

December 29, 1988

Docket No.: 50-461

Mr. Dale L. Holtzscher
Acting Manager - Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

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Dear Mr. Holtzscher:

SUBJECT: TECHNICAL SPECIFICATION CHANGE REQUEST TO CORRECT TYPOGRAPHICAL ERRORS (TAC NO. 69553)

Re: Clinton Power Station, Unit No. 1

The Commission has issued the enclosed Amendment No. 15 to the Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated October 30, 1987.

This amendment revises Technical Specification Sections 3.4.1.1 and 4.6.6.3 and Tables 3.3.2-1, 3.3.7.4-1 and 3.6.4-1 in order to correct typographical errors.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Janice A. Stevens, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No.15 to License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555
December 29, 1988

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Mr. Dale L. Holtzscher
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Clinton Power Station
Unit 1

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Illinois Power Company* (IP), Soyland Power Cooperative, Inc., and Western Illinois Power Cooperative, Inc. (the licensees) dated October 30, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and Western Illinois Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

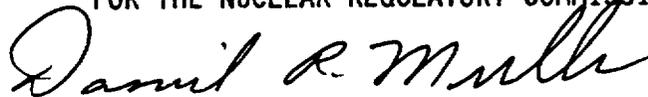
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Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No.15 , are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 29, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

3/4 3-16

3/4 3-17

3/4 3-82

3/4 6-33

3/4 6-71

3/4 4-2

Insert

3/4 3-16

3/4 3-17

3/4 3-82

3/4 6-33

3/4 6-71

3/4 4-2

TABLE 3.3.2-1 (Continued)

CRVICS INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ††</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
4. <u>REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION (Continued)</u>				
e. RCIC Equipment Room Ambient Temp. - High	V	1 ^(k)	1, 2, 3	27
f. RCIC Equipment Room Δ Temp. - High	V	1 ^(k)	1, 2, 3	27
g. Main Steam Line Tunnel Ambient Temp. - High	E	1 ^(k)	1, 2, 3	27
h. Main Steam Line Tunnel Δ Temp. - High	F	1 ^(k)	1, 2, 3	27
i. Main Steam Line Tunnel Temperature Timer	X	1 ^(k)	1, 2, 3	27
j. Drywell Pressure - High	L ^(h)	1 ^(k)	1, 2, 3	27
k. Manual Initiation	R	1 ⁽ⁱ⁾	1, 2, 3	26
l. RHR/RCIC Steam Line Flow - High	V	1 ^(k)	1, 2, 3	27
m. RHR Heat Exchanger A, B Ambient Temperature - High	T	1/room ^(k)	1, 2, 3	28
n. RHR Heat Exchanger A, B Δ Temp. - High	S	1/room ^(k)	1, 2, 3	28

TABLE 3.3.2-1 (Continued)

CRVICS INSTRUMENTATION

<u>TRIP FUNCTION</u>	<u>ISOLATION SIGNAL ††</u>	<u>MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM</u>	<u>APPLICABLE OPERATIONAL CONDITION</u>	<u>ACTION</u>
5. <u>RHR SYSTEM ISOLATION</u>				
a. RHR Heat Exchanger A, B Ambient Temperature - High	T	1/room ^(k)	1, 2, 3	28
b. RHR Heat Exchanger A, B ΔTemp. - High	S	1/room ^(k)	1, 2, 3	28
c. Reactor Vessel Water Level - Low, Level 3	A	2 ^(a)	1, 2, 3	28
d. Reactor Vessel Water Level - Low Low Low, Level 1	U	2 ^(k)	1, 2, 3	28
e. Reactor Vessel (RHR Cut-in Permissive) Pressure - High	X	2 ^(a)	1, 2, 3	28
f. Drywell Pressure - High				
1) RHR Test Lines	L	2 ^(k)	1, 2, 3	28
2) Fuel Pool Cooling	L	2 ^(a)	1, 2, 3	28
g. Manual Initiation	R	1	1, 2, 3	26

