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Docket No. 50-254
50-265

AUG 9 1976

Commonwealth Edison Company
ATTN: Mr. R. L. Bolger
Assistant Vice President
Post Office Box 767
Chicago, Illinois 60690

Gentlemen:

In response to your request dated January 9, 1976, the Commission has issued the enclosed Amendment Nos. 26 and 27 to Facility License Nos. DPR-29 and DPR-30 for the Quad Cities Nuclear Power Station Units No. 1 and No. 2, respectively. The amendments change the surveillance frequency of the core and containment cooling system logic system functional tests from once per six months to once per refueling outage.

Copies of our Safety Evaluation and the Federal Register Notice related to this action also are enclosed.

Sincerely,

Original signed by
Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:
1. Amendment No. 26 to License No. DPR-29
2. Amendment No. 27 to License No. DPR-30
3. Safety Evaluation
4. Notice

cc w/enclosures:
See next page

V. Stello OK'd issuance on 3:30 today thru Ben Rosenberg.
DLZ 8/3/76

OFFICE	ORB #2:OR	OR:ORB #2	OELD	OR:ORB #2		
SURNAME	RMDiggs	PWO'Connor	KARMAN	DLZiemann		
DATE	8/12/76	8/12/76	8/13/76	8/19/76		

Form AEC-318 (Rev. 9-53) AECM 0240 ☆ U. S. GOVERNMENT PRINTING OFFICE: 1974-526-166

cc w/enclosures:
Mr. Charles Whitmore
President and Chairman
Iowa-Illinois Gas and
Electric Company
206 East Second Avenue
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Mr. John W. Rowe
Isham, Lincoln & Beale
Counselors at Law
One First National Plaza, 42nd Floor
Chicago, Illinois 60603

Anthony Z. Roisman, Esquire
Roisman, Kessler and Cashdan
1712 N Street, N. W.
Washington, D. C. 20036

Moline Public Library
504 - 17th Street
Moline, Illinois 61265

Mr. Robert Watts, Chairman
Rock Island County Board of
Supervisors
Rock Island County Court House
Rock Island, Illinois 61201

cc w/enclosures and cy of CECo's
filing dated 1/9/76:
Mr. Leroy Stratton
Bureau of Radiological Health
Illinois Department of Public Health
Springfield, Illinois 62706

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 26
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated January 9, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 9, 1976

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 27
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated January 9, 1976, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 9, 1976

ATTACHMENT TO LICENSE AMENDMENT NOS. 26 AND 27

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET NOS. 50-254 AND 50-265

Replace existing page 95 of the Technical Specifications with the attached revised page. The changed area on the revised page is denoted by a marginal line.

3.5 LIMITING CONDITION FOR OPERATION	4.5 SURVEILLANCE REQUIREMENT
	Item / **Frequency**
	least 4500 gpm against a system head corresponding to a reactor vessel pressure of 90 psig.
	c. Pump Operability — Once/month
	d. Motor Operated Valve — Once/month
	e. Core Spray Header Δp instrumentation check — Once/day calibrate — Once/3 months test — Once/3 months
	f. Logic System Functional Test — Each refueling outage
2. From and after the date that one of the core spray subsystems is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days unless such subsystem is sooner made operable, provided that during such seven days all active components of the other core spray subsystem and the LPCI mode of the RHR system and the diesel generators required for operation of such components if no external source of power were available shall be operable.	2. When it is determined that one core spray subsystem is inoperable the operable core spray subsystem and the LPCI mode of the RHR system and the diesel generators required for operation of such components if no external source of power were available shall be demonstrated to be operable immediately. The operable core spray subsystem shall be demonstrated to be operable daily thereafter.

