	1, 2, 3	NA	No	9.0
	1CM074						
	1CM075						
Amendment No. 14	67) Equipment Hatch	1	NA	1, 2, 3	NA	No	9.0
	1CM099						
	68) Suppression Pool Level	177	NA	1, 2, 3	NA	No	9.0
	1E51 - F437A ^(h)						
	1E51 - F437B ^(h)						
	69) Suppression Pool Level	179	NA	1, 2, 3	NA	No	9.0
	1E22 - F381A ^(h)						
	1E22 - F381B ^(h)						
	1SM027A ^(h)						
	1SM027B ^(h)						
	70) Suppression Pool Level	181	NA	1, 2, 3	NA	No	9.0
	1SM026A ^(h)						
	1SM026B ^(h)						
	71) Suppression Pool Level	183	NA	1, 2, 3	NA	No	9.0
	1CM100A ^(h)						
	1CM100B ^(h)						

TABLE 3.6.4-1 (Continued)
CONTAINMENT ISOLATION VALVES

TABLE NOTATIONS

- (a) May be opened on an intermittent basis under administrative control during applicable OPERATIONAL CONDITIONS.
- (b) Excess flow check valve actuation differential pressure.
- (c) Isolation valving for instrument lines which penetrate the containment conform to the requirements of NRC Regulatory Guide 1.11. The in-service inspection program will provide assurance of the operability and integrity of these isolation provisions. Type "C" testing will not be performed on the instrument line isolation valves. The instrument lines will be within the boundaries of the Type "A" test, open to the media (containment atmosphere or suppression pool water) to which they will be exposed under postulated accident conditions. Instrument taps from the process line located between the process isolation valves and the penetration, and not themselves penetrating containment, will be Type "A" and/or "C" tested along with the process line isolation valves.
- (d) Excess flow check valve.
- (e) The RHR system may be operating in the shutdown cooling mode during the Type A test. These valves are tested using water but the results are not required to be added to the Type A test results. The LPCS, HPCS, and RHR may be aligned in the normal standby or injection mode during the Type A test. This will expose the closed loop outside containment to containment pressure through the suppression pool. This is the closest valve alignment to the post-LOCA alignment possible. Type C water test results on these suction valves will not be added to the Type A test results.
- (f) Valves shall be closed in accordance with SECONDARY CONTAINMENT INTEGRITY.
- (g) Valves shall be "sealed closed" by utilizing mechanical devices to seal or lock the valve closed or to prevent power from being supplied to the valve operator.
- (h) OPERABILITY of these valves is not required until completion of corresponding plant modification.
- # When handling irradiated fuel in secondary containment and during CORE ALTERATIONS and operations with a potential for draining the reactor vessel.
- ## Isolates on RCIC low steam line pressure only.
- † Isolation signal descriptions are provided in Table 3.6.4-2.
- * For test pressure = 9.0 psig, the valve(s) shall be pressurized using air or nitrogen, and for test pressure = 9.9 psig, the valve(s) shall be pressurized using water.
- ** With any control rod withdrawn. Not applicable to any control rods removed per Specification 3.9.10.1 or 3.9.10.2.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 14 TO FACILITY OPERATING LICENSE NO. NPF-62
CLINTON POWER STATION, UNIT NO. 1
ILLINOIS POWER COMPANY, ET AL.
DOCKET NO. 50-461

1.0 INTRODUCTION

By letter dated October 30, 1987, the Illinois Power Company, et al. (the licensees) proposed changing the Technical Specifications (TS) for the Clinton Power Station to correct typographical errors and clarify existing requirements regarding Note (a) in Table 3.6.4-1.

A Notice of Consideration of Issuance of Amendment to License and Proposed No Significant Hazards Consideration Determination and Opportunity For Hearing related to the requested action was published in the Federal Register on January 27, 1988 (53 FR 2319). No requests for hearing and no public comments were received.

2.0 EVALUATION

The licensees propose to correct typographical errors regarding Note (a) in Technical Specification 3.6.4, Table 3.6.4-1, "Containment Isolation Valves," Section 3, Test Connections, Vents and Drains, pages 3/4 6-42 through 3/4 6-51. Additionally, the licensees propose to clarify existing requirements regarding Note (a) on page 3/4 6-61. The licensees propose to make the following changes:

- (1) Note (a) as it appears in the "Applicable Operational Conditions" column of Table 3.6.4-1 (Containment Isolation Valves) would be relocated on pages 3/4 6-42 through 3/4 6-51 to appear next to the heading "Test Connections, Vents, and Drains."
- (2) The note as written on page 3/4 6-61 under Table Notations would be modified to include, "during applicable OPERATIONAL CONDITIONS."
- (3) The typographical error on page 3/4 6-42 would be changed from Note (b) to Note (a).

The way that the "a" (reference to Note (a)) is located now on each of the above pages in Table 3.6.4-1 makes it appear as if note (a) is applicable only to the last OPERATIONAL CONDITION in the list for each valve, rather than all of the listed conditions. This is a typographical error carried over from corrections that had been made to the draft Technical Specifications. The intent of note (a) is to allow the applicable valves to be opened on an intermittent basis

under administrative control during all of the applicable OPERATIONAL CONDITIONS. Rather than attach "(a)" to every applicable OPERATIONAL CONDITION number or symbol listed for every applicable valve, the "(a)" will be moved to the valve heading since note (a) is applicable to all of the test connection, vent, and drain valves and their corresponding applicable OPERATIONAL CONDITIONS listed in Table 3.6.4-1.

To ensure that there is no doubt that the note is applicable during any and all of the applicable OPERATIONAL CONDITIONS for the appropriate valves, the wording of the note itself on page 3/4 6-61 is being modified to read: "May be opened on an intermittent basis under administrative control during applicable OPERATIONAL CONDITIONS."

On page 3/4 6-42 a "(b)" is attached to the heading "Test Connections, Vents, and Drains" where it is intended to insert "(a)". This is a typographical error in that note (b) deals with excess flow check valves and not test connection, vent, or drain valves. Therefore, this "(b)" would be replaced by the correct "(a)".

Based on the above, the staff finds the changes to Technical Specifications 3.6.4, Table 3.6.4-1, Section 3 acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to the changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributor: Roger Mendez, Region III

Dated: December 21, 1988