BWR SRO Examination Outline

Facility: Limerick Generating Station Date of Exam: 05/04/2001 Exam Level: Tier Group K/A Category Points														
Tier	Group					K/A C	ategor	y Point	is				Point Total	
		К1	К2	КЗ	K4	К5	К6	A1	A2	А3	A4	G*		
1. Emergency &	1	4	5	5				4	5			3	26	
Abnormal Plant	2	4	3	2				1	7			0	17	
Evolutions	Tier Totals	8	8	7				5	12			3	43	
2. Plant	1 2 0 3 1 3 2 4 2 1 2 3 2 2 2 2 2 1 1 1 0 1 0 1												23	
Systems	2 2 2 2 2 1 1 0 1 0 1												13	
	3	0	0	0	0	0	1	1	1	1	0	0	4	
	Tier Totals	2	6	3	3	2	4	40						
3. Generic K	nowledge an	d Abi	lities		Ca	t 1	Ca	t 2	Ca	t 3	Cat	4	<u></u>	
					3	3	7	7	3	3	4		17	
 Note: 1. Ensure that at least two topics from every K/A category are sampled within each tier (i.e., the "Tier Totals" in each K/A category shall not be less than two). 2. Actual point totals must match those specified in the table. 3. Select topics from many systems; avoid selecting more than two or three K/A topics from a given system unless they relate to plant-specific priorities. 4. Systems/evolutions within each group are identified on the associated outline. 5. The shaded areas are not applicable to the category/tier. 6.* The generic K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. 7. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings for the RO license level, and the point totals for each system and category. K/As below 2.5 should be justified on the basis of plant-specific priorities. Enter the tier totals for each category in the table above. 														

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ES-401	•			BWF	R SRO	Exa	nination Outline	Form ES	401-1
	Em	erger	ncy a	nd Ab	norm	al Pla	nt Evolutions - Tier 1/Group 1		
E/APE # / Name / Safety Function	К1	K2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Pts
295003 Partial or Complete Loss of AC Pwr / VI			х				AK3.02 - Selective tripping (21)	3.1	1
295006 SCRAM / I		x					AK2.02 - Reactor water level control system (23)	3.8	
		ļ			X		AA2.05 - Whether a reactor SCRAM has occurred (51S)	4.6	1
295007 High Reactor Pressure / III		X					AK2.03 - RHR/LPCI: Plant-Specific (24)	3.2	1
295009 Low Reactor Water Level / II				x			AA1.02 - Reactor water level control (8)	4.0	1
295010 High Drywell Pressure / V	X						AK1.01 - Downcomer submergence: Mark-I&II (25)	3,4	1
·	ļ				X		AA2.02 - Drywell pressure (52S)	3.9	
295013 High Suppression Pool Temp. / V	X						AK1.03 - Localized heating (27)	3.3	1
295014 Inadvertent Reactivity Addition / I	X						AK1.06 - Abnormal reactivity additions (9)	3.9	1
295015 Incomplete SCRAM / I				х			AA1.01 - CRD hydraulics (28)	3.9	1
						x	2.1.2 - Knowledge of operator responsibilities during all modes of plant operation. (29)	4.0	1
295016 Control Room Abandonment / VII				х			AA1.07 - Control room/local control transfer mechanisms (30)	4.3*	1
						×	2.2.3 - (multi-unit) Knowledge of the design, procedural, and operational differences between units. (53S)	3.3	1
295017 High Off-site Release Rate / IX		x					AK2.10 - Process radiation monitoring system (31)	3.6	1
295023 Refueling Accidents Cooling Mode / VIII	ļ		X				AK3.02 - Interlocks associated with fuel handling equipment (33)	3.8*	1
295024 High Drywell Pressure / V			X				EK3.01 - Drywell spray operation Mark-I&II (34)	4.0	1
295025 High Reactor Pressure / III		X					EK2.01 - RPS (92)	4.1	1
295026 Suppression Pool High Water Temp. / V				X			EA1.01 - Suppression pool cooling (35)	4.1	1
a a second processing with the second and the second second second second second second second second second s					Х		EA2.02 - Suppression pool level (54S)	3.9	1
	the miles	en and	li de la compañía de						
295030 Low Suppression Pool Water Level / V					X		EA2.01 - Suppression pool Level (55S)	4.2*	1
295031 Reactor Low Water Level / II			х				EK3.02 - Core coverage (37)	4.7*	1
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / I			х				EK3.02 - SBLC injection (40)	4.5*	1
295038 High Off-site Release Rate / IX		X					EK2.03 - Plant ventilation systems (41)	3.8	1
						х	2.1.7 - Ability to evaluate plant performance and make operation judgments based on operating characteristics, reactor behavior, and instrument interpretation. (56S)	4.4	1
500000 High Containment Hydrogen Conc. / V	X						EK1.01 - Containment integrity (44)	3.9	1
					Х		EA2.01 - Hydrogen monitoring system availability (57S)	3.5	1
K/A Category Totals:	4	5	5	4	5	3	Group Point Total:		26

