Docket Nos. 50-254 and 50-265

Mr. Thomas J. Kovach Director of Nuclear Licensing Commonwealth Edison Company Post Office Box 767 Chicago, Illinois 60690 Distribution
Docket file
PDIII-2 r/f
GHolahan
DMuller
DHagan
BGrimes
WJones
ACRS (10)
ARM/LFMB
Plant file

NRC & Local PDRs LLuther MVirgilio TRoss OGC EJordan TMeek (8) EButcher GPA/PA

Dear Mr. Kovach:

SUBJECT:

AMENDMENT OF TECHNICAL SPECIFICATIONS SURVEILLANCE

REQUIREMENTS FOR TYPE A TESTS (TAC NOS. 72927 AND 72928)

Reference:

(1) Amendment application from R. Stols (CECo) to

T. E. Murley (NRR) dated April 11, 1989

The Commission has issued the enclosed Amendment Nos.  $118\,$  and  $114\,$  to Facility Operating License Nos. DPR-29 and DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2. These amendments are in response to reference (1).

These amendments revise the frequency requirement of Technical Specification 4.7.A.2.6 for Type A Containment Leak Rate surveillance testing to conform with the language of 10 CFR Part 50, Appendix J.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly <u>Federal Register</u> notices.

Sincerely.

Thierry Ross, Project Manager

Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 118 to License No. DPR-29

2. Amendment No. 114 to License No. DPR-30

3. Safety Evaluation

cc w/enclosures:
See next page

8906120199 890605 PDR ADOCK 05000254 PDC PDC

PDIII-2:LA PDIII-2:PM LLuther:dmj TRoss | F 5/12/89 5/17/89

SPLB for JCraig 5/18/89

PDITION:PD DMaller 5/14/89 OGC LIKE 1 H L+W 15 5/3/189 CPV



### UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

June 5, 1989

Docket Nos. 50-254 and 50-265

Mr. Thomas J. Kovach
Director of Nuclear Licensing
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Kovach:

SUBJECT:

AMENDMENT OF TECHNICAL SPECIFICATIONS SURVEILLANCE

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Thierry Ross, Project Manager Project Directorate III-2

Division of Reactor Projects - III,

IV, V and Special Projects

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2. Amendment No. 114 to License No. DPR-30

3. Safety Evaluation

cc w/enclosures: See next page Mr. Thomas J. Kovach Commonwealth Edison Company Quad Cities Nuclear Power Station Units 1 and 2

#### cc:

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## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118 License No. DPR-29

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

#### B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Paul C. Shemanski, Acting Director

Project Directorate III-2

Division of Reactor Projects - III, IV. V and Special Projects

Attachment: Changes to the Technical Specifications

Date of Issuance: June 5, 1989

# FACILITY OPERATING LICENSE NO. DPR-29

#### DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.7/4.7-4

**INSERT** 

3.7/4.7-4

#### QUAD-CITIES DPR-29

- 11.5 scf per hour for any one main steam isolation valve when tested at 25 psig.
- b. With the measured overall integrated containment leakage rate exceeding 0.75  $L_a$  or 0.75  $L_t$ , as applicable, restore the overall integrated leakage rate(s) to  $\leq$  0.75  $L_a$  or  $\leq$  0.75  $L_t$ , as applicable.

- c. With the measured combined leakage rate for all penetrations and valves, except for main steam isolation valves, subject to Type B and C tests exceeding 0.60 L, restore the combined leakage rate for all penetrations and valves, except for main steam isolation valves, subject to Type B and C tests to 0.60 La.
- b. If any periodic Type A test fails to meet either 0.75 L or 0.75 L, the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either 0.75 L or 0.75 L, a Type A test shall be performed at each shutdown for refueling or approximately every 18 months, whichever occurs first, until two consecutive Type A tests meet either 0.75 L or 0.75 L, at which time the above test schedule mâŷ be resumed.
- The accuracy of each Type A test shall be verified by a supplemental test which;
  - Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within 0.25 L or 0.25 L<sub>t</sub>.
  - 2) Has a duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.



## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

#### COMMONWEALTH EDISON COMPANY

AND

#### IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

#### QUAD CITIES NUCLEAR POWER STATION, UNIT 2

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 114 License No. DPR-30

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 11, 1989, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

#### B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 114, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Paul C. Shemanski, Acting Director

Project Directorate III-2 Division of Reactor Projects - III,

IV, V and Special Projects

Paul C. S. formarchi

Attachment: Changes to the Technical Specifications

Date of Issuance: June 5, 1989

#### ATTACHMENT TO LICENSE AMENDMENT NO. 114

#### FACILITY OPERATING LICENSE NO. DPR-30

#### DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the page identified below and inserting the attached page. The revised page is identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

INSERT

3.7/4.7-3

3.7/4.7-3

- b) b) 
  Lt, 
  0 percent
  by weight of the
  containment air per
  24 hours at a
  reduced pressure of
  Pt, 25 psig.
- 2) A combined leakage rate of  $\leq$  0.60  $L_a$  for all penetrations and valves, except for main steam isolation valves subject to Type B and C tests when pressurized to  $P_a$ .
- 3) 11.5 scf per hour for any one main steam isolation valve when tested at 25 psig.
- b. With the measured overall integrated containment feakage rate exceeding 0.75  $L_a$  or 0.75  $L_t$ , as applicable, restore the overall integrated leakage rate(s) to  $\leq$  0.75  $L_a$  or  $\leq$  0.75  $L_t$ , as applicable.
- c. With the measured combined leakage rate for all penetrations and valves, except for main steam isolation valves, subject to Type B and C tests exceeding 0.60 La, restore the combined leakage rate for all penetrations and valves, except for main steam isolation valves, subject to Type B and C tests to 0.60 La.
- d. Leakage shall be limited to a leakage rate of less than or equal to 3.75 percent of La for any one air lock when pressurized to 10 psig.

- b. If any iodic Type A test fails to meet either  $0.75 L_a$  or  $0.75 L_t$ , the test schedule for subsequent Type A tests shall be reviewed and approved by the Commission. If two consecutive Type A tests fail to meet either 0.75 La or 0.75 Lt, a Type A test shall be performed at each shutdown for refueling or approximately every 18 months, whichever occurs first, until two consecutive Type A tests meet either 0.75 La or 0.75 Lt, at which time the above test schedule may be resumed.
- c. The accuracy of each Type A test shall be verified by a supplemental test which:
  - Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within 0.25 La or 0.25 L+.
  - 2) Has a duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test.
  - 3) Requires the quantity of gas injected into the containment or bled from the containment during the supplemental test to be equivalent to at least 25 percent of the total measured leakage at Pa, 48 psig, or Pt, 25 psig.
- d. Type B and C tests shall be conducted at P<sub>a</sub>, 48 psig, at intervals no greater than 24 months except for tests involving:
  - 1) Air locks, which shall be tested at 10 psig at least once per 18 months, and Amendment No. 114



### UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. DPR-29

AND AMENDMENT NO. 114 TO FACILITY OPERATING LICENSE NO. DPR-30

#### COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-254/265

#### 1.0 INTRODUCTION

Current Technical Specifications (4.7.A.2.6) require performing a Type A Containment Leak Rate test at least every 18 months in the event that two consecutive Type A tests failed to meet specified acceptance criteria. Commonwealth Edison Company (CECo, the licensee) proposed an amendment that would require Type A testing at each shutdown for refueling or approximately every 18 months, whichever occurs first, in the event two consecutive tests failed to meet specified criteria.

#### 2.0 EVALUATION

The proposed amendment will change the language of current Quad Cities Technical Specification 4.7.A.2.b to conform with language contained in 10 CFR Part 50, Appendix J. The present language of the Quad Cities Technical Specifications requires a Type A retest at least every 18 months, if two consecutive Type A tests failed to meet either the 0.75 L or 0.75 L criteria. 10 CFR Part 50, Appendix J allows performing a Type A retest at each plant shutdown for refueling or approximately every 18 months, whichever occurs first. The proposed change will allow additional time between Type A retests over present Technical Specification requirements, but this additional time between tests is not significant and is consistent with 10 CFR Part 50, Appendix J.

The proposed change would allow sufficient flexibility to accommodate an 18-month fuel cycle and conduct the Type A retest during a refueling outage. This is consistent with provisions of 10 CFR Part 50, Appendix J which are intended to preclude unnecessary plant shutdowns (before a scheduled refueling outage) just to perform the Type A retest. Furthermore, there is no plant specific basis for requiring Quad Cities to comply with Type A surveillance frequencies more restrictive than prescribed by Appendix J. Consequently for these reasons, CECo's amendment request is considered acceptable.

#### 3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to the surveillance requirement of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of these amendments.

#### 4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Thierry Ross

Dated: June 5, 1989