

January 19, 1989

Docket Nos.: 50-254 and 50-265

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Bliss:

SUBJECT: MAIN STEAM ISOLATION VALVE LOCAL LEAK RATE TESTING (TAC NOS. 71181 AND 71182)

Re: Quad Cities Nuclear Power Station, Units 1 and 2

The Commission has issued the enclosed Amendment Nos. 113 and 109 to Facility Operating License Nos. DPR-29 and DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2. These amendments are in response to your application dated November 16, 1988. The required surveillance interval for conducting local leak rate tests on Main Steam isolation valves will now be changed from every 18 months to once per operating cycle (not to exceed 24 months).

A copy of our Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

151

Thierry M. Ross, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 113 to License No. DPR-29
2. Amendment No. 109 to License No. DPR-30
3. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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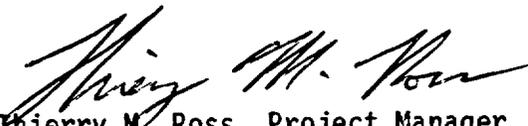
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Thierry M. Ross, Project Manager
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See next page

Mr. Henry E. Bliss
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Units 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 113
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated November 16, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

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P PDC

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 113, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 19, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 113

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.7/4.7-4

INSERT

3.7/4.7-4

QUAD-CITIES
DPR-29

e. With the measured leakage rate exceeding 11.5 scf per hour for any one main steam isolation valve, restore the leakage rate to \leq 11.5 scf per hour for any one main steam isolation valve prior to increasing the reactor coolant temperature above 212°F.

2) Main steam isolation valves, which shall be leak tested at least once per operating cycle at a frequency not to exceed 24 months and at a pressure of 25 psig.

3) Bolted double-gasketed seals which shall be tested at a pressure of 48 psig whenever the seal is closed after being opened and each operating cycle.

4) While valve MO 1-220-1 is inoperable, valves MO 1-220-2, MO 1-220-3, and MO 1-220-4 shall be VERIFIED closed after each closure.

e. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurements system.

3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

a. Except as specified in Specifications 3.7.A.3.b below, two pressure sup-

3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

a. The pressure suppression chamber-reactor building vacuum



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 109
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated November 16, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 109, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 19, 1989

ATTACHMENT TO LICENSE AMENDMENT NO.109

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.7/4.7-4

INSERT

3.7/4.7-4

QUAD-CITIES
DPR-30

e. With the measured leakage rate exceeding 11.5 scf per hour for any one main steam isolation valve, restore the leakage rate to \leq 11.5 scf per hour for any one main steam isolation valve prior to increasing the reactor coolant temperature above 212°F.

2) Main steam isolation valves, which shall be leak tested at least once per operating cycle at a frequency not to exceed 24 months and at a pressure of 25 psig.

3) Bolted double-gasketed seals which shall be tested at a pressure of 48 psig whenever the seal is closed after being opened and each operating cycle.

e. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurements system.

3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

a. Except as specified in Specifications 3.7.A.3.b below, two pressure sup-

3. Pressure Suppression Chamber-Reactor Building Vacuum Breakers

a. The pressure suppression chamber-reactor building vacuum



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 113 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 109 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-254/265

1.0 INTRODUCTION

By a letter dated November 16, 1988, Commonwealth Edison Company (CECo, the licensee) requested approval to amend the Technical Specification (TS) surveillance requirements for local leak rate testing (LLRT) of the Main Steam Isolation Valves (MSIV). Currently, TS 4.7.A.2.d.2 requires a surveillance interval of once every 18 months. CECo proposed to revise this interval to once per operating cycle, but not to exceed 24 months.

2.0 EVALUATION

When TS 4.7.A.2.d.2 was originally drafted, the Quad Cities Nuclear Power Station (QCNPS) operated on 12-month fuel cycles. At that time, an 18-month surveillance interval for LLRT of MSIVs was deemed appropriate. Since testing of MSIVs can only be accomplished during a plant shutdown, this interval ensured that the MSIVs were tested at least every refueling outage without impacting upon normal QCNPS operations. As fuel cycles were extended to 18 months in duration, the TS surveillance requirement was never revised to adjust for the longer time between refueling outages. Consequently, LLRT of MSIVs will be required before the next refueling outage which would necessitate a forced unit shutdown to comply with TS. To date, QCNPS has met the 18-month surveillance interval by invoking the 25% allowable extension. But this is inadequate for the long-term as the surveillance interval can only be extended to a total of 325% in three intervals.

In accordance with 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water Cooled Power Reactors," all primary containment isolation valves (including MSIVs) are required to be leak rate tested locally every 24 months. However, TS specifically mandate MSIV LLRTs every 18 months to ensure they are tested each fuel cycle. The proposed TS amendment would revise MSIV surveillance intervals to be consistent with 10 CFR Part 50, Appendix J while maintaining the original intent of TS 4.7.A.2.d.2 to test MSIVs each fuel cycle. This change is important in order to prevent unnecessary unit shutdowns.

The NRC staff considers the licensee's amendment request to be acceptable as it conforms with applicable regulations and continues to meet the original intention of the TS requirement.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a surveillance test requirement for a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Thierry Ross

Dated: January 19, 1989