ES-401	Em	ergen	icy a	BWR nd Ab	SRO	Exar al Pla	nination Outline nt Evolutions - Tier 1/Group 2	Form ES	-401-1
E/APE # / Name / Safety Function	K1	К2	КЗ	A1	A2	G	K/A Topic(s)	Imp.	Pts
295001 Partial or Complete Loss of Forced Core Flow Circulation / I & IV	x						AK1.02 - Power/flow distribution (20)	3.5	1
295002 Loss of Main Condenser Vacuum / III					х		AA2.02 - Reactor Power: Plant-Specific (63S)	3.3	1
295004 Partial or Total Loss of DC Pwr / VI				х			AA1.02 - Systems necessary to assure safe plant shutdown (22)	4.1	1
295005 Main Turbine Generator Trip / III	х						AK1.03 - Pressure effects on reactor level (6)	3.7	1
295008 High Reactor Water Level / II	х						AK1.03 - Feed flow/steam flow mismatch (7)	3.2	1
2. Die & hansennen und Bringsladte. W						0.124.13			
295012 High Drywell Temperature / V		2	х				AK3.01 - Increased drywell cooling (26)	3.6	1
295018 Partial or Total Loss of CCW / VIII					х		AA2.05 - System pressure (58S)	2.9	- 1
295019 Partial or Total Loss of Inst. Air / VIII			х				AK3.03 - Service air isolation: Plant-Specific (10)	3.2	1
295020 Inadvertent Cont. Isolation / V & VII					х		AA2.06 - Cause of isolation (59S)	3.8	1
295021 Loss of Shutdown Cooling / IV	х						AK1.01 - Decay heat (32)	3.8	1
295022 Loss of CRD Pumps / I					х		AA2.02 - CRD system status (91)	3.4	1
295028 High Drywell Temperature / V		х					EK2.04 - Drywell ventilation (36)	3.6	1
295029 High Suppression Pool Water Level / V					х		EA2.01 - Suppression pool water level (60S)	3.9*	1
295032 High Secondary Containment Area Temperature / V					х		EA2.01 - Area temperature (61S)	3.8	1
295033 High Secondary Containment Area Radiation Levels / IX		х					EK2.03 - Secondary containment ventilation: Plant-Specific (38)	3.9	1
295034 Secondary Containment Ventilation High Radiation / IX					х		EA2.01 - Ventilation radiation levels (62S)	4.2	1
295035 Secondary Containment High Differential Pressure / V									
295036 Secondary Containment High Sump/Area Water Level / V		X .					EK2.01 - Secondary containment equipment and floor drain system (39)	3.2	1
600000 Plant Fire On Site / VIII									
K/A Category Point Totals:	4	3	2	1	7	0	Group Point Total:		17

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ES-401				l F	BWR Plant	SRO Syste	Exam ems -	inatio Tier 2	n Out /Grou	line p 1		For	rm ES-4	101-1
System # / Name	К1	К2	КЗ	К4	K5	К6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Pts
	16.200										K.			
202002 Recirculation Flow Control	X											K1.06 - A.C. electrical (78)	3.0	1
203000 RHR/LPCI: Injection Mode								х				A2.16 - Loss of coolant accident (96)	4.5*	1
206000 HPCI	g. 6j	3. 11. 11. 11.									x	2.4.10 - Knowledge of annunciator response procedures (80)	3.1	1
TOP OF DESIGN THE CASE OF STORE STORE			2.32.23							6				
209001 LPCS										-		2.4.45 - Ability to prioritize and interpret the significance of each annunciators or alarm. (64S)	3.5	1
Percian Contraction		anos est Ricelto - St												
211000 SLC			×									K3.01 - †Ability to shutdown the reactor in certain conditions (98)	4.4*	1
212000 RPS							х					A1.04 - RPS - bus voltage: Plant-Specific (81)	3.0	1
215004 Source Range Monitor	ļ	L			X							K5.01 - Detector operation (84)	2.6	1
215005 APRM / LPRM				X								K4.06 - Effects of detector aging on LPRM/APRM readings (85)	2.8	1
216000 Nuclear Boiler Instrumentation	<u> </u>				x							K5.02 - Vessel pressure measurement (86)	3.2	1
217000 RCIC			x									K3.02 - Reactor vessel pressure (100)	3.6	1
218000 ADS					х						·	K5.01 - ADS Logic (87)	3.8	1
223001 Primary CTMT and Auxiliaries							Х					A1.05 - Hydrogen concentration (89)	3.3	1
223002 PCIS/Nuclear Steam Supply Shutoff							X					A1.01 - System indicating lights and alarms (90)	3.5	1
226001 RHR/LPCI: CTMT Spray Mode							х					A1.03 - †Suppression chamber pressure: Mark I-II (11)	3.8	1
239002 SRV's						х						K6.04 - D.C. power: Plant-Specific(1)	3.2	1
											х	2.4.46 - Ability to verify that the alarms are consistent with the plant conditions. (65S)	3.6	1
241000 Reactor/Turbine Pressure Regulator						х						K6.01 - A.C. electrical power (14)	2.9	1
								х				A2.04 - Failed open/closed control/ governor valve (15)	3.6	1
259002 Reactor Water Level Control										х		A4.06 - DP/Single/three element control selector switch: Plant-Specific (3)	3.2	1
261000 SGTS	X											K1.07 - Elevated release stack (16)	3.2	1
262001 AC Electrical Distribution			х									K3.04 - Uninterruptible power supply (17)	3.3	1
264000 EDG's										х		A4.01 - Adjustment of exciter voltage (4)	3.4	1
290001 Secondary CTMT									х			A3.01 - Secondary containment isolation (5)	4.0	1
K/A Category Point Totals:	2	0	3	1	3	2	4	2	1	2	3	Group Point Total:		23