TABLE 3.3.7.4-1

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

INSTRUMENT	DIVISION I		DIVISION II	
	<u>EQUIPMENT NUMBER</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>EQUIPMENT NUMBER</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. SRV 51D Temp., Supp. Pool Temp.	1C61-R506	1	1C61-R513	1
2. SRV 51C Temp., Supp. Pool Temp.	1C61-R507	1	1C61-R514	1
3. SRV 51G Temp., Supp. Pool Temp.	1C61-R508	1	1C61-R512	1
4. Supp. Pool. Lvl.	1C61-R504	1	1C61-R511	1
5. RPV Lvl.	1C61-R010	1	1C61-R509	1
6. RPV Press.	1C61-R011	1	1C61-R510	1
7. Upper DW Temp.	1C61-R501	1		NA
8. Lower DW Temp.	1C61-R502	1		NA
9. SX Strnr. Dsch. Outlet Press.	1C61-R503	1	1PI-SX024B	1
10. RCIC Cond. Storage Tnk. Lvl.	1C61-R505	1		NA
11. RHR Loop Flow	1C61-R005	1	1E12-R008B*	1
12. RCIC Turb. Speed	1C61-R003	1		NA
13. RCIC Pump Flow	1C61-R001	1		NA
14. RCIC Turb. Flow Cntl.	1C61-R001	1		NA

*Division II RHR pump flow is determined by RHR pump discharge pressure instrumentation at panel 1H22-P021.

TABLE 3.6.4-1 (Continued)

CONTAINMENT ISOLATION VALVES

	<u>VALVE NUMBER</u>	<u>PENETRATION NUMBER</u>	<u>ISOLATION SIGNAL†</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>MAXIMUM ISOLATION TIME (Seconds)</u>	<u>SECONDARY CONTAINMENT BYPASS PATH (YES/NO)</u>	<u>TEST PRESSURE (psig)*</u>
<u>Automatic Isolation Valves (Continued)</u>							
14)	HPCS Test Line 1E22-F023	33	B, L	1, 2, 3	68	No	9.9
15)	Supp. Pool Cleanup Suction 1SF004	34	B, L, R	1, 2, 3	84	Yes	9.9
16)	RCIC 1E51-F077	41	L, V##	1, 2, 3	21	No	9.0
17)	RHR Head Spray 1E12-F023	42	A, S, T, X, R	1, 2, 3	39	No	9.0
18)	RCIC Steam Supply 1E51-F063 1E51-F064 1E51-F076	43	V, S, T, E, F, X V, S, T, R††, B††, E, F, X V, S, T, E, F, X	1, 2, 3	41 41 8	No	9.0
19)	RCIC Turb Vac Bkr Line 1E51-F078	44	L, V, ##	1, 2, 3	27	No	9.0
20)	Main Steam Drain Line 1B21-F016 1B21-F019	45	C, D, E, G, H, J, U, X, F, R C, D, E, G, H, J, U, X, F, R	1, 2, 3, #(f)	26 26	Yes	9.0

††A single manual isolation switch (R) isolates outboard steam supply line isolation valve (F064) and the RCIC pump suction from suppression pool valve (F031) only following a manual or automatic Reactor Vessel Water Level 2(B) RCIC initiation.

CONTAINMENT SYSTEMS

STANDBY GAS TREATMENT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.6.6.3 (Continued)

1. Verifying that the subsystem satisfies the in-place penetration and bypass leakage testing acceptance criteria of less than 0.05% and uses the test procedure guidance in Regulatory Positions C.5.a, C.5.c and C.5.d of Regulatory Guide 1.52, Revision 2, March 1978*, and the system flow rate is 4000 cfm \pm 10%.

 2. Verifying, within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978*, for a methyl iodide penetration of less than 0.175%; when tested in accordance with ASTM D3803-79 methods, with the following parameters:
 - a) Bed Depth - 4 inches
 - b) Velocity - 40 fpm
 - c) Temperature - 80°C
 - d) Relative Humidity - 70%and

 3. Verifying a subsystem flow rate of 4000 cfm \pm 10% during system operation when tested in accordance with ANSI N510-1980.
- c. After every 720 hours of charcoal adsorber operation, by verifying, within 31 days after removal, that a laboratory analysis of a representative carbon sample obtained in accordance with Regulatory Position C.6.b of Regulatory Guide 1.52, Revision 2, March 1978*, meets the laboratory testing criteria of Regulatory Position C.6.a of Regulatory Guide 1.52, Revision 2, March 1978*, for a methyl iodide penetration of less than 0.175%; in accordance with ASTM D3803-79 methods, with the following parameters:
- a) Bed Depth - 4 inches
 - b) Velocity - 40 fpm
 - c) Temperature - 30°C
 - d) Relative Humidity - 70%

*ANSI N510-1980 shall be used in place of ANSI N510-1975 as referenced in Regulatory Guide 1.52, Revision 2, March 1978.

