

May 13, 1993

Docket Nos. 50-254
and 50-265

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III, Suite 500
1400 OPUS Place
Downers Grove, Illinois 60515

DISTRIBUTION:
Docket File
PDIII-2 p/f
J. Zwolinski
C. Patel
OGC
G. Hill (8)
C. Grimes
OPA

B. Clayton, RIII
NRC & Local PDRs
J. Roe
J. Dyer
C. Moore
D. Hagan
W. Jones
ACRS (10)
OC/LFDCB

Dear Mr. Farrar:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M81082 AND M81083)

The Commission has issued the enclosed Amendment No. 141 to Facility Operating License No. DPR-29 and Amendment No. 136 to Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The amendments are in response to your applications dated August 29, 1991, September 30, 1991, and a portion of your application dated October 2, 1991.

The amendments delete Technical Specification (TS) Section 3.11/4.11, "High Energy Piping Integrity (Outside Containment)," TS Section 3.12/4.12, "Fire Protection Systems," the Fire Brigade manning requirements from TS 6.1.C, and changes the license conditions regarding fire protection.

Also, to address the capability for safe shutdown following a fire, by letter dated March 23, 1993, you proposed to submit the technical specifications for safe shutdown makeup pump as part of the ongoing Technical Specification Upgrade program. We find your proposal to be acceptable.

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By:

Chandu P. Patel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 141 to DPR-29
2. Amendment No. 136 to DPR-30
3. Safety Evaluation

cc w/enclosures:
See next page

190064

NRC FILE 50-254-1000

OFC	LA: PDIII-2	PM: PDIII-2	D: PDIII-2	SPLR	OGC
NAME	CMOORE	CPATEL	JDYER	CMCCRACKEN	dlc
DATE	4/13/92	4/12/92	5/13/92	4/20/93	5/13/92

9305240007 930513
PDR ADOCK 05000254
P PDR

Handwritten initials/signature in the bottom right corner.

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The amendments delete Technical Specification (TS) Section 3.11/4.11, "High Energy Piping Integrity (Outside Containment)," TS Section 3.12/4.12, "Fire Protection Systems," the Fire Brigade manning requirements from TS 6.1.C, and changes the license conditions regarding fire protection.

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Sincerely,

Original Signed By:

Chandu P. Patel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 141 to DPR-29
2. Amendment No. 136 to DPR-30
3. Safety Evaluation

cc w/enclosures:
See next page

OFC	LA: PD III-2	PM: PD III-2	D: PD III-2	SPL	OGC
NAME	CMOORE	CPATEL	JDYER	CMCCRACKEN	
DATE	4/13/92	4/12/92	5/13/92	4/20/93	5/13/92



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 13, 1993

Docket Nos. 50-254
and 50-265

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III, Suite 500
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Downers Grove, Illinois 60515

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The amendments delete Technical Specification (TS) Section 3.11/4.11, "High Energy Piping Integrity (Outside Containment)," TS Section 3.12/4.12, "Fire Protection Systems," the Fire Brigade manning requirements from TS 6.1.C, and changes the license conditions regarding fire protection.

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A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Chandu P. Patel

Chandu P. Patel, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 141 to DPR-29
2. Amendment No. 136 to DPR-30
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. D. L. Farrar
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Unit Nos. 1 and 2

cc:

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Vice President
Iowa-Illinois Gas and
Electric Company
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Davenport, Iowa 52808

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Sidley and Austin
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Chicago, Illinois 60690

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Station Manager
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Cordova, Illinois 61242

Resident Inspector
U. S. Nuclear Regulatory Commission
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Rock Island, Illinois 61201

Illinois Department of Nuclear Safety
Office of Nuclear Facility Safety
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Springfield, Illinois 62704

Regional Administrator, Region III
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Robert Neumann
Office of Public Counsel
State of Illinois Center
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Chicago, Illinois 60601



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 141
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Commonwealth Edison Company (the licensee) dated August 29, 1991, September 30, 1991, and October 2, 1991, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraphs 3.B. and 3.F of Facility Operating License No. DPR-29 are hereby amended to read as follows:

9305240012 930513
PDR ADDCK 05000254
P PDR

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 141, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

