

Docket
File



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 30, 1990

Docket No. 50-461

Mr. Dale L. Holtzscher
Acting Manager - Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

Dear Mr. Holtzscher:

SUBJECT: TECHNICAL SPECIFICATION CHANGE REQUEST CONCERNING ACCIDENT
MONITORING INSTRUMENTATION AND TIME-LIMIT VALUES SPECIFIED
FOR TIMES - CLINTON POWER STATION, UNIT NO. 1 (TAC NO. 66553)

The Commission has issued the enclosed Amendment No. 29 to the Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment consists of changes to the Technical Specifications in response to your application dated October 30, 1987.

This amendment revises Technical Specification Table 3.3.7.5-1 in order to insert exceptions to Specification 3.0.4 into the ACTION statements and to correct a typographical error. Three proposed changes to Technical Specification Table 3.3.2-2 concerning time-limit values specified for timers, one proposed change in order to resolve an inconsistency, and one proposed change in order to delete the safety/relief valve acoustic monitors from the Accident Monitoring Instrumentation have been found to be unacceptable on the basis that they are not in conformance with the Standard Technical Specifications and you have not submitted sufficient technical justification to show that these changes are necessary and acceptable. However, due to the generic nature of these proposed changes, you have the option to resubmit these proposed changes, after conferring with the BWR Owners Group, for NRC evaluation in accordance with the staff policy for generic Technical Specification changes

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and revisions to standard Technical Specifications. A copy of our Safety Evaluation is also enclosed. The Notice of Issuance and Notice of Partial Denial are being filed with the Office of the Federal Register for publication.

Sincerely,

/s/

John B. Hickman, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 29 to License No. NPF-62
2. Safety Evaluation
3. Notice of Issuance
4. Notice of Denial

cc w/enclosures:

See next page

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*See previous concurrence

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Mr. Dale L. Holtzscher
Illinois Power Company

Clinton Power Station
Unit 1

cc:

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c/o County Clerk's Office
DeWitt County Courthouse
Clinton, Illinois 61727



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 29
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Illinois Power Company* (IP), Soyland Power Cooperative, Inc., and Western Illinois Power Cooperative, Inc. (the licensees) dated October 30, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and Western Illinois Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

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Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 29, are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John W. Craig, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 30, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 29

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 3-87

3/4 3-88

Insert

3/4 3-87

3/4 3-88

TABLE 3.3.7.5-1
ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
1. Reactor Vessel Pressure	2	1	1, 2, 3	80
2. Reactor Vessel Water Level	2	1	1, 2, 3	80
3. Suppression Pool Water Level	4	2	1, 2, 3	80
4. Suppression Pool Water Temperature	2/quadrant†	1/quadrant†	1, 2, 3	80
5. Drywell Pressure	2	1	1, 2, 3	80
6. Drywell Air Temperature	2	1	1, 2, 3	80
7. Drywell/Containment Hydrogen and Oxygen Concentration Analyzer and Monitor	2	1	1, 2, 3	80
8. Containment Pressure ##	2/division	1/division	1, 2, 3	80
9. Containment Temperature	2	1	1, 2, 3	80
10. Safety/Relief Valve Acoustic Monitor	1/valve***	1/valve***	1, 2, 3	80
11. Containment/Drywell High Range Gross Gamma Radiation Monitors	4**	2*	1, 2, 3	81
12. HVAC Stack High Range Radioactivity Monitor#	1	1	1, 2, 3	81
13. SGTS Exhaust High Range Radioactivity Monitor#	1	1	1, 2, 3	81
14. Primary Containment Isolation Valve Position Indication ††	2/valve	1/valve	1, 2, 3	82

TABLE NOTATIONS

* One each for containment and drywell.

** Two each for containment and drywell.

*** Thermocouples in the SRV discharge line can serve as backup to the acoustic tail pipe monitors indication should one channel of the position indication become inoperable.

High range noble gas monitors and iodine/particulate sampler.

For Divisions I and II only.

† These instruments monitor suppression pool water temperature when pool water level is below instruments of Specification 3.6.3.1.

