

April 3, 1990

Docket No. 50-265

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[75388 AMENDMENT]	

Mr. Thomas J. Kovach
 Nuclear Licensing Manager
 Commonwealth Edison Company-Suite 300
 OPUS West III
 1400 OPUS Place
 Downers Grove, IL 60515

Dear Mr. Kovach:

SUBJECT: TECHNICAL SPECIFICATION AMENDMENT TO REFLECT REMOVAL OF
 HEAD SPRAY AND CONTROL ROD DRIVE RETURN LINE PIPING
 (TAC NO. 75388)

The Nuclear Regulatory Commission has issued the enclosed Amendment No.119 to Appendix A, Technical Specifications, of Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station (QCNSP), Unit 2. This amendment is in response to an application from Commonwealth Edison Company dated December 11, 1989.

This amendment revises the Technical Specifications to reflect removal of the Head Spray and Control Rod Drive return line piping and to delete the term "(min)" from Table 3.7-1.

Our related Safety Evaluation report is also enclosed. A Notice of Issuance for these amendments will be included in the Commission's biweekly Federal Register notices.

Sincerely,

151

Leonard N. Olshan, Project Manager
 Project Directorate III-2
 Division of Reactor Projects - III,
 IV, V and Special Projects
 Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 119 to License No. DPR-30
2. Safety Evaluation

cc w/enclosures:
 See next page

LA:PD3/2:DRSP
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*OGC
 copy sent on 3/16/90
 of amended TSPS 75388
 2) Project file
 file issued on 3/16/90
 expiration of 30-day
 notice period 3/12/90*

April 3, 1990

Docket No. 50-265

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Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
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SUBJECT: TECHNICAL SPECIFICATION AMENDMENT TO REFLECT REMOVAL OF
HEAD SPRAY AND CONTROL ROD DRIVE RETURN LINE PIPING
(TAC NO. 75388)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 119 to Appendix A, Technical Specifications, of Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station (QCNS), Unit 2. This amendment is in response to an application from Commonwealth Edison Company dated December 11, 1989.

This amendment revises the Technical Specifications to reflect removal of the Head Spray and Control Rod Drive return line piping and to delete the term "(min)" from Table 3.7-1.

Our related Safety Evaluation report is also enclosed. A Notice of Issuance for these amendments will be included in the Commission's biweekly Federal Register notices.

Sincerely,

LSI

Leonard N. Olshan, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

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cc w/enclosures:
See next page

LA:PD3/2:DRSP
LLUTHER
3/7/90

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LOLSHAN/bj
3/7/90

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JCRAIG
4/4/90

OGC
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

April 3, 1990

Docket No. 50-265

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, IL 60515

Dear Mr. Kovach:

SUBJECT: TECHNICAL SPECIFICATION AMENDMENT TO REFLECT REMOVAL OF
HEAD SPRAY AND CONTROL ROD DRIVE RETURN LINE PIPING
(TAC NO. 75388)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 119 to Appendix A, Technical Specifications, of Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station (QCNP), Unit 2. This amendment is in response to an application from Commonwealth Edison Company dated December 11, 1989.

This amendment revises the Technical Specifications to reflect removal of the Head Spray and Control Rod Drive return line piping and to delete the term "(min)" from Table 3.7-1.

Our related Safety Evaluation report is also enclosed. A Notice of Issuance for these amendments will be included in the Commission's biweekly Federal Register notices.

Sincerely,

A handwritten signature in cursive script that reads "Leonard N. Olshan".

Leonard N. Olshan, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 119 to License No. DPR-30
2. Safety Evaluation

cc w/enclosures:
See next page

Mr. Thomas J. Kovach
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Units 1 and 2

cc:

Mr. Stephen E. Shelton
Vice President
Iowa-Illinois Gas and
Electric Company
P. O. Box 4350
Davenport, Iowa 52808

Michael I. Miller, Esq.
Sidley and Austin
One First National Plaza
Chicago, Illinois 60690

Mr. Richard Bax
Station Manager
Quad Cities Nuclear Power Station
22710 206th Avenue North
Cordova, Illinois 61242

Resident Inspector
U. S. Nuclear Regulatory Commission
22712 206th Avenue North
Cordova, Illinois 61242

