

February 8, 1990

Docket No. 50-461

Mr. Dale L. Holtzscher
Acting Manager - Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

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Dear Mr. Holtzscher:

SUBJECT: TECHNICAL SPECIFICATION CHANGE REQUEST TO REVISE THE ACTION REQUIRED IF ONE OR MORE OF THE DRYWELL/CONTAINMENT HYDROGEN AND OXYGEN CONCENTRATION ANALYZERS/MONITORS ARE INOPERABLE FOR THE CLINTON POWER STATION, UNIT NO. 1 (TAC NO. 69550)

The Commission has issued the enclosed Amendment No.31 to Facility Operating License No. NPF-62 for the Clinton Power Station, Unit No. 1. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated May 18, 1988.

This amendment revises Technical Specification Table 3.3.7.5-1, "Accident Monitoring Instrumentation," which specifies Operability requirements for the Drywell/Containment Hydrogen and Oxygen Concentration Analyzers/Monitors and includes Action to be taken if one or more of the monitors are inoperable. This revision makes the action consistent with the guidance provided in NRC Generic Letter 83-36.

A copy of our Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

John B. Hickman, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000461
P PNU

Enclosures:

1. Amendment No. 31 to License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

PD32:LA
LLUTHER/hj
12/11/89

JBH
PD32:PM
JHICKMAN
1/02/90

[Signature]
PD32:D
JCRAIG
1/10/89

[Signature]
OGC
R Bachmann
2/5/90

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[AMENDMENT CLINTON 11/89]

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 8, 1990

Docket No. 50-461

Mr. Dale L. Holtzscher
Acting Manager - Licensing and Safety
Clinton Power Station
P. O. Box 678
Mail Code V920
Clinton, Illinois 61727

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SUBJECT: TECHNICAL SPECIFICATION CHANGE REQUEST TO REVISE THE ACTION REQUIRED IF ONE OR MORE OF THE DRYWELL/CONTAINMENT HYDROGEN AND OXYGEN CONCENTRATION ANALYZERS/MONITORS ARE INOPERABLE FOR THE CLINTON POWER STATION, UNIT NO. 1 (TAC NO. 69550)

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Sincerely,

A handwritten signature in cursive script, appearing to read "John B. Hickman".

John B. Hickman, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 31 to License No. NPF-62
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Dale L. Holtzscher
Illinois Power Company

Clinton Power Station
Unit 1

cc:

Mr. J. S. Perry
Vice President
Clinton Power Station
P. O. Box 678
Clinton, Illinois, 61727

Illinois Department
of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Mr. R. D. Freeman
Manager-Nuclear Station Engineering Dept.
Clinton Power Station
P. O. Box 678
Clinton, Illinois 61727

Mr. Donald Schopfer
Project Manager
Sargent & Lundy Engineers
55 East Monroe Street
Chicago, Illinois 60603

Sheldon Zabel, Esquire
Schiff, Hardin & Waite
7200 Sears Tower
233 Wacker Drive
Chicago, Illinois 60606

Resident Inspector
U. S. Nuclear Regulatory Commission
RR#3, Box 229 A
Clinton Illinois 61727

Mr. L. Larson
Project Manager
General Electric Company
175 Curtner Avenue, N/C 395
San Jose, California 95125

Regional Administrator, Region III
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137

Chairman of DeWitt County
c/o County Clerk's Office
DeWitt County Courthouse
Clinton, Illinois 61727



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

ILLINOIS POWER COMPANY, ET AL.

DOCKET NO. 50-461

CLINTON POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 31
License No. NPF-62

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Illinois Power Company* (IP), and Soyland Power Cooperative, Inc. (the licensees) dated May 18, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-62 is hereby amended to read as follows:

*Illinois Power Company is authorized to act as agent for Soyland Power Cooperative, Inc. and has exclusive responsibility and control over the physical construction, operation and maintenance of the facility.

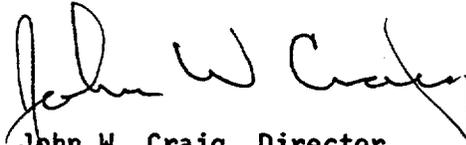
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Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 31, are hereby incorporated into this license. Illinois Power Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John W. Craig, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 8, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 31

FACILITY OPERATING LICENSE NO. NPF-62

DOCKET NO. 50-461

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

3/4 3-87

3/4 3-88

Insert

3/4 3-87

3/4 3-88

TABLE 3.3.7.5-1

ACCIDENT MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>REQUIRED NUMBER OF CHANNELS</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE OPERATIONAL CONDITIONS</u>	<u>ACTION</u>
1. Reactor Vessel Pressure	2	1	1, 2, 3	80
2. Reactor Vessel Water Level	2	1	1, 2, 3	80
3. Suppression Pool Water Level	4	2	1, 2, 3	80
4. Suppression Pool Water Temperature	2/quadrant†	1/quadrant†	1, 2, 3	80
5. Drywell Pressure	2	1	1, 2, 3	80
6. Drywell Air Temperature	2	1	1, 2, 3	80
7. Drywell/Containment Hydrogen and Oxygen Concentration Analyzer and Monitor	2	1	1, 2, 3	83
8. Containment Pressure ##	2/division	1/division	1, 2, 3	80
9. Containment Temperature	2	1	1, 2, 3	80
10. Safety/Relief Valve Acoustic Monitor	1/valve***	1/valve***	1, 2, 3	80
11. Containment/Drywell High Range Gross Gamma Radiation Monitors	4**	2*	1, 2, 3	81
12. HVAC Stack High Range Radioactivity Monitor#	1	1	1, 2, 3	81
13. SGTS Exhaust High Range Radioactivity Monitor#	1	1	1, 2, 3	81
14. Primary Containment Isolation Valve Position Indication ††	2/valve	1/valve	1, 2, 3	82

TABLE NOTATIONS

* One each for containment and drywell.