†† One channel consists of the open limit switch, and the other channel consists of the closed limit switch for each automatic isolation valve in Table 3.6.4-1 Part 1, "Automatic Isolation Valves."

TABLE 3.3.7.5-1 (Continued)
ACCIDENT MONITORING INSTRUMENTATION

ACTION

- ACTION 80 -
- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours. The provisions of Specification 3.0.4 are not applicable.
 - b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- ACTION 81 -
- With the number of OPERABLE Channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable Channel(s) to OPERABLE status within 72 hours, or:
- a. Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
 - b. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
 - c. The provisions of Specification 3.0.4 are not applicable.
- ACTION 82 -
- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, verify the valve(s) position by use of alternate indication methods; restore the inoperable channel(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, verify the valve(s) position by use of alternate indication methods; restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - c. The provisions of Specification 3.0.4 are not applicable.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 29 TO FACILITY OPERATING LICENSE NO. NPF-62
CLINTON POWER STATION, UNIT NO. 1
ILLINOIS POWER COMPANY, ET. AL.
DOCKET NO. 50-461

1.0 INTRODUCTION

By letter dated October 30, 1987, the licensee for Clinton Unit 1 proposed various changes to the plant's Technical Specifications. The Instrumentation and Control Systems Branch has reviewed the requested changes in our area of review and finds two acceptable and three unacceptable as discussed below.

2.0 EVALUATION

Licensee Package Number 6, Change 2, proposed to refer to Specification 3.6.3.1 in lieu of Specification 3.5.3.1 in Note . under "TABLE NOTATIONS" on page 3/4 3-87. We find this change acceptable on the basis that it is purely an editorial change necessary to correct a typographical error.

Licensee's Package Number 6, Change 1, proposed to remove the applicability of the provisions of Specification 3.0.4 in action statements for accident monitoring instrumentation contained in Table 3.3.7.5-1 to permit entry into Operational Conditions 1, 2, or 3 with inoperable channels and while in appropriate action statements. This change consists of revisions to Table 3.3.7.5-1 in order to insert exceptions to Specification 3.0.4 into associated ACTION statements. These exceptions would permit entry into OPERATIONAL CONDITIONS 1, 2, and 3, with an accident monitoring instrumentation channel(s) inoperable, as provided in the individual ACTION statements. Although the instrumentation is required to be OPERABLE in OPERATIONAL CONDITIONS 1, 2 and 3, the instrumentation is designed and intended to be used to "assess plant and environs conditions during and following an accident" (Ref. Regulatory Guide 1.97). Thus, entry into Conditions 1, 2 and 3 does not necessarily correspond to entry into the conditions for which the accident monitoring instrumentation will be needed. Conditions 1, 2 and 3 do correspond to the OPERATIONAL CONDITIONS in which a design basis accident is most likely to be initiated. Thus, entry into CONDITIONS 1, 2 and 3 should be permitted with the number of OPERABLE channels less than the REQUIRED NUMBER OF CHANNELS requirement, but not with the number of OPERABLE channels less than the MINIMUM CHANNELS OPERABLE requirement (except when the ACTION provides compensatory measures). Exceptions to Specification 3.0.4 would therefore be inserted into ACTIONS 80, 81 and 82. We find this change acceptable on the basis that it is consistent with staff policy for instrumentation which does not initiate automatic mitigation functions and with current standard technical specifications.

Licensee's Package Number 4 proposed to change the specific entries and format for trip setpoints and allowable values for timers in Table 3.3.2-2.

Licensee's Package Number 6, Change 3, proposed to revise the action statement applicable to containment/drywell, HVAC stack and SGTS exhaust high range radiation monitors in Table 3.3.7.5-1. Licensee's Package Number 6, Change 4, proposed to delete safety/relief valve acoustic monitors from Tables 3.3.7.5-1 and 4.3.7.5-1. We find these proposed changes to be unacceptable at this time on the basis that they are not in conformance with the Standard Technical Specifications and the licensee has not submitted sufficient technical justification to show that these changes are necessary and acceptable. Due to the generic nature of these proposed changes, they should be evaluated in accordance with the staff policy for generic technical specification changes and revisions to standard technical specifications.