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ES-401	<u> </u>				BWR Plant	SRO Syste	Exam ems -	inatio Tier 2	n Out 2/Grou	ine p 2		For	m ES-4	01-1
System # / Name	К1	K2	КЗ	К4	К5	К6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Pts
201001 CRD Hydraulic		X										K2.03 - Backup SCRAM valve solenoids	3.6*	1
201002 RMCS		-							x	<u> </u>		A3.01 - Control red block actuation (77)	21	
												A stor - control too block actuation (77)	3.1	
201006 RWM		-					W EEKS - 1997 - 1884			and a second				
202001 Recirculation			x									K3.05 - Recirculation system MG sets: Plant-Specific (95)	3.3	1
204000 RWCU				x								K4.07 - Draining of reactor water to various locations (79)	2.9	1
205000 Shutdown Cooling	X											K1.01 - Reactor pressure (97)	3.6	1
214000 RPIS													0.0	
215002 RBM		X										K2.03 - APRM channels; BWR-3.4.5 (82)	2.9	1
215003 IRM											x	2.4.47 - Ability to diagnose and recognize trends in an accurate and timely manner utilizing the appropriate control room reference material. (83)	3.7	1
219000 RHR/LPCI: Torus/Pool Cooling Mode					х							K5.04 - Heat exchanger operation (88)	29	1
230000 RHR/LPCI: Torus/Pool Spray Mode														•
234000 Fuel Handling Equipment			X									K3.04 - †core modifications/alterations (13)	3.8	1
reskied. Brachta die east		2	kar ti i di Encirci data											
245000 Main Turbine Gen. And Auxiliaries													C C C C C C C C C C C C C C C C C C C	Phillippine Barry Street
259001 Reactor Feedwater				x								K4.11 - Recirculation runbacks: Plant- Specific (2)	3.5	1
262002 UPS (AC/DC)														
263000 DC Electrical Distribution														
271000 Offgas							X					A1.01 - Condenser Vacuum (18)	3.0	1
272000 Radiation Monitoring														· · · · · · · · · · · · · · · · · · ·
286000 Fire Protection														
290003 Control Room HVAC														
300000 Instrument Air	X											K1.04 - Cooling water to compressor (42)	2.9	1
400000 Component Cooling Water						x						K6.05 - Motors (43)	2.5	1
K/A Category Point Totals:	2	2	2	2	1	1	1	0	1	0	1	Group Point Total:		13

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ES-401				 	BWR Plant	SRO Syst	Exan ems -	ninatio Tier 2	n Out 2/Grou	line p 3		For	Form ES-401-1		
System # / Name	K1	К2	КЗ	К4	К5	К6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Pts	
201003 Control Rod and Drive Mechanism								X				A2.04 - Single control rod SCRAM (94)	3.6	1	
215001 Traversing In-core Probe							x					A1.03 - Valve status: Mark-I &II (Not-BWR 1) (99)	2.8	1	
233000 Fuel Pool Cooling and Cleanup									X			A3.02 - Pump trip(s) (12)	2.6	1	
239001 Main and Reheat Steam											<u> </u>				
256000 Reactor Condensate			<u> </u>						<u>†</u>						
268000 Radwaste				<u> </u>											
288000 Plant Ventilation				1		x						K6.01 - A.C. electrical (19)	27	1	
290002 Reactor Vessel Internals		1								<u> </u>	<u> </u>				
K/A Category Point Totals:	0	0	0	0	0	1	1	1	1	0	0	Group Point Total:	L	4	

Generic Knowledge and Abilities Outline (Tier 3)

Facility: Limeri	ck Generat	ting Station Date of Exam: 05/04/2000	Exam I	_evel: SRO
Category	K/A #	Торіс	Imp.	Points
Conduct of Operations	2.1.1	Knowledge of conduct of operations. (45)	3.8	1
	2.1.12	Ability to apply technical specifications for a system. (66S)	4.0	1
	2.1.34	Ability to maintain primary and secondary plant chemistry within allowable limits. (67S)	2.9	1
	Total			
Equipment	2227	Knowledge of the refueling process (46)		3
Control	2.2.27	Knowledge of the rendeling process. (46)	3.5	1
	2.2.22	Knowledge of limiting conditions form operations and safety limits. (47)	4.1	1
	2.2.23	Ability to track limiting conditions for operations. (68S)	3.8	1
	2.2.14	Knowledge of the process for making configurations changes. (69S)	3.0	1 ,
	2.2.8	Knowledge of the process for determining if the proposed change, test, or experiment involves an unreviewed safety question.(70S)	3.3	1
	2.2.20	Knowledge of the process for managing troubleshooting activities. (71S)	3.3	1
	2.2.25	Knowledge of bases in technical specifications for limiting conditions for operations and safety limits. (72S)	3.7	1
	Total			7
Radiation Control	2.3.1	Knowledge of 10 CFR 20 and related facility radiation control requirements. (48)	3.0	1
	2.3.9	Knowledge of the process for performing a containment purge. (73S)	3.2	1
	2.3.10	Ability to perform procedures to reduce excessive levels of radiation and guard against personnel exposure. (74S)	3.3	1
	Total			3
Emergency Procedures and Plan	2.4.4	Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures. (49)	4.3	1
	2.4.48	Ability to interpret control room indications to verify the status and operation of system, and understand how operator actions and directives affect plant and system conditions. (50)	3.8	1
	2.4.16	Knowledge of EOP implementation hierarchy and coordination with other support procedures. (76)	4.0	1
-	2.4.9	Knowledge of lower power / shutdown implications in accident (e.g. LOCA or loss of RHR) mitigation strategies. (75S)	3.9	1
	Total			4
Tier 3 Point Tot	al (SRO)			17