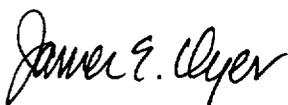
F. Fire Protection

Commonwealth Edison Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility and as approved in the Safety Evaluation Reports dated July 27, 1979 with supplements dated November 5, 1980 and February 12, 1981; December 30, 1982; December 1, 1987 with supplement dated April 20, 1988; December 11, 1987 with supplement dated July 21, 1988; and February 25, 1991, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Pages 4 and 5 of License
No. DPR-29*
Changes to the Technical
Specifications

Date of Issuance: May 13, 1993

*pages 4 and 5 have been retyped and are attached, for convenience, for the composite license to reflect this change

3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50 and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

A. Maximum Power Level

Commonwealth Edison is authorized to operate Quad-Cities Unit No. 1 at power levels not in excess of 2511 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- C. Deleted per Amendment No. 103, dated December 15, 1987.

D. Equalizer Valve Restriction

The valves in the equalizer piping between the recirculation loops shall be closed at all times during reactor operation.

- E. The licensee shall fully implement and maintain in effect all provisions of the Commission approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions of 10 CFR 73.55 (51 FR 27817 and 27822), and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). These plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Quad-Cities Nuclear Power Station Security Plan," with revisions submitted through July 15, 1992; "Quad-Cities Nuclear Power Station Security Personnel Training and Qualification Plan," with revisions submitted through July 15, 1992; and "Quad-Cities Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through July 15, 1992. Changes made in accordance with 10 CFR 73.55, shall be implemented in accordance with the schedule set forth therein.

- F. Commonwealth Edison Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility and as approved

in the Safety Evaluation Reports dated July 27, 1979 with supplements dated November 5, 1980 and February 12, 1981; December 30, 1982; December 1, 1987 with supplement dated April 20, 1988; December 11, 1987 with supplement dated July 21, 1988; and February 25, 1991, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

G. Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

H. Deleted by incorporation into 3.E above, per Amendment No. 64 dated March 19, 1981.

I. (OPEN)

J. Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel;
2. Procedures for monitoring; and
3. Provisions for maintenance of sampling and analysis equipment.

K. Deleted by Amendment No. 103 dated December 15, 1987.

ATTACHMENT TO LICENSE AMENDMENT NO. 141

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
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QUAD CITIES
DPR-29

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QUAD-CITIES
DPR-29

6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW, INVESTIGATION, AND AUDIT

- A. Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.
1. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of department responsibilities and relationships, and job descriptions for key personnel positions, or in the equivalent forms of documentation. These requirements shall be documented in the Quality Assurance Manual or the Management Plan for Nuclear Operations, Section 3 Organization Authority, Activities: Section 6 Interdepartmental Relationships.
 2. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
 3. The Senior Vice President-Nuclear Operations shall have the corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
 4. The Individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
- B. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, health physics personnel, equipment operators, and key maintenance personnel.
- The amount of overtime worked by Unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).
- C. The shift manning for the station shall be as shown in Figure 6.1-3. The Operating Assistant Superintendent, Operating Engineer, Shift Engineers, and Shift Foremen shall have a Senior Operator's License. The Fuel Handling Foreman has a limited Senior Operator's License. The Vice President BWR Operations on the corporate level has responsibility for the Fire Protection Program. The Maintenance Assistant Superintendent will be responsible for implementation of the Fire Protection Program.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 136
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Commonwealth Edison Company (the licensee) dated August 29, 1991, September 30, 1991, and October 2, 1991, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraphs 3.B. and 3.F of Facility Operating License No. DPR-30 are hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 136, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

F. Fire Protection

Commonwealth Edison Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility and as approved in the Safety Evaluation Reports dated July 27, 1979 with supplements dated November 5, 1980 and February 12, 1981; December 30, 1982; December 1, 1987 with supplement dated April 20, 1988; December 11, 1987 with supplement dated July 21, 1988; and February 25, 1991, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James E. Dyer, Director
Project Directorate III-2
Division of Reactor Projects - III/IV/V
Office of Nuclear Reactor Regulation

Attachment:
Pages 4, 5, and 6 of License
No. DPR-30*
Changes to the Technical
Specifications

Date of Issuance: May 13, 1993

*pages 4, 5, and 6 have been retyped and are attached, for convenience, for the composite license to reflect this change

3. This license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50 and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

A. Maximum Power Level

Commonwealth Edison is authorized to operate Quad-Cities Unit No. 2 at power levels not in excess of 2511 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

- C. Deleted per Amendment No. 104, dated June 17, 1988.