** Two each for containment and drywell.

*** Thermocouples in the SRV discharge line can serve as backup to the acoustic tail pipe monitors indication should one channel of the position indication become inoperable.

High range noble gas monitors and iodine/particulate sampler.

For Divisions I and II only.

† These instruments monitor suppression pool water temperature when pool water level is below instruments of Specification 3.5.3.1.

†† One channel consists of the open limit switch, and the other channel consists of the closed limit switch for each automatic isolation valve in Table 3.6.4-1 Part 1, "Automatic Isolation Valves."

TABLE 3.3.7.5-1 (Continued)
ACCIDENT MONITORING INSTRUMENTATION

ACTION

- ACTION 80 -
- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.
 - b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, restore the inoperable channel(s) to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.
- ACTION 81 -
- With the number of OPERABLE Channels less than required by the Minimum Channels OPERABLE requirement, either restore the inoperable Channel(s) to OPERABLE status within 72 hours, or:
- a. Initiate the preplanned alternate method of monitoring the appropriate parameter(s), and
 - b. Prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within 14 days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.
- ACTION 82 -
- a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3.7.5-1, verify the valve(s) position by use of alternate indication methods; restore the inoperable channel(s) to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
 - b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, verify the valve(s) position by use of alternate indication methods; restore the inoperable channel(s) to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.
- ACTION 83 -
- a. With the number of OPERABLE channels one less than the required number of channels shown in Table 3.3.7.5-1, restore the inoperable channel to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
 - b. With the number of OPERABLE channels less than the minimum channels OPERABLE requirements of Table 3.3.7.5-1, restore at least one channel to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 31 TO FACILITY OPERATING LICENSE NO. NPF-62

CLINTON POWER STATION, UNIT NO. 1

ILLINOIS POWER COMPANY

DOCKET NO. 50-461

1.0 INTRODUCTION

By letter dated May 18, 1988, the Illinois Power Company (IP), et al. (the licensees) requested an amendment to Facility Operating License No. NPF-62 for the Clinton Power Station (CPS), Unit 1. The proposed amendment would revise Technical Specification Table 3.3.7.5-1, "Accident Monitoring Instrumentation," which specifies Operability requirements for the Drywell/Containment Hydrogen and Oxygen Concentration Analyzers/Monitors and includes Action to be taken if one or more of the monitors are inoperable. This revision would make the action consistent with guidance provided in NRC Generic Letter (GL) 83-36.

2.0 EVALUATION

NRC Generic Letter 83-36 provided guidance to licensees on the Technical Specifications requirements for several NUREG-0737 items. This guidance included specific operability and action requirements for accident monitoring instrumentation. The action recommended for an inoperable hydrogen monitor(s) reads as follows:

- a. With the number of OPERABLE channels one less than the required number of channels shown in Table 3.3.7.5-1, restore the inoperable channel to OPERABLE status within 30 days or be in at least HOT SHUTDOWN within the next 12 hours.
- b. With the number of OPERABLE channels less than the minimum channels OPERABLE requirements of Table 3.3.7.5-1, restore at least one channel to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.

The action that is currently specified in the CPS Technical Specifications for the monitors states that with the number of OPERABLE channels (monitors) less than the Required Number of Channels shown in Table 3.3.7.5-1, the inoperable channels (monitors) must be restored to OPERABLE status within 7 days or be in at least HOT SHUTDOWN within the next 12 hours.

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It further states that with the number of OPERABLE channels (monitors) less than the Minimum Channels OPERABLE requirements of Table 3.3.7.5-1, the inoperable channels (monitors) must be restored to OPERABLE status within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

As can be seen, the action requirements that are currently specified for Clinton are more restrictive than the requirements that were provided in GL 83-36. The actions are consistent with guidance that was provided in the standard technical specifications prior to GL 83-36. However, difficulties associated with performing repairs on the hydrogen monitors have created operational difficulties. Since the units operate at a high temperature (280-300 degrees F), maintenance and troubleshooting activities are time consuming. Extreme caution must be exercised when working on the units due to the high temperature or a cooldown period must be allowed. In addition, there is a 12-hour heat up time associated with the monitor to allow the system to return to operating temperatures.

The hydrogen monitors are not used during normal operation of the plant. They are only intended to provide information to the operations staff under post-accident conditions. In addition, the hydrogen monitors do not initiate automatic mitigation functions. The post-accident sampling system can provide limited compensatory measures for the hydrogen monitors, although the sampling system results are delayed and not continuous.

NRC GL 83-36 and previous guidance provided only the more restrictive recommended ACTION statement for the Oxygen Concentration Instrumentation. However, the installation of an Oxygen Concentration Instrumentation was not required for the Clinton Power Station since they did not have an inerted containment. Therefore, the analysis of the ACTION recommended for the Hydrogen Concentration Analyzer/Monitor applies to the combined Oxygen and Hydrogen Concentration instrumentation at CPS. As a result, the less restrictive action requirements may be used at CPS without affecting the safety of the plant.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment only involves a change in a requirement with respect to the use of a facility component located within the restricted area. The staff has determined that the amendment involves no significant increased in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The proposed changes to Technical Specification Table 3.3.7.5-1, in order to revise the ACTION statement for inoperable Drywell/Containment Hydrogen and

Oxygen Concentration Analyzers/Monitors, are acceptable. The CPS Technical Specifications will remain consistent with NRC guidance, operational flexibility will be increased, and safety system operation will be unchanged.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and, (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety to the public.

Principal Contributor: John B. Hickman, NRR/PDIII-2

Dated: February 8, 1990