BWR RO Examination Outline

Facility: Limerick	Generating S	tatior	1	Da	te of	Exam	: 05,	/04/2	001		Exar	n Lev	el: RO
Tier	Group					K/A Ca	ategory	/ Point	S				Point Total
		К1	К2	КЗ	К4	К5	К6	A 1	A2	A3	A4	G*	
1. Emergency &	1	4	3	3				2	0			1	13
Abnormal Plant	2	4	6	3	Stary.			3	2			1	19
Evolutions	3	1	1	2				0	0			0	4
	Tier Totals	9	10	8				5	2			2	36
2. Plant Systems	1	3	2	3	3	3	2	3	3	2	2	2	28
	2	2	2	3	2	2	1	2	2	1	2	0	19
	3	0	0	1	0	0	1	1	0	1	0	0	4
	Tier Totals	5	4	7	5	5	4	6	5	4	4	2	51
3. Generic Kr	nowledge an	d Ab	oilities	<u> </u>	Cat	: 1	Cat	t 2	Ca	t 3	Ca	t 4	
					З		4		3	3	3	3	13
 Note: 1. Ensure that at least two topics from every K/A category are sampled within each tier (i.e., the "Tier Totals" in each K/A category shall not be less than two). 2. Actual point totals must match those specified in the table. 3. Select topics from many systems; avoid selecting more than two or three K/A topics from a given system unless they relate to plant-specific priorities. 4. Systems/evolutions within each group are identified on the associated outline. 5. The shaded areas are not applicable to the category/tier. 6.* The generic K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. 7. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings for the RO license level, and the point totals for each system and category. K/As below 2.5 should be justified on the basis of 													

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ES-401	Em	erger	ncy a	BW nd At	R RO	Exam al Pia	- nination Outline Int Evolutions - Tier 1/Group 1	Form ES-	401-2
E/APE # / Name / Safety Function	К1	K2	К3	A1	A2	G	K/A Topic(s)	Imp.	Pts
295005 Main Turbine Generator Trip / III	x						AK1.03 - Pressure effects on reactor level (6)	3.5	1
295006 SCRAM / I		x					AK2.02 - Reactor water level control system (23)	3.8	1
295007 High Reactor Pressure / III		x					AK2.03 - RHR/LPCI: Plant-Specific (24)	3.1	1
295009 Low Reactor Water Level / II				х			AA1.02 - Reactor water level control (8)	4.0	1
295010 High Drywell Pressure / V	X						AK1.01 - Downcomer submergence: Mark-I&II (25)	3.0	1
295014 Inadvertent Reactivity Addition / I	x						AK1.06 - Abnormal reactivity additions. (9)	3.8	1
295015 Incomplete SCRAM / I	T			X			AA1.01 - CRD hydraulics (28)	3.8	1
						×	2.1.2 - Knowledge of operator responsibilities during all modes of plant operation (29)	3.0	1
295024 High Drywell Pressure / V			х				EK3.01 - Drywell spray operation: Markl&II (34)	3.6	1
295025 High Reactor Pressure / III		X					EK2.01 - RPS (92)	4.1*	1
295031 Reactor Low Water Level / II	T		X				EK3.02 - Core Coverage (37)	4.4*	1
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / I			x				EK3.02 - SBLC injection (40)	4.3*	1
500000 High Containment Hydrogen Conc. / V	X						EK1.01 - Containment integrity (44)	3.3	1
K/A Category Totals:	4	3	3	2	0	1	Group Point Total:		13

ES-401	Em	erger	ncy a	BWP nd Abr	RO E	xam Plar	ination Outline nt Evolutions - Tier 1/Group 2	Form ES-4	01-2
E/APE # / Name / Safety Function	K1	К2	К3	A1	A2	G	K/A Topic(s)	lmn	Pte
295001 Partial or Complete Loss of Forced Core Flow Circulation / I & IV	x						AK1.02 - Power/Flow distribution (20)	3.3	1
295002 Loss of Main Condenser Vacuum / III		X					AK2.05 - Feedwater system (51R)	2.7	1
295003 Partial or Complete Loss of AC Pwr / VI			x				AK3.02 - Selective tripping (21)	2.9	1
295004 Partial or Complete Loss of DC Pwr / VI				х			AA1.02 - Systems necessary to assure safe plant shutdown (22)	3.8	1
295008 High Reactor Water Level / II	x						AK1.03 - Feed flow/steam flow mismatch (7)	3.2	
A SELECTION OF THE READER CONTRACTOR									
295012 High Drywell Temperature / V			х				AK3.01 - Increased drywell cooling (26)	3.5	1
295013 High Suppression Pool Temp. / V	X						AK1.03 - Localized heating (27)	3.0	
295016 Control Room Abandonment / VII				x			AA1.07 - Control room/local control transfer mechanisms (30)	4.2*	1
295017 High Off-site Release Rate / IX		x					AK2.10 - Process radiation monitoring system (31)	3.3	1
295018 Partial or Complete Loss of CCW / VIII		x					AK2.02 - Plant Operations (52R)	3.4	1
295019 Partial or Complete Loss of Inst. Air / VIII			х				AK3.03 - Service air isolations: Plant-Specific (10)	3.2	
295020 Inadvertent Cont. Isolation / V & VII	x						AK1.05 - Loss of Drywell/Containment cooling (53R)	3.3	
295022 Loss of CRD Pumps / I					x		AA2.02 - CRD system status (91)	3.3	1
295026 High Suppression Pool Water Temp. / V				х			EA1.01 - Suppression pool cooling (35)	4.1	
2216227 sight some normal function of					\$ 2				
295028 High Drywell Temperature / V		x					EK2.04 - Drywell Ventalation (36)	3.6	1
295029 High Suppression Pool Water Level / V					x		EA2.03 - Drywell/containment water level (54R)	3.4	1
295030 Low Suppression Pool Water Level / V									· · · · ·
295033 High Secondary Containment Area Radiation Levels / IX		х					EK2.03 - Secondary containment ventilation: Plant-Specific (38)	3.7	1
295034 Secondary Containment Ventilation High Radiation / IX						x	2.1.14 - Knowledge of systm status criteria which required the notification of plant personnel (55R)	2.5	1
295038 High Off-site Release Rate / IX					х		EA2.03 - Plant ventilation system (41)	3.6	
600000 Plant Fire On Site / VIII									┝╌─┤
K/A Category Point Totals:	4	6	3	3	2	1	Group Point Total:		19