D. Equalizer Valve Restriction

The valves in the equalizer piping between the recirculation loops shall be closed at all times during reactor operation.

- E. The licensee shall fully implement and maintain in effect all provisions of the Commission approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions of 10 CFR 73.55 (51 FR 27817 and 27822), and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). These plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Quad-Cities Nuclear Power Station Security Plan," with revisions submitted through July 15, 1992; "Quad-Cities Nuclear Power Station Security Personnel Training and Qualification Plan," with revisions submitted through July 15, 1992; and "Quad-Cities Nuclear Power Station Safeguards Contingency Plan," with revisions submitted through July 15, 1992. Changes made in accordance with 10 CFR 73.55, shall be implemented in accordance with the schedule set forth therein.

- F. Commonwealth Edison Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility and as approved

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The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

G. Deleted by incorporation into 3.E above, per Amendment No. 58 dated March 19, 1981.

H. Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and
2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

I. Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel;
2. Procedures for monitoring; and
3. Provisions for maintenance of sampling and analysis equipment.

J. Deleted

K. Post-Accident Sampling

A program will be established, implemented and maintained which will ensure the capability to obtain and analyze reactor coolant,

radioactive iodines and particulates in plant chimney effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

1. Training of personnel,
 2. Procedures for sampling and analysis, and
 3. Provisions for maintenance of sampling and analysis equipment.
4. This license is effective as of the date of issuance, and shall expire at midnight, December 14, 2012.

FOR THE ATOMIC ENERGY COMMISSION

Original signed by:

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Enclosures: Appendices A and B -- Technical Specifications

Date of Issuance: December 14, 1972

ATTACHMENT TO LICENSE AMENDMENT NO. 136

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

INSERT

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iii

iv

iv

Section 3.11/4.11

-

Section 3.12/4.12

-

6.1-1

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3.11/4.11 HIGH ENERGY PIPING INTEGRITY - Deleted per Amendment No. 136 .	
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6.8 Offsite Dose Calculation Manual (ODCM)	6.8-1
6.9 Process Control Program (PCP)	6.9-1
6.10 Major Changes to Radioactive Waste Treatment Systems (Liquid, Gaseous, Solid)	6.10-1
6.11 Radiation Protection Program	6.11-1
6.12 High Radiation Area	6.12-1

6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW, INVESTIGATION, AND AUDIT

- A. Onsite and offsite organizations shall be established for unit operation and corporate management, respectively. The onsite and offsite organizations shall include the positions for activities affecting the safety of the nuclear power plant.
1. Lines of authority, responsibility, and communication shall be established and defined for the highest management levels through intermediate levels to and including all operating organization positions. These relationships shall be documented and updated, as appropriate, in the form of organization charts, functional descriptions of department responsibilities and relationships, and job descriptions for key personnel positions, or in the equivalent forms of documentation. These requirements shall be documented in the Quality Assurance Manual or the Management Plan for Nuclear Operations, Section 3 Organization Authority, Activities: Section 6 Interdepartmental Relationships.
 2. The Station Manager shall be responsible for overall unit safe operation and shall have control over those onsite activities necessary for safe operation and maintenance of the plant.
 3. The Senior Vice President-Nuclear Operations shall have the corporate responsibility for overall plant nuclear safety and shall take any measures needed to ensure acceptable performance of the staff in operating, maintaining, and providing technical support to the plant to ensure nuclear safety.
 4. The individuals who train the operating staff and those who carry out health physics and quality assurance functions may report to the appropriate onsite manager; however, they shall have sufficient organizational freedom to ensure their independence from operating pressures.
- B. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions; e.g., licensed Senior Operators, licensed Operators, health physics personnel, equipment operators, and key maintenance personnel.