3.5 LIMITING CONDITION FOR OPERATION	4.5 SURVEILLANCE REQUIREMENT
	Item — **Frequency**
	least 4500 gpm against a system head corresponding to a reactor vessel pressure of 90 psig.
	c. Pump Operability — Once/month
	d. Motor Operated Valve — Once/month
	e. Core Spray Header Δp instrumentation check — Once/day calibrate — Once/3 months test — Once/3 months
	f. Logic System Functional Test — Each refueling outage
2. From and after the date that one of the core spray subsystems is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days unless such subsystem is sooner made operable, provided that during such seven days all active components of the other core spray subsystem and the LPCI mode of the RHR system and the diesel generators required for operation of such components if no external source of power were available shall be operable.	2. When it is determined that one core spray subsystem is inoperable the operable core spray subsystem and the LPCI mode of the RHR system and the diesel generators required for operation of such components if no external source of power were available shall be demonstrated to be operable immediately. The operable core spray subsystem shall be demonstrated to be operable daily thereafter.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 26 AND 27 TO
FACILITY LICENSE NOS. DPR-29 AND DPR-30

COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES UNIT NOS. 1 AND 2

DOCKET NOS. 50-254 AND 50-265

INTRODUCTION

By letter dated January 9, 1976, the Commonwealth Edison Company requested an amendment to Facility License Nos. DPR-29 and DPR-30 for the Quad Cities Station Unit Nos. 1 and 2. The request involves a revision to the surveillance frequency for the Core Spray system and the Low Pressure Coolant Injection (LPCI) mode of the Residual Heat Removal (RHR) system functional test from once per six months to once each refueling outage.

DISCUSSION

The surveillance frequency specified for the Quad Cities Unit Nos. 1 and 2 logic system functional test of the core spray system and the LPCI mode of the RHR system is once per 6 months. The logic system surveillance for the Automatic Depressurization system and containment cooling mode of the RHR system is required once each refueling outage. Because the reactor must be shut down to perform these tests on the core spray system and the LPCI mode of the RHR system, Commonwealth Edison requested that these test frequencies be changed to once each refueling outage to provide consistent test requirements for all of these related systems.

EVALUATION

We have reviewed the proposed change that would increase the surveillance interval for the core spray system and the LPCI mode of the RHR system logic systems from six months to once each refueling outage. Our review considered the following:

1. The surveillance frequency of the logic system of containment cooling mode of the RHR system, the Automatic Depressurization system, and the High Pressure Coolant Injection System (HPCI), three comparable systems, at Quad Cities Station have been specified as once each refueling outage since their original issuance. No abnormal degradation has been observed in these logic systems.

2. The Commission's Standard Technical Specifications currently being issued for new facilities require the core spray and the LPCI mode of the RHR to be tested each refueling outage, approximately every 18 months.

3. The surveillance frequency specified for the logic systems of similar reactors such as Pilgrim Unit No. 1, Dresden Unit Nos. 2 and 3, and Monticello require that the core spray and the LPCI mode of the RHR be functionally tested each refueling outage. No abnormal degradation of the core spray or LPCI mode of the RHR has been observed at these plants.

4. The performance of these functional surveillance tests on a six-month interval requires that the reactor be shut down. In the period since Quad Cities Unit Nos. 1 and 2 were licensed for operation on October 1, 1971 and March 21, 1972, respectively, no failures of the logic systems have occurred. Individual components of the core spray system and LPCI mode of the RHR system have failed to perform as required. These failures were detected by the required surveillance tests of these components which have been and will continue to be performed during reactor operation on a more frequent basis. The frequency of component tests were chosen to be consistent with their observed failure rate.

Because the interval between failures of the logic system has been demonstrated by experience to be in excess of the interval between refueling outages, and the other considerations discussed above, we have concluded that it is acceptable to extend the surveillance interval for the core spray system and LPCI mode of the RHR system from semiannually to once each refueling interval.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 9, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-254 AND 50-265

COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 26 and 27 to Facility Operating License Nos. DPR-29 and DPR-30, respectively, issued to Commonwealth Edison Company (acting for itself and on behalf of the Iowa-Illinois Gas and Electric Company), which revised Technical Specifications for operation of the Quad Cities Station Unit Nos. 1 and 2 (the facilities) located in Rock Island County, Illinois. The amendments are effective as of their date of issuance.

The amendments changed the surveillance frequency of the core and containment cooling system logic system functional tests from once each six months to once per each refueling outage.

The application for these amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of the amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for the amendments dated January 9, 1976, (2) Amendment Nos. 26 and 27 to License Nos. DPR-29 and DPR-30, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Moline Public Library, 504 - 17th Street, Moline, Illinois 60625. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this ninth day of August, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors