

February 6, 1989

Docket No.: 50-265

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Bliss:

SUBJECT: DRYWELL TEMPERATURE INDICATOR AMENDMENT (TAC 68661)

Re: Quad Cities Nuclear Power Station, Unit 2

The Commission has issued the enclosed Amendment No. 110 to Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Unit 2. This amendment is in response to your application dated July 7, 1988 to revise the specified location for drywell temperature indication.

A copy of our Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

151
Thierry Ross, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 110 to License No. DPR-30
2. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION

Docket file	NRC & Local PDRs	PDIII-2 r/f	GHolahan	EJordan
LLuther	TRoss	OGC/WF	DHagan	ACRS(10)
BGrimes	TMeek(8)	WJones	EButcher	
GPA/PA	ARM/LFMB	Plant file	MVirgilio	

PDIII-2:PM
TRoss:dmj *TR*
1/26/89

PDIII-2:LA
LLuther *LL*
1/26/89

OGC
R Bachmann
1/31/89

PDIII-2:PD
DMuller *DM*
1/21/89

8902100172 890206
PDR ADDCK 05000265
P PDC

DF01
11
c/p1
cc



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 6, 1989

Docket No.: 50-265

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Bliss:

SUBJECT: DRYWELL TEMPERATURE INDICATOR AMENDMENT (TAC 68661)

Re: Quad Cities Nuclear Power Station, Unit 2

The Commission has issued the enclosed Amendment No.110 to Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Unit 2. This amendment is in response to your application dated July 7, 1988 to revise the specified location for drywell temperature indication.

A copy of our Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, reading "Thierry Ross", is positioned above the typed name.

Thierry Ross, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No.110 to License No. DPR-30
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Henry E. Bliss
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Units 1 and 2

cc:

Mr. Stephen E. Shelton
Vice President
Iowa-Illinois Gas and
Electric Company
P. O. Box 4350
Davenport, Iowa 52808

Michael I. Miller, Esq.
Sidley and Austin
One First National Plaza
Chicago, Illinois 60603

Mr. Richard Bax
Station Manager
Quad Cities Nuclear Power Station
22710 206th Avenue North
Cordova, Illinois 61242

Resident Inspector
U. S. Nuclear Regulatory Commission
22712 206th Avenue North
Cordova, Illinois 61242

Chairman
Rock Island County Board
of Supervisors
1504 3rd Avenue
Rock Island County Office Bldg.
Rock Island, Illinois 61201

Mr. Michael E. Parker, Chief
Division of Engineering
Illinois Department of Nuclear Safety
1035 Outer Park Drive,
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated July 7, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

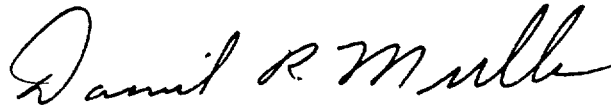
8902100179 890206
PDR ADDCK 05000265
PDC

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 110, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "Daniel R. Muller".

Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 6, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 110

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.2/4.2 - 15

3.2/4.2 - 18

3.2/4.2 - 18a

INSERT

3.2/4.2 - 15

3.2/4.2 - 18

3.2/4.2 - 18a

TABLE 3.2-4

POSTACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS[2]

Minimum Number of Operable Chan- nels[1] [3]	Parameter	Instrument Readout Location Unit 2	Number Provided	Range
1	Reactor pressure	902-5	1 2	0-1500 psig 0-1200 psig
1	Reactor water level	902-3	2	-243 inches+57 inches
1	Torus water temperature	902-21	2	0-200°F
1	Torus air temperature	902-21	2	0-600°F
	Torus water level indicator	902-3	1	-5 inches +5 inches (narrow range)
2[6]	Torus water level indicator	902-3	2	0-30 feet (wide range)
	Torus water level sight glass		1	18 inch range (narrow range)
1	Torus pressure	902-3	1	-5 inches Hg to 5 psig
2	Drywell pressure	902-3	1	-5 inches Hg to 5 psig -10 inches Hg to 70 psig
			2	0 to 250 psig
2	Drywell temperature	902-3	6	0-600°F
2	Neutron monitoring	902-5	4	0.1-10 ⁸ CPS
2[4]	Torus to drywell differential pressure		2	0-3 psid
1(8)	Drywell Hydrogen concentration	902-55, 56	2	0-4%
2(7)	Drywell radiation monitor	902-55, 56	2	1 to 10 ⁸ R/hr
2/valve[5]	Main Steam RV posi- tion, acoustic monitor	902-21	1 per valve	NA
	Main Steam RV position, temperature monitor	902-21	1 per valve	0-600°F
2/valve[5]	Main Steam SV posi- tion, acoustic monitor	902-21	1 per valve	NA
	Main Steam SV position, temperature monitor	902-21	1 per valve	0-600°F

QUAD-CITIES
DPR-30

TABLE 4.2-2

POSTACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

Minimum Number of Operable Channels*	Parameter	Instrument Readout Location Unit 2	Calibration	Instrument Check
1	Reactor pressure	902-5	Once every 3 months	Once per day
1	Reactor water level	902-3	Once every 3 months	Once per day
1	Torus water temperature	902-21	Once every 3 months	Once per day
1	Torus air temperature	902-21	Once every 3 months	Once per day
2	Torus water level (narrow range)	902-3	Once every 3 months	Once per day
	Torus water level indicator (wide range)	902-3	Once every 18 months	Once per 31 days
	Torus water level sight glass		N/A	None
1	Torus pressure	902-3	Once every 3 months	Once per day
2	Drywell pressure	902-3	Once every 3 months	Once per day
2	Drywell temperature	902-3	Once every 3 months	Once per day
2	Neutron monitoring	902-5	Once every 3 months	Once per day
2	Torus to drywell differential pressure		Once every 6 months	None
1	Drywell hydrogen concentration	902-55, 56	Once every 3 months	Once per 31 days
2	Drywell radiation monitor	902-55, 56	Once every 18 months ***	Once per 31 days
2/valve	Main Steam RV Position, acoustic monitor	902-21	**	Once per 31 days
	Main Steam RV Position, temperature monitor	902-21	Once every 18 months	Once per 31 days

QUAD-CITIES
DPR-30

TABLE 4.2-2 (con'd)

Minimum Number of Operable <u>Channels*</u>	<u>Parameter</u>	Instrument Readout Location <u>Unit 2</u>	<u>Calibration</u>	Instrument <u>Check</u>
2/valve	Main Steam SV Position, acoustic monitor	902-21	**	Once per 31 days
	Main Steam SV Position, temperature monitor	902-21	Once every 18 months	Once per 31 days

* Instrument channels required during power operation to monitor postaccident conditions.

** Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular safety or relief valve.

*** Calibration shall consist of an electronic calibration of the channel, not including the detector, for range decades above 10 R/hr; and a one-point calibration check of the detector below 10 R/hr with an installed or portable gamma source.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. DPR-30
COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY
QUAD CITIES NUCLEAR POWER STATION, UNIT 2
DOCKET NO. 50-265

1.0 INTRODUCTION

As part of the Detailed Control Room Design Review (DCRDR) for Quad Cities Nuclear Power Station (QCNPS), Unit 2, Commonwealth Edison Company (CECo, the licensee) implemented a human factors modification of the drywell temperature indicator. During the Spring 1988 refueling outage for Unit 2, the drywell temperature indicator was relocated. By letter dated June 20, 1988, CECo committed to revise the QCNPS, Unit 2, Technical Specifications (TS) to reflect this human factors modification. In conformance with their commitment, CECo submitted a TS amendment application on July 7, 1988 to revise the drywell instrument panel location. In addition, changes to correct typographical errors were proposed for associated TS pages.

2.0 EVALUATION

There are two types of changes associated with this proposed license amendment. The first type of change results from completion of a Unit 2 DCRDR modification which relocated the drywell temperature indicator from the 902-21 (back) panel to the 902-3 (front) panel. This change needs to be incorporated into TS Tables 3.2-4 and 4.2-2 of DPR-30. The remaining changes correct typographical errors discovered to exist in these same tables. They revise the "instrument readout location" for all parameters, from panel 901 to panel 902, to reflect the fact that these instruments are located on Unit 2 panels not Unit 1.

The drywell temperature indication has been relocated from the back to the front panel in the Control Room. This is a human factored enhancement over the previous location and should provide easier access for the operators. Since the range, function, and operation of the drywell pressure instrument remains unchanged, the NRC staff concludes that this amendment is acceptable. Furthermore, all proposed editorial corrections to revise panel locations are also considered appropriate and acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to the installation of a component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Thierry Ross

Dated: February 6, 1989