ES-401	Em	erger	ncy a	BW nd At	R RO	Exan al Pia	nination Outline Int Evolutions - Tier 1/Group 3	Form ES	-401-2
E/APE # / Name / Safety Function	К1	K2	КЗ	A1	A2	G	K/A Topic(s)	Imp,	Pts
295021 Loss of Shutdown Cooling / IV	X						AK1.01 - Decay heat (32)	3.6	1
295023 Refueling Accidents / VIII			X				AK3.02 - Interlocks associated with fuel handling equipment (33)	3.4	1
295032 High Secondary Containment Area Temperature / V									
295035 Secondary Containment High Differential Pressure / V			x				EK3.02 - Secondary containment ventilation response (56R)	3.3	1
295036 Secondary Containment High Sump/Area Water Level / V		x					EK2.01 - Secondary containment equipment and floor drain system (39)	3.1	1
K/A Category Point Totals:	1	1	2	0	0	0	Group Point Total:		4

ES-401				 I	BWR Plant	RO Svst	Exami ems -	inatior Tier 2	ı Outli 2/Grou	ine ID 1		For	m ES-4	01-2
System # / Name	К1	К2	кз	К4	К5	к6	A1	A2	A3		G	K/A Topic(s)	Imp	Dea
201001 CRD Hydraulic		X							<u> </u>	1		K2.03 - Backup SCRAM valve solenoids	3.5*	1
				X	1						1	(93)	3.6	
												K4.04 - Scramming controlrods with inoperative SCRAM solenoid valves (back- up SCRAM valves (57R)		
201002 RMCS									Х			A3.01 - Control rod block actuation (77)	3.2	1
										la serve				
202002 Recirculation Flow Control	X											K1.06 - A.C. electrical (78)	2.9	1
203000 RHR/LPCI: Injection Mode	1				1			X				A2.16 - Loss of coolant accident (96)	4.4*	1
	X											K1.04 - Keep fill system (58R)	3.3	1
206000 HPCI						10110.001 10/00/2					×	2.4.10 - Knowledge of annunciator response procedures. (80)	3.0	1
				lân Petrala										
209001 LPCS	1	X										K2.03 - Initiation logic (59R)	2.9*	1
	a marine never		and the second		27137517257287287				X			A3.03 - System pressure (60R)	3.5	1
209002 HPCS				(4)(4)(9)	-1000			(BURKS	100.000	30.468				(Second
212000 885	ļ		<u>^</u>					ļ				K3.01 - †Ability to shutdown the reactor in certain conditions (98)	4.3*	1
212000 hr3			ļ				X					A1.04 - RPS bus voltage: Plant-Specific (81)	2.8*	1
215003 IRM											X	2.4.47 - Ability to diagnose and recognize	3.4	1
								:				utilizing the appropriate control room		
215004 SRM					х				·····	1		K5.01 - Detector operation (84)	26	1
215005 APRM / LPRM				×								K4.06 - Effects of detector aging on	2.6	1
216000 Nuclear Boiler Instrumentation					х							K5.02 - Vessel pressure measurement (86)	31	1
								X	1			A2.06 - Loss of power supply (61R)	29	
217000 RCIC			х							1		K3.02 - Reactor vessel pressure (100)	36	
218000 ADS					Х							K5.01 - ADS logic operation (87)	3.8	1
223001 Primary CTMT and Auxiliaries							х					A1.05 - Hydrogen concentration (89)	3.1	1
223002 PCIS/Nuclear Steam Supply Shutoff							х					A1.01 - System indicating lights and alarms (90)	3.5	1
239002 SRV's						х						K6.04 - D.C. power: Plant-Specific (1)	3.0	1
			х									K3.02 - Reactor over pressurization (62R)	4.2*	1
241000 Reactor/Turbine Pressure Regulator						Х						K6.01 - A.C. electrical power (14)	2.8	1
								X				A2.04 - Failed open/closed control/governor	3.7	1
259001 Reactor Feedwater				х								K4.11 - Recirculation runbacks: Plant-	3.5	1
259002 Reactor Water Level Control										x		A4.06 - DP/Single/three element control	3.1	1
261000 SGTS	x											K1 07 Elevented release start (10)	2.1	
264000 EDG's										x		A4.01 - Adjustment of exciter voltage (4)	3.1	
K/A Category Point Totals:	3	2	3	3	3	2	3	3	2	2	2	Group Point Total:	0.0	28