3/4.4 REACTOR COOLANT SYSTEM

3/4.4.1 RECIRCULATION SYSTEM

RECIRCULATION LOOPS

LIMITING CONDITION FOR OPERATION

- f) Reduce the volumetric flow rate of the operating recirculation loop to $\leq 33,000$ gpm*,
 - g) Perform Surveillance Requirement 4.4.1.1.4 if thermal power is $\leq 30\%^{**}$ of RATED THERMAL POWER or the recirculation loop flow in the operating loop is $\leq 50\%^{**}$ of rated loop flow.
- 2. The provisions of Specification 3.0.4 are not applicable.
 - 3. Otherwise, place the unit in HOT SHUTDOWN within 12 hours.
- b. With no reactor coolant system recirculation loops in operation, immediately initiate action to reduce THERMAL POWER so that it is in the unrestricted zone of Figure 3.4.1.1-1 within 4 hours and initiate measures to place the unit in at least STARTUP within 6 hours and in HOT SHUTDOWN within the next 6 hours.
 - c. With one or two reactor coolant system recirculation loops in operation and total core flow less than 45% but greater than $(39)\%^{\#}$ of rated core flow and THERMAL POWER within the restricted zone of Figure 3.4.1.1-1, and with the APRM or LPRM† neutron flux noise levels greater than three times their established baseline noise levels, immediately initiate corrective action to restore the noise levels to within the required limits within 2 hours by increasing core flow or by reducing THERMAL POWER.
 - d. With one or two reactor coolant recirculation loops in operation, and total core flow less than or equal to $(39)\%^{\#}$, and THERMAL POWER within the restricted zone of Figure 3.4.1.1-1, within 15 minutes initiate corrective action to reduce THERMAL POWER to within the unrestricted zone of Figure 3.4.1.1-1, or increase core flow to greater than $(39)\%^{\#}$ within 4 hours.

*This value represents the design volumetric recirculation loop flow which produces 100% core flow at 100% THERMAL POWER. The actual value to be applied will be determined during the Startup Test Program.

**Initial values. Final values to be determined during Startup Testing based upon the threshold THERMAL POWER and recirculation loop flow which will sweep the cold water from the vessel bottom head preventing stratification.

#Value to be established during Startup Test Program. (Core flow with both recirculation pumps at rated speed and minimum control valve position.)

†Detector levels A and C of one LPRM string per core octant plus detectors A and C of one LPRM string in the center of the core should be monitored.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE NO. NPF-62

CLINTON POWER STATION, UNIT NO. 1

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

1.0 INTRODUCTION

By letter dated October 30, 1987, the Illinois Power Company, et al. (the licensees) proposed changing the Technical Specifications (TS) for the Clinton Power Station to correct typographical errors.

A notice of consideration of issuance of amendment to license and proposed no significant hazards consideration determination and opportunity for hearing related to the requested action was published in the Federal Register on January 27, 1988 (53 FR 2319). No requests for hearing and no public comments were received.

2.0 EVALUATION

The licensees proposed to make several changes in the Technical Specifications to correct typographical errors. The following changes were requested:

- (1) On page 3/4 3-16 and 3/4 3-17, Table 3.3.2.1, the "(a)" that appears next to the MINIMUM OPERABLE CHANNELS PER TRIP SYSTEM heading would be deleted, as it does not apply to all the items listed in the table.
- (2) On page 3/4 3-82, Table 3.3.7.4-1, Item 7 and 8 shall read 1C61-R501 and 1C61-R502, respectively, to match the instrumentation labels in the plant.
- (3) ACTION a.1.g. associated with LIMITING CONDITION FOR OPERATION 3.4.1.1 on page 3/4 4-2 would be changed from "4.4.1.1.2" to "4.4.1.1.4." The surveillance requirement number was changed with the issuance of a previous amendment, but this associated reference was not.
- (4) Delete the third "41" listed for Item 18 under MAXIMUM ISOLATION TIME on page 3/4 6-33, Table 3.6.4-1, as there is no additional valve listed.
- (5) Delete the heading "Make Up Filter System," on page 3/4 6-71, under Surveillance Requirement 4.6.6.3.C, because these words are not applicable. The Standby Gas Treatment System only operates in the exhaust mode and does not contain a make-up system.

The staff reviewed the changes proposed by the licensees. The changes correct typographical errors or delete phrases which are not applicable. The staff finds the licensees' proposed Technical Specification changes acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to the changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributor: Roger Mendez, Region III

Dated: December 29, 1988