Based on the above evaluation, the staff concludes that two of the proposed changes concerning accident monitoring instrumentation are acceptable. The bases for the staff's acceptance are that the proposed changes (1) do not remove any existing requirements while providing adequate clarification and (2) meet the appropriate guidelines of the Generic Electric Standard TS (NUREG-0123, Revision 3, dated December 1980).

3.0 ENVIRONMENTAL CONSIDERATION

Pursuant to 10 CFR 51.21, 51.32, and 51.35, an environmental assessment and finding of no significant impact have been prepared and published (55 FR 2717) in the Federal Register on January 26, 1990. Accordingly, based upon the environmental assessment, the Commission has determined that the issuance of this amendment will not have a significant effect on the quality of the human environment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety to the public.

Principal Contributor: F. Burrows, SICB/DEST

Dated: January 30, 1990

January 19, 1990

Docket No. 50-461

Mr. Dale L. Holtzscher
Acting Manager - Licensing & Safety
Clinton Power Station
Post Office Box 678
Mail Code V920
Clinton, Illinois 61727

Dear Mr. Holtzcher:

SUBJECT: ENVIRONMENTAL ASSESSMENT AND FINDING OF NO SIGNIFICANT IMPACT -
TECHNICAL SPECIFICATION CHANGES CONCERNING ACCIDENT MONITORING
INSTRUMENTATION AND TIME-LIMIT VALUES - CLINTON POWER STATION,
UNIT 1 (TAC NO. 66553)

Pursuant to 10 CFR 51.119, the Commission has requested the Office of the
Federal Register to publish the enclosed "Environmental Assessment and Finding
of No Significant Impact." This notice is in regard to your request dated
October 30, 1987 for changes to the Technical Specifications concerning
accident monitoring instrumentation and time-limit values.

/s/

John B. Hickman, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosure:
As stated

cc w/enclosure:

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UNITED STATES NUCLEAR REGULATORY COMMISSIONILLINOIS POWER COMPANY, ET AL.DOCKET NO. 50-461ENVIRONMENTAL ASSESSMENT AND FINDING OFNO SIGNIFICANT IMPACT

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to the Illinois Power Company (IP), and Soyland Power Cooperative, Inc. (the licensees) for Clinton Power Station, Unit 1, located in DeWitt County, Illinois.

ENVIRONMENTAL ASSESSMENTIdentification of Proposed Action:

In general, the proposed license amendment would revise the Technical Specifications (TS) related to accident monitoring instrumentation and time-limit values specified for timers.

Specifically, the licensees requested the proposed changes to Technical Specification Table 3.3.7.5-1 in order to insert exceptions to Specification 3.0.4 into the ACTION statements and to correct a typographical error; these proposed changes have been approved. Five additional proposed changes have been found to be unacceptable on the basis that they are not in conformance with the Standard Technical Specifications and the licensee has not submitted sufficient technical justification to show that these changes are acceptable. Three of these changes concern the time-limit values specified for timers in Table 3.3.2-2, one change concerns an inconsistency existing between ACTION 81 associated with Table 3.3.7.5-1 and the general ACTION and Limiting Condition for Operation specified under Specification 3.3.7.5, and one change concerns

the deletion of the safety/relief valve acoustic monitors from the Accident Monitoring Instrumentation.

This revision to the Clinton Power Station license would be made in response to the licensees' application for amendment dated October 30, 1987.

The Need for the Proposed Action:

Pursuant to 10 CFR 50.90, the licensees have proposed an amendment to Facility Operating License No. NPF-62 which consists of seven changes to the TS concerning accident monitoring instrumentation and time-limit values specified for timers.