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ES-401 BWR RO Examination Outline Form ES-401-2 Plant Systems - Tier 2/Group 2								01-2						
System # / Name	К1	K2	К3	К4	K5	К6	A1	A2	A3	A4	G	K/A Topic(s)	Imp	Dte
201003 Control Rod and Drive Mechanism								x				A2.04 - Single control rod SCRAM (94)	3.5	1
17401101121 (State)														
201006 RWM				x								K4.01 - Insert blocks/errors: P-Spec(Not- BWR6) (63R)	3.4	1
202001 Recirculation			x									K3.05 - Recirculation system MG sets: Plant-Specific (95)	3.3	1
204000 RWCU				x								K4.07 - Draining of reactor water to various locations (79)	2.9	1
205000 Shutdown Cooling	x			['							1	K1.01 - Reactor Pressure (97)	3.6	
214000 RPIS				· · · · · ·		['		X				A2.01 - Failed reed switches (64R)	3.1	
215002 RBM		x		<u> </u>	<u> </u>	['						K2.03 - APRM channels: BWR-3,4,5 (82)	2.8	
219000 RHR/LPCI: Torus/Pool Cooling Mode		ļ	<u> </u>	<u> </u>	X	['						K5.04 - Heat exchanger operation (88)	2.9	1
226001 RHR/LPCI: CTMT Spray Mode							X					A1.03 - †Suppression chamber pressure: Mark-I-II (11)	3.5	1
230000 RHR/LPCI: Torus/Pool Spray Mode			<u> </u>	<u> </u>	\Box	\Box				1			<u> </u>	
239001 Main and Reheat Steam	Ī									x		A4.03 - System flow (65R)	3.5	1
245000 Main Turbine Gen. and Auxiliaries				<u> </u>	<u> </u>	<u> </u>							1	
256000 Reactor Condensate	Ţ		X		\Box	\Box				1		K3.05 - HPCI: Plant-Specific (66R)	3.3	1
262001 AC Electrical Distribution			x		<u> </u>						1	K3.04 - Uninterruptible power supply (17)	3.3	1
262002 UPS (AC/DC)				<u> </u>						1			<u> </u>	
263000 DC Electrical Distribution					x							K5.01 - Hydrogen generation during battery charging (67R)	2.6	1
271000 Offgas			<u> </u>				X	\Box				A1.01 - Condenser vacuum (18)	3.3	1
272000 Radiation Monitoring		x										K2.02 - Offgas radiation monitoring system (68R)	2.5	1
286000 Fire Protection			\Box											
290001 Secondary CTMT									х			A3.01 - Secondary containment isolation (5)	3.9	1
290003 Control Room HVAC			\Box							х		A4.02 - Fans (69R)	2.8	1
300000 Instrument Air	X											K1.04 - Cooling water to compressor (42)	2.8	1
400000 Component Cooling Water					Ē	X						K6.05 - Motors (43)	2.8	1
K/A Category Point Totals:	2	2	3	2	2	1	2	2	1	2	0	Group Point Total:		19

ES-401	BWR RO Examination Outline Plant Systems - Tier 2/Group 3						Form ES-401-2							
System # / Name	К1	K2	КЗ	К4	К5	К6	A1	A2	A3	A4	G	K/A Topic(s)	Imp.	Pts
215001 Traversing In-core Probe							x					A1.03 - Valve status: Mark-I&II(Not BWR1)(99)	2.6	1
233000 Fuel Pool Cooling and Cleanup									X			A3.02 - Pump trip(s) (12)	2.6	1
234000 Fuel Handling Equipment			x									K3.04 - †Core modifications/alterations (13)	2.9	1
Parteleter Ballenge voor henre secondaal														
268000 Radwaste												na na na na na kata na kata na		
288000 Plant Ventilation						x						K6.01 - A.C. electrical (19)	2.7	1
290002 Reactor Vessel Internals														† · · · · · · · · · · · · · · · · · · ·
K/A Category Point Totals:	0	0	1	0	0	1	1	0	1	0	0	Group Point Total		4

Generic Knowledge and Abilities Outline (Tier 3)

Facility: Lime	rick Genera	ting Station Date of Exam: 05/04/2001	Exam	Level: RO		
Category	K/A #	Торіс	lmp.	Points		
Conduct of Operations	2.1.1	Knowledge of conduct of operations. (45)	3.7	1		
	2.1.25	Ability to obtain and interpret station reference materials such as graphs, monographs, and tables which contain performance data. (70R)	2.8	1		
	2.1.10	Knowledge of conditions and limitations in the facility license. (71R)	2.7	1		
				<u> </u>		
	Total			3		
Equipment Control	2.2.27	Knowledge of the refueling process. (46)	2.6	1		
	2.2.22	Knowledge of limiting conditions for operations and safety limits. (47)	3.4	1		
	2.2.28	Knowledge of new and spent fuel movement procedures. (72R)	2.6	1		
	2.2.2	Ability to manipulate the console controls as required to operate the facility between shutdown and designated power levels. (73R)	4.0	1		
				······································		
	Total			4		
Radiation Control	2.3.1	Knowledge of 10 CFR 20 and related facility radiation control requirements. (48)	2.6	1		
	2.3.9	Knowledge of the process for performing a containment purge. (74R)	2.5	1		
	2.3.2	Knowledge of facility ALARA program. (75R)	2.5	1		
	Total			3		
Emergency Procedures and Plan	2.4.4	Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures. (49)	4.0	1		
	2.4.48	Ability to interpret control room indications to verify the status and operation of system, and understand how operator actions and directives affect plant and system conditions. (50)	3.5	1		
	2.4.16	Knowledge of EOP implementation hierarchy and coordination with other support procedures. (76)	3.0	1		
	Total					
3 Tim 2 D / (T) (100)						
Tier 3 Point Total (RO) 13						

Facility: Limerick Generating Station			Date of Examination: 05/04/2001		
Examina	tion Level RO		Operating Test Number:		
Administrative Topic/Subject Description		Describe method of evaluation: 1. ONE Administrative JPM, OR 2. TWO Administrative Questions			
A.1	Shift Turnover Question: Working Hour Restrictions (A-C-40)				
		Question: Shift Turnover Documentation	on (NOM-C-4.1)		
	Fuel Handling	Question: Inadvertent criticality during	re-load (ON-120)		
		Question: Continuous communication requirements (FH-105)			
A.2	Modifications	fications Question: Application of temporary alteration (MOD-C-7)			
		Question: Independent verification (NC	DM-C-9.1)		
A.3	Exposure Limits and Controls	and Question: Locked High Rad area controls (HP-C-202)			
		Question: Response to Personnel Con	tamination (HP-C-818)		
A.4	Emergency Response Facilities	Question: Activation of ERP facilities (E	ERP-140)		
		Question: Technical Support Center fu	nctions (ERP-700)		

Administrative Topics Outline

Facility:	Limerick Generating Stati	on_	Date of Examination: 05/04/2001			
Examinat	tion Level SRO		Operating Test Number:			
Administrative Topic/Subject Description		Describe method of evaluation: 1. ONE Administrative JPM, OR 2. TWO Administrative Questions				
A.1	A.1 Shift Staffing Question: Minimum Staf		Staffing (A-C-40)			
Ques		Question: Manipulations by Non-Operations Personnel (NOM-C-8.9)				
Fuel Handling Qu		Question: Core Component Transfer Authorization Sheet Revision (RE-C-40)				
		Question: Drywell Access during fuel tra	insfer (HP-300)			
A.2	Maintenance	Question: Evaluate troubleshooting valve	e manipulation effect on equipment (AG-CG-41)			
		Question: Restoration of T.S. Equipment	(NOM-C-8.5)			
A.3	A.3 Dose Control Question: Planned Sp		P-C-108)			
		Question: Administrative dose extension	(HP-C-106)			
A.4	E-Plan P.A.R.	Question: Determine the P.A.R. for given	set of conditions (ERP-200)			
		Question: Emergency notifications with re	educed communications capability (ERP-110)			

ES-301 Control Room Systems and Facility Walk-Through Test Outline Form ES-301-2

Facility: Limerick Generating Station	Date of Examin	nation: 05/	04/2001		
Exam Level: RO	Operating Test	No.:			
B.1 Control Room Systems	· · · · · · · · · · · · · · · · · · ·				
System / JPM Title		Type Code*	Safety Function		
a. Transfer D13 from 101 to 201		DAS	6		
b. Manually Initiate Standby Liquid Control (SLC)		DAS	1		
c. Shutdown Cooling Flow Adjustments - RHRSW Hi Ra	ıd	DAS	4		
d. Shift HPCI Suction from CST to the Suppression Poo	I	DS	2		
e. Main Turbine Bypass Valve Exercise Test		DS	3		
f. Manually Initiate a Control Room Radiation Isolation		DS	9		
g. Primary Containment N2 Make-up		NS	5		
B.2 Facility Walk-Through	· · · · · · · · · · · · · · · · · · ·				
a. Inadvertent Opening of a Relief Valve		DR	3		
b. Remote Alignment of LPCI valves for Fire Safe Shutd	own	DR	2		
c. Bypass Refuel Floor HVAC isolations and Re-start Ref HVAC (T-229)	fuel Floor	NRA	9		
* Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (A)Iternate path, (C)ontrol room, (S)imulator, (L)ow-Power, (R)CA					

ES-301 Control Room Systems and Facility Walk-Through Test Outline Form ES-301-2

Facility: Limerick Generating Station	Date of Examir	nation: 05/	04/2001					
Exam Level: SRO(I) Operating Test No.:								
B.1 Control Room Systems	B.1 Control Room Systems							
System / JPM Title		Type Code*	Safety Function					
a. Transfer D13 from 101 to 201	DAS	6						
b. Manually Initiate Standby Liquid Control (SLC)	DAS	1						
c. Shutdown Cooling Flow Adjustments - RHRSW H	DAS	4						
d. Shift HPCI Suction from CST to the Suppression	DS	2						
e. Main Turbine Bypass Valve Exercise Test DS								
f. Manually Initiate a Control Room Radiation Isolati	DS	9						
g. Primary Containment N2 Make-up NS								
B.2 Facility Walk-Through			<u> </u>					
a. Inadvertent Opening of a Relief Valve		DR	3					
b. Remote Alignment of LPCI valves for Fire Safe S	Shutdown	DR	2					
c. Bypass Refuel Floor HVAC isolations and Re-start Refuel Floor NAR 9 HVAC (T-229)								
* Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (A)Iternate path, (C)ontrol room, (S)imulator, (L)ow-Power, (R)CA								

ES-301 Control Room Systems and Facility Walk-Through Test Outline Form ES-301-2

Fac	ility: Limerick Generating Station	Date of Examination: 05	5/04/2001			
Exa	m Level: SRO(U)	Operating Test No.:				
B.1	Control Room Systems		<u></u>			
	System / JPM Title	Type Code*	Safety Function			
a.	Primary Containment N2 Make-up	NS	5			
b.	Main Turbine Bypass Valve Exercise Test	DS	3			
c.	Shutdown Cooling Flow Adjustments - RHRSW	/ Hi Rad DAS	4			
d.						
e.						
f.						
g.						
B.2 I	acility Walk-Through					
a.	Bypass Refuel Floor HVAC isolations and Re-st HVAC (T-229)	art Refuel Floor NAR	9			
b.	Remote Alignment of LPCI valves for Fire Safe	Shutdown DR	2			
c.						
 * Type Codes: (D)irect from bank, (M)odified from bank, (N)ew, (A)Iternate path, (C)ontrol oom, (S)imulator, (L)ow-Power, (R)CA 						

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Appendi	x D	5	Scenario Outline	Form ES-D-		
Facility:L(GS	Scen	ario No.: A1(2,3,4)	Op-Test No.:		
Examiner	rs:		Operators	S:		
Initial Cor	nditions: 95% F	ower, OPC	ON 1, "1A" Chlorine De	etector failed downscale.		
Turnover regulator	: 1A Chlorine D y action has no	etector faile t been identi	d downscale at the en fied (Tech Spec).	d of last shift. Required		
Event	Malf.	Event		Event		
No.	• No.	Type*		Description		
1	N/A	N (ALL)	Maximize Main Gen	erator Reactive Load		
2	MED124B	C (PRO/	Loss of Isophase Bu	is cooling due to loss of pow		
		SRO)	and failure of the sta	indby fan		
3	N/A	R (ALL)	Power drop to <20,0	ou amps		
4	MAD141C MAD141B	I (ALL)	SRV fails open elect	rically		
5	MRP029C MRP407C	M (ALL)	ATWS (electrical)			
6	MSL559	C (SRO/ RO)	Standby Liquid Cont	rol Rupture		
7	MCR411B	C (ALL)	CRD Flow Control V	alve fails closed		
				·		

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponet, (M)ajor

Appen	dix D
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Scenario Outline

raulily. LOS

S_____ Scenario No.: B1(2,3,4) Op-Test No.:_____

Examiners: _____ Operators: _____

Initial Conditions: 90% Power, OPCON 1, "B" EHC Pump Blocked and out of service for maintenance._____

Turnover: Place H_2O_2 analyzers in service for monthly reading.

Event	Malf.	Event	Event
No.	No.	Type*	Description
1	N/A	N (ALL)	Place H ₂ O ₂ analyzers in service for monthly reading
2	MNS160A	l (PRO/ SRO)	Inadvertent isolation of Instrument Gas (Tech Spec)
3	MED263B	C (ALL)	Loss of D12 Safeguard Bus
4	MRR504A	C (ALL)	"1A" Recirc Pump Trip / Shaft Failure
5	N/A	R (ALL)	Reduce power to 33% with control rods
6	MRR440A	M (ALL)	Recirc Loop Rupture
7	MRH171A	I (PRO/ SRO)	"1A" RHR pump fails to start

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponet, (M)ajor

Appendix D

Scenario Outline

Facility:LGS		Scenario No.: C1		Op-Test No.:	
Examiners:		Operators:		: 	
Initial Condit	tions: 100% Pov	wer	annel A1 and A2 RP	S Manual SCRAM Channels	
Functional T	est				
Event	Malf.	Event	Event		
1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1 1	N/A	N (ALL)	RPS Surveillance T	RPS Surveillance Test	
2	MRDF016F SIMINS MFLCPR 1.12	C (ALL)	Rod Scrams during RPS Test, Thermal Limit MFLCPR >1		
3	N/A	R (ALL)	Power Reduced to <80%		
4	MPR217A	I (ALL)	RBM Fails (Tech Spec)		
5	MRC465	M (ALL)	Steam Line break in RCIC Room		
6	MRC464A MRC464B	C (PRO/ SRO)	RCIC isolation valve	alves fail to isolate	
7	RRE002	C (ALL)	Steam leak migrate through a broken de	ites into the HPCI Room door seal	
8	MAD144D	C (SRO/ PRO)	"H" SRV fails to ope	en	

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponet, (M)ajor

Appendix D		Scenario Outline		Form ES	
Facility:L	GS	Scer	ario No.: D1 (Spare)	Op-Test No.:	
Examiners:		,	Operators: _		
Initial Cond	itions: 100% Pc	ower, OPCC			
Turnover:_S	Swap from "1A"	to "1B" RE	CW pump for planned n	naintenance	
Event	Malf.	Event	Event		
No.	No.	Type*	Description		
1	N/A	N (PRO/ SRO)	Swap RECW pumps for maintenance		
2	MFH564A	I (ALL)	Level Switch fails on 2A Feed Water Heater, causing a string isolation		
3	N/A	R (ALL)	Reduce power to <85%		
4	MED282A	C (ALL)	Loss of Division 1 DC (Tech Spec)		
5	MFW252	M (ALL)	"1A" Loop Feedwater line rupture inside containment		
6	MCS184A	I (SRO/ PRO)	"1A" Core Spray injection valve fails to open		
7	MSC183D	C (SRO/ PRO)	"1D" Core Spray Pum	p fails to auto start	
	1	1	1		