Chairman
Rock Island County Board
of Supervisors
1504 3rd Avenue
Rock Island County Office Bldg.
Rock Island, Illinois 61201

Illinois Department of Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road, Bldg. #4
Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 119
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated December 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

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B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 119, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John W. Craig, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 3, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 119

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.7/4.7-20

3.7/4.7-20a

3.7/4.7-20b

3.7/4.7-23

INSERT

3.7/4.7-20

3.7/4.7-20a

3.7/4.7-20b

3.7/4.7-23

QUAD-CITIES
DPR-30

TABLE 3.7-1

PRIMARY CONTAINMENT ISOLATION

Isolation Group	Valve Identification	Valve Number for Units		Number of Power-Operated Valves		Maximum Operating Time (sec)	Normal Operating Position	Action on Initiating Signal
		1 and 2		Inboard	Outboard			
Main Steam Isolation								
1	Main steam isolation valve	A0-203-1A, 1B, 1C, 1D	4			$3 \leq T < 5$	0	GC
1	Main steam isolation valve	A0-203-2A, 2B, 2C, 2D		4		$3 \leq T < 5$	0	GC
1	Main steam drain isolation valve	MO-220-1	1			≤ 35	C	SC
1	Main steam drain isolation valve	MO-220-2		1		≤ 35	C	SC
Sampling								
1	Recirculating sample valve	AO-220-44	1			≤ 5	0	GC
1	Recirculation sample valve (NOTE: Valve can be reopened after isolation for sampling)	AO-220-45		1		≤ 5	0	GC
RHR								
2	RHR discharge to radwaste	MO-1001-20	1			≤ 25	C	SC
2	RHR discharge to radwaste	MO-1001-21		1		≤ 25	C	SC

QUAD-CITIES
DPR-30

TABLE 3.7-1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION

Isolation Group	Valve Identification	Valve Number for Units 1 and 2	Number of Power-Operated Valves Inboard	Outboard	Maximum Operating Time (sec)	Normal Operating Position	Action on Initiating Signal
2	Reactor shutdown cooling supply	MO-1001-47		1	≤40	C	SC
2	Reactor shutdown cooling supply	MO-1001-50	1		≤40	C	SC
Pressure Suppression							
2	Drywell purge valve	AO-1601-21	1		≤10	C	SC
2	Vent valve	AO-1601-22		1	≤10	C	SC
2	Drywell vent valve	AO-1601-23	1		≤10	C	SC
2	Vent to reactor building exhaust system	AO-1601-24		1	≤10	C	SC
2	Nitrogen purge	AO-1601-55		1	≤10	O	GC
2	Torus purge valve	AO-1601-56	1		≤10	O	GC
2	Makeup valve	MO-1601-57	1		≤15	O	GC
2	Torus makeup valve	AO-1601-58		1	≤15	C	SC

QUAD-CITIES
DPR-30

TABLE 3.7-1 (Cont'd)

PRIMARY CONTAINMENT ISOLATION

Isolation Group	Valve Identification	Valve Number for Units 1 and 2	Number of Power-Operated Valves Inboard	Outboard	Maximum Operating Time (sec)	Normal Operating Position	Action on Initiating Signal
2	Drywell makeup valve	AO-1601-59		1	≤15	O	GC
2	Torus vent valve	AO-1601-60	1		≤10	C	SC
2	Torus 2-inch vent relief	AO-1601-61	1		≤15	C	SC
2	Drywell 2-inch vent relief	AO-1601-62	1		≤15	C	SC
2	Vent to SGT system	AO-1601-63		1	≤10	C	SC
2	Drywell pneumatic Supply isolation	AO-4720		2	≤10	O	GC
		AO-4721					

QUAD-CITIES
DPR-29

TABLE 3.7-2

PRIMARY CONTAINMENT LEAKAGE TEST PENETRATIONS

Double-Gasketed Seals

X-1	Equipment hatch
X-2	Personnel air lock
X-6	Control rod drive hatch
X-35A through G	TIP drives
X-4	Drywell head access hatch
X-200A & B	Suppression chamber access hatch
1 through 8	Drywell head shear lug (inspection hatch)