The first approved change consists of revisions to Table 3.3.7.5-1 in order to insert exceptions to Specification 3.0.4 into associated ACTION statements. These exceptions would permit entry into OPERATIONAL CONDITIONS 1, 2 and 3 with an accident monitoring instrumentation channel(s) inoperable, as provided in the individual ACTION statements. Although the instrumentation is required to be OPERABLE in OPERATIONAL CONDITIONS 1, 2 and 3, the instrumentation is designed and intended to be used to "assess plant and environs conditions during and following an accident" (Ref. Regulatory Guide 1.97). Thus, entry into Conditions 1, 2 and 3 does not necessarily correspond to entry into the conditions for which the accident monitoring instrumentation will be needed. Conditions 1, 2 and 3 do correspond to the OPERATIONAL CONDITIONS in which a design basis accident is most likely to be initiated. Thus, entry into CONDITIONS 1, 2 and 3 should be permitted with the number of OPERABLE channels less than the REQUIRED NUMBER OF CHANNELS requirement, but not with the number of OPERABLE channels less than the MINIMUM CHANNELS OPERABLE requirement

(except when the ACTION provides compensatory measures). Exceptions to Specification 3.0.4 would therefore be inserted into ACTIONS 80, 81 and 82.

The second approved change consists of a revision to Table 3.3.7.5-1 in order to correct a typographical error. The "." note for the suppression pool water temperature sensors refers to Specification 3.5.3.1 instead of 3.6.3.1 where requirements for the other suppression pool temperature sensors are specified.

Environmental Impacts of the Proposed Action:

The proposed changes to Technical Specification Table 3.3.7.5-1 would add notations to remove the applicability of the provisions of Technical Specification 3.0.4 for accident monitoring instrumentation. This change would permit entry into Operational Conditions 1, 2, or 3 with inoperable channels and while in appropriate action statements. This change is consistent with staff's policy for instrumentation which does not initiate automatic mitigation functions and with current standard technical specifications.

The proposed change to note "." under TABLE NOTATIONS for Technical Specification Table 3.3.7.5-1 would revise the reference to Technical Specification 3.6.3.1 rather than 3.5.3.1. This change is editorial to correct a typographical error and is acceptable.

The Commission has determined that the proposed changes have no adverse effect on the probability of any accident. The proposed changes would not result in any increase in the potential radiological releases during normal operations, transients, and for accidents. No changes are being made in the types or amounts of any radiological effluents that may be released offsite and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. With regard to non-radiological

impacts, the proposed amendment involves systems located entirely within the restricted area as defined in 10 CFR Part 20. They do not affect non-radiological plant effluents and have no other environmental impact. Therefore, the staff also concludes that there are no significant non-radiological environmental impacts associated with the proposed amendment.

Accordingly, the Commission findings in the "Final Environmental Statement related to the operation of Clinton Power Station, Unit No. 1" dated May 1982 regarding radiological environmental impacts from the plant during normal operation or after accident conditions, are not adversely altered by this action.

Alternative to the Proposed Actions:

The principal alternative would be to deny the requested amendment. This alternative, in effect, would be the same as a "no action" alternative. Since the Commission has concluded that there are no significant environmental effects that would result from the proposed action, any alternative with equal or greater environmental impact need not be evaluated.

Alternative Use of Resources:

This action does not involve the use of resources not previously considered in the Nuclear Regulatory Commission's Final Environmental Statement for the Clinton Power Station, Unit 1, dated May 1982.

Agencies and Persons Consulted:

The NRC staff reviewed the licensees' request of October 30, 1987 and did not consult other agencies or persons.

FINDING OF NO SIGNIFICANT IMPACT:

The Commission has determined not to prepare an environmental impact

statement of the proposed license amendment.

Based upon this environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment.

The Notice of Consideration of Issuance of Amendment and Opportunity for Hearing in connection with this action was published in the Federal Register on February 16, 1988 (53 FR 4475) and February 18, 1988 (53 FR 4919). No request for hearing or petition for leave to intervene was filed following this notice.

For further details with respect to this action, see the request for amendment dated October 30, 1987, and the Final Environmental Statement for the Clinton Power Station dated May 1982, which are available for public inspection at the Commission Public Document Room, 2120 L Street, N.W., Washington, D.C. 20555 and at the Vespasian Warner Public Library, 120 West Johnson Street, Clinton, Illinois 61727.

Dated at Rockville, Maryland this 19th day of January 1990.

FOR THE NUCLEAR REGULATORY COMMISSION



John W. Craig, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects