Docket Nos. 50-254 and 50-265

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EJordan, MNBB 3701 WJones, MNBB 7112 ACRS(10)

OC/LFMB NRC & Local PDRs

Downers Grove, Illinois 60515

Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 80626 AND 80627)

The Commission has issued the enclosed Amendment No. 131 to Facility Operating License No. DPR-29 and Amendment No. 126 to Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The amendments are in response to your application dated June 13, 1991.

The amendments return information to Table 4.2-1 that had been previously approved and inadvertently omitted in subsequent amendments.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By:

Leonard N. Olshan, Project Manager Project Directorate III-2 Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 131 to DPR-29
- Amendment No. 126 to DPR-30

Safety Evaluation

cc w/enclosures: See next page 9108200152 910814 PDR ADDCK 05000254 PDŘ

NRC FILE CENTER COPY

OFFICIAL RECORD COPY [AMENDMENT 80626/27] DOCUMENT NAME:

PD/PD OGC-WF1 Office: Surname: 7 /30 /91 Date:

Mr. Thomas J. Kovach Commonwealth Edison Company Quad Cities Nuclear Power Station Unit Nos. 1 and 2

cc: Mr. Stephen E. Shelton Vice President Iowa-Illinois Gas and Electric Company P. O. Box 4350 Davenport, Iowa 52808

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 131 License No. DPR-29

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated June 13, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

B. <u>Technical Specifications</u>

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 131, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard J. Barrett, Director Project Directorate III-2

Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 14, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 131

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT		
3.2/4.2-28	3.2/4.2-28		
3.2/4.2-29	3.2/4.2-29		

QUAD-CITIES DPR-29

TABLE 4.2-1 (Cont'd)

	trument nnel	Instrument Functional Test [2]	Calibration ^[2]	Instrument Check [2]			
RCI	C Isolation						
1. 2. 3.	Steamline high flow Turbine area high temperature Low reactor pressure	Once/3 months[9] Refueling outage Once/3 months	Once/3 months[9] Refueling outage Once/3 months	None None None			
HPCI Isolation							
1. 2.	Steamline high flow Steamline area high temperature	[1] [9][10] Refueling outage	[9][10] Refueling outage	None None			
3.	Low reactor pressure	[1] [10]	[10]	None			
Reactor Building Ventilation System Isolation and Standby Gas Treatment System Initiation							
1.	Refueling floor radiation monitors	[1]	Once/3 months	Once/day			
Steam Jet Air Ejector Off-Gas Isolation							
1.	Off-gas radiation monitors	[1] [4]	Refueling outage	Once/day			
Control Room Ventilation System Isolation							
1. 2. 3. 4.	Reactor low water level Drywell high pressure Main steamline high flow Toxic gas analyzers (chlorine, ammonia, sulphur dioxide)	[1] [1] [1] Once/month	Once/3 months Once/3 months Once/3 months Once/18 months	Once/day None Once/day Once/day			

Notes

- [1] Initially once per month until exposure hours (M as defined on Figure 4.1-1) are 2.0 X 10⁵; thereafter, according to Figure 4.1-1 with an interval not less than 1 month nor more than 3 months. The compilation of instrument failure rate data may include data obtained from other boiling water reactors for which the same design instrument operates in an environment similar to that of Quad Cities Units 1 and 2.
- [2] Functional tests, calibrations, and instrument checks are not required when these instruments are not required to be operable or tripped.

QUAD-CITIES DPR-29

TABLE 4.2-1 (Cont'd)

- [3] This instrumentation is excepted from the functional test definition. The function test shall consist of injecting a simulated electric signal into the measurement channel.
- [4] This instrument channel is excepted from the functional test definitions and shall be calibrated using simulated electrical signals once every 3 months.
- [5] Functional tests shall be performed before each startup with a required frequency not to exceed once per week. Calibrations shall be performed during each startup or during controlled shutdowns with a required frequency not to exceed once per week.
- [6] The positioning mechanism shall be calibrated every refueling outage.
- [7] Logic system functional tests are performed as specified in the applicable section for these systems.
- [8] Functional tests shall include verification of operation of the degraded voltage 5 minute timer and 7 second inherent timer.
- [9] Verification of the time delay setting of $3 \le t \le 9$ seconds shall be performed during each refueling outage.
- [10] Trip units are functionally tested monthly. A calibration of the trip unit is to be performed concurrent with the functional testing. Transmitters are calibrated once per operating cycle.



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 126 License No. DPR-30

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated June 13, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 126, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Richard J. Barrett, Director Project Directorate III-2

Division of Reactor Projects - III/IV/V Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: August 14, 1991

ATTACHMENT TO LICENSE AMENDMENT NO. 126

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.2/4.2-17

0/4 0 1

INSERT

3.2/4.2-17

QUAD-CITIES DPR-30

TABLE 4.2-1 (Cont'd)

	Inst <u>Chan</u>	rument nel	Instrument Functional Test [2]	Calibration ^[2]	Instrumen Check ^[2]	t	
	HPCI	Isolation					
	1. 2. 3.	Steamline high flow Steamline area high temperature Low reactor pressure	(1) (9) (10) Refueling outage (1) (10)	(9) (10) Refueling outage (10)	None None None	1	
Reactor Building Ventilation System Isolation and Standby Treatment System Initiation					itiation		
	1.	Refueling floor radiation monitors	(1)	Once/3 months	Once/day		
Steam Jet Air Ejector Off-Gas Isolation							
	1.	Off-gas radiation monitors	(1) (4)	Refueling outage	Once/day		
	Control Room Ventilation System Isolation						
	1. 2. 3. 4.	Reactor low water level Drywell high pressure Main steamline high flow Toxic gas analyzers (chlorine, ammonia, sulphur dioxide)	(1) (1) (1) Once/Month	Once/3 months Once/3 months Once/3 months Once/18 months	Once/day None Once/day Once/day		

Notes

- 1. Initially once per month until exposure hours (M as defined on Figure 4.1-1) are 2.0×10^5 ; thereafter, according to Figure 4.1-1 with an interval not less than 1 month nor more than 3 months. The compilation of instrument failure rate data may include data obtained from other boiling water reactors for which the same design instrument operates in an environment similar to that of Quad Cities Units 1 and 2.
- 2. Functional tests, calibrations, and instrument checks are not required when these instruments are not required to be operable or tripped.
- This instrumentation is excepted from the functional test definition. The function test shall consist of injecting a simulated electric signal into the measurement channel.
- 4. This instrument channel is excepted from the functional test definitions and shall be calibrated using simulated electrical signals once every 3 months.
- 5. Functional tests shall be performed before each startup with a required frequency not to exceed once per week. Calibrations shall be performed during each startup or during controlled shutdowns with a required frequency not to exceed once per week.
- 6. The positioning mechanism shall be calibrated every refueling outage.
- Logic system functional tests are performed as specified in the applicable section for these systems.
- Functional tests shall include verification of operation of the degraded voltage. 5 minute timer and 7 second inherent timer.
- 9. Verification of the time delay setting of $3 \le t \le 9$ seconds shall be performed during each refueling outage.
- 10. Trip units are functionally tested monthly. A calibration of the trip unit is to be performed concurrent with the functional testing. Transmitters are calibrated once per operating cycle.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO AMENDMENT NO. 131 TO FACILITY OPERATING LICENSE NO. DPR-29 AND AMENDMENT NO. 126 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

and

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-254 AND 50-265

1.0 INTRODUCTION

By letter dated June 13, 1991, Commonwealth Edison Company (the licensee) proposed changes to the Technical Specifications (TS) for Quad Cities Nuclear Power Station, Units 1 and 2. The proposed changes return information to Table 4.2-1 that had been previously approved and inadvertently omitted in subsequent amendments.

2.0 EVALUATION

Amendment No. 90 for Unit 1, dated August 2, 1984, (49 FR 33379) and Amendment No. 86 for Unit 2, dated May 30, 1985, (50 FR 25488) included changes to Table 4.2-1. These changes revised the calibration and functional testing requirements for High Pressure Coolant Injection (HPCI) isolation instrumentation for steam line high flow and low reactor pressure and added a note to further define testing requirements (Note 10). However, these changes were inadvertently omitted in later amendments.

Thus, by letter dated June 13, 1991, the licensee proposed an amendment that restores the information that was reviewed and approved in Amendment No. 90 for Unit 1, and Amendment No. 86 for Unit 2. The proposed amendments are, therefore, administrative in nature, and we find them acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (56 FR 31432). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: L. Olshan

Date: August 14, 1991