Pipe Penetrations

X-7A through D	Main steam
X-8	Main steamline drain
X-9A & B	Feedwater
X-10	Reactor core isolation cooling
X-11	High-pressure coolant injection
X-12	Residual heat removal
X-13A & B	Residual heat removal
X-14	Reactor coolant cleanup
X-16A & B	Core spray
X-23 & 24	Reactor building closed cooling water
X-25 & 26	Drywell ventilation
X-47	Standby liquid control

Electrical Penetrations

X-100B, C, E, F, & G
X-101A, B & D
X-102B
X-103
X-104A through D & F
X-105C
X-106A & B
X-107A & B



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 119 TO FACILITY OPERATING LICENSE NO. DPR-30
COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY
QUAD CITIES NUCLEAR POWER STATION, UNIT 2
DOCKET NO. 50-265

1.0 INTRODUCTION

By letter dated December 11, 1989, Commonwealth Edison Company (the licensee) proposed changes to the Technical Specifications (TS) for Quad Cities Nuclear Power Station, Unit 2. The proposed changes would reflect removal of the Head Spray and Control Rod Drive (CRD) return line piping. These lines are being removed as part of the Intergranular Stress Corrosion Cracking (IGSCC) mitigation program. (The identical changes were made for Unit 1 in Amendment No. 123, issued December 4, 1989.) The proposed change also deletes the term "(min)" from the main steamline isolation valve operating times in Table 3.7-1.

2.0 EVALUATION

Current Technical Specification 3.7.D.2 requires that all containment isolation valves contained in Table 3.7-1 shall be operable during reactor power operation. Table 3.7-1 contains a description and associated requirements for operating position and operating time for reactor head spray valves M0-1001-60 and M-1001-63. In addition to TS 3.7.D.2, Table 3.7-2 in TS lists the Primary Containment Leakage Test Penetration which includes penetrations for the reactor vessel head spray and CRD return lines.

The proposed changes to Technical Specifications would delete references to the reactor vessel head spray and CRD return lines from Tables 3.7-1 and 3.7-2 as a result of modifications that removed these lines.

Modifications to the reactor vessel head spray and CRD return lines were performed as part of the Station's Intergranular Stress Corrosion Cracking mitigation program, as put forth in the response to Generic Letter 88-01. The piping was determined to be unnecessary and highly susceptible to IGSCC.

The head spray system provided a means to augment reactor cooldown and reduction of pressure following a shutdown. The head spray system is part of the Residual Heat Removal System and allows water to be diverted to a spray nozzle in the steam dome of the vessel. Operation of the head spray system during reactor shutdown is optional. This system is not used to mitigate accidents, nor during normal or transient reactor operation.

The CRD return line was previously removed from service to prevent thermal stresses on the reactor vessel due to temperature differentials caused by the return of cooler CRD water. The CRD return line provides a return flowpath to the reactor vessel following CRD movement. The current flowpath is reverse flow through the exhaust header and return to the vessel through the CRD seals. Since the return line was capped, this piping is no longer utilized.

As a result of removing the head spray piping, valves MO-1001-60 and MO-1001-63 were eliminated. These valves provided for isolation of primary containment and were normally closed. Once the reactor head spray line was removed, the necessity for this containment isolation feature was eliminated. The remaining head spray piping (reactor side) was blanked with a blind flange which ensures integrity of the reactor coolant pressure boundary.

Removal of the CRD return line and head spray piping resulted in the enclosure of two drywell penetrations which were listed on Table 3-7.2. This table delineates the penetrations which require Type C local leak rate testing. The integrity of the closed penetration will continue to be tested during containment integrated leak rate tests. Closure of these penetrations eliminated possible leakage paths from containment. Welded caps over the penetrations (inside primary containment) were designed to be consistent with containment design pressures and temperature.

The deletion of the term "(min)" from the main steamline isolation valve (MSIV) operating times in Table 3.7-1 is an administrative change. The times listed in the table are in seconds, as specified in the heading of the column. The MSIV closure times described in the Final Safety Analysis Report and Technical Specification bases are in seconds. Thus, the term "(min)" is an error and should be removed.

Based on the above discussion, the staff concludes that the proposed amendment is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted areas as defined in 10 CFR Part 20. The staff has determined that this

amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(9). This amendment also involves changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, with respect to these items, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR §51.22(c)(10). Pursuant 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributors: Thierry Ross, NRR/PD3/2
Leonard Olshan, NRR/PD3/2

Dated: April 3, 1990