



UNITED STATES  
ATOMIC ENERGY COMMISSION  
WASHINGTON, D.C. 20545

Docket files

April 11, 1974

Docket Nos. 50-254  
and 50-265

Commonwealth Edison Company  
ATTN: Mr. J. S. Abel  
Nuclear Licensing Administrator  
Boiling Water Reactors  
Post Office Box 767  
Chicago, Illinois 60690

Change No. 15  
License Nos. DPR-29  
and DPR-30

Gentlemen:

Your letter dated February 27, 1974, proposed changes to the Technical Specifications of Facility Operating License Nos. DPR-29 and DPR-30. These changes are to modify the Technical Specifications concerning draining the torus and the inservice inspection program which is delineated in Table 4.6.1 of the Technical Specifications.

You have proposed to drain the torus for inspection and maintenance purposes while simultaneously performing control rod drive maintenance. Draining the torus removes an alternate supply of core residual heat removal (RHR) water required in the unlikely event water is drained from the reactor when a control rod drive is removed from the reactor vessel thimble. By procedural requirements the condensate storage tank containing a minimum of 230,000 gallons of water shall be immediately available as a primary source of core cooling and the fuel pool gate shall be open to provide additional water by draining the pool through the core to a minimum of 33 feet above the pool floor. This provides reasonable assurance that adequate capacity of water is available for core cooling, if required, during the portion of the refueling outage when the torus is drained. As a result of discussions with your representatives, we understand that procedures will be developed for performing this work prior to beginning the work. The procedures will specify that (1) no more than one control rod drive housing will be opened at any time, (2) a blind flange will be installed on the control rod drive housing whenever a control rod drive has been removed for maintenance, (3) work will not be performed in the reactor vessel while a control rod drive housing is open, and (4) a special flange, which you have developed, is available to be used to seal the control rod drive housing in the event a leak develops during the short interval that a drive

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housing is open. Based on the considerations that control rod maintenance concurrent with draining of the torus will occur very infrequently, procedural measures will be taken to minimize the period of time that a control rod drive housing is open and that makeup water capabilities exist, we have concluded that adequate provisions are taken to protect against unintentional draining of the reactor vessel while performing control rod maintenance with the torus drained. We have previously evaluated the loss of coolant accident during reactor operation and concluded that adequate emergency core cooling is available for this accident to protect the health and safety of the public. During refueling, only a small percentage of heat is generated in the core relative to that assumed in the loss of coolant accident analysis. Based upon the above precautions taken to preclude the loss of coolant during a single control rod drive removal and the fact that the loss of coolant accident under much more severe conditions has been previously evaluated, we have concluded that the proposed action results in no significant hazards consideration and that reasonable assurance exists that the health and safety of the public will not be endangered.

In our discussions with you regarding maintenance of more than one control rod drive, you describe a procedure to assure an adequate shutdown margin. We have concluded that this procedure is acceptable.

The proposed change to the Technical Specifications regarding the inservice inspection program is to conform with recent revisions of Section XI of the ASME Boiler and Pressure Vessel Code and to further standardize the inspection requirements of Dresden Units 2 and 3 and Quad-Cities Units 1 and 2. The proposal would revise Category E concerning pressure-retaining, partial penetration welds in vessels and add Category O concerning pressure-retaining welds in control rod drive housings. We have concluded that the proposed changes conform to the requirements of Regulatory Guide 1.51 and Section XI of the ASME Boiler and Pressure Vessel Code and therefore are acceptable.

We have concluded that the proposed changes to the Technical Specifications do not present significant hazards considerations and that there is reasonable assurance that the health and safety of the public will not be endangered.

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Pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specifications of Facility Operating License Nos. DPR-29 and DPR-30 are hereby changed to include the above requests by replacing pages 109, 110, 124, 140 and 143 with the revised pages and adding pages 102A, 109A, and 129A appended hereto.

Sincerely,

Donald J. Skovholt  
Assistant Director  
for Operating Reactors  
Directorate of Licensing

Enclosure:  
Revised pages

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