The amount of overtime worked by Unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

- C. The shift manning for the station shall be as shown in Figure 6.1-3. The Operating Assistant Superintendent, Operating Engineer, Shift Engineers, and Shift Foremen shall have a senior operator's license. The Fuel Handling Foreman has a limited Senior Operator's License. The Vice President BWR Operations on the corporate level has responsibility for the Fire Protection Program. The Maintenance Assistant Superintendent will be responsible for implementation of the Fire Protection Program.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 141 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 136 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-254 AND 50-265

1.0 INTRODUCTION

By letter dated September 30, 1991, Commonwealth Edison Company (the licensee) proposed that the existing license conditions on fire protection be replaced with the standard condition noted in Generic Letter (GL) 86-10 and that the existing Technical Specification (TS) Section 3.12/4.12, "Fire Protection Systems" be deleted for Quad Cities Station, Units 1 and 2. Two enclosures, the proposed Final Safety Analysis Report (FSAR) revision and the fire protection implementing procedure, were inadvertently omitted from the September 30, 1991, letter and were sent by letter dated November 5, 1991. By letter dated October 2, 1991, the licensee proposed many changes to Section 6.0, "Administrative Controls." One of the changes deletes the fire brigade manning requirements from TS Section 6.1.C. By letter dated August 29, 1991, the licensee proposed that TS Section 3.11/4.11, "High Energy Piping Integrity (Outside Containment)" be deleted.

2.0 BACKGROUND

Following the fire at the Browns Ferry Nuclear Power Plant on March 22, 1975, the Commission undertook a number of actions to ensure that improvements were implemented in the Fire Protection Programs for all power reactor facilities. Because of the extensive modification of Fire Protection Programs and the number of open issues resulting from staff evaluations, a number of revisions and alterations occurred in these programs over the years. Consequently, licensees were requested by GL 86-10 to incorporate the final NRC approved Fire Protection Program into their FSARs. In this manner, the Fire Protection Program -- including the systems, the administrative and technical controls, the organization, and other plant features associated with fire protection -- would have a status consistent with that of other plant features described in the FSAR. In addition, the Commission concluded that a standard license condition, requiring compliance with the provisions of the Fire Protection Program as described in the FSAR, should be used to ensure uniform enforcement

of fire protection requirements. Finally, the Commission stated that with the requested actions, licensees may request an amendment to delete the fire protection TS that would now be unnecessary.

The licensees for the Callaway and Wolf Creek plants submitted lead-plant proposals to remove fire protection requirements from their TS. This action was an industry effort to obtain NRC guidance on an acceptable format for license amendment requests to remove fire protection requirements from the TS.

Section 3.11/4.11 was placed in the Technical Specifications on March 8, 1976, by Amendments 24 (Unit 1) and 23 (Unit 2). They were intended to remain in effect until the completion of modifications which would acceptably mitigate the consequences of high energy line breaks outside containment.

3.0 EVALUATION

Generic Letter 86-10 recommended the removal of fire protection requirements from the TS. Although a comprehensive Fire Protection Program is essential to plant safety, the basis for this recommendation is that many details of this program that are currently addressed in the TS can be modified without affecting nuclear safety. Such modifications can be made provided that there are suitable administrative controls over these changes. These details, that are presently included in TS and which are removed by this amendment, do not constitute performance requirements necessary to ensure safe operation of the facility and, therefore, do not warrant being included in TS. At the same time, suitable administrative controls ensure that there will be careful review and analysis by competent individuals of any changes in the Fire Protection Program including those technical and administrative requirements removed from the TS to ensure that nuclear safety is not adversely affected. These controls include: (1) the TS administrative controls that are applicable to the Fire Protection Program, (2) the license condition on implementation of, and subsequent changes to, the Fire Protection Program, and (3) the 10 CFR 50.59 criteria for evaluating changes to the Fire Protection Program as described in the FSAR.

The TS changes proposed by the licensee are in accordance with the guidance provided by GL 88-12, as addressed in the items below.

(1) Specification 3.12 and 4.12, fire suppression systems, their associated Surveillance Requirements, and Bases (including fire barriers and fire detection instrumentation, and their associated Surveillance Requirements and Bases) were removed.

(2) The portion of Specification 6.1.C dealing with fire brigade staffing requirements was removed. Other administrative technical specifications were approved as part of Amendment Nos. 135 and 131 for Units 1 and 2, respectively, on July 24, 1992.

The licensee proposed to change license condition 3.F to read as follows:

Commonwealth Edison Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility and as approved in the Safety Evaluation Reports dated July 27, 1979 with supplements dated November 5, 1980 and February 12, 1981; December 30, 1982; December 1, 1987 with supplement dated April 20, 1988; December 11, 1987 with supplement dated July 21, 1988; and February 25, 1991, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

As required by GL 86-10, the licensee confirmed that the NRC approved Fire Protection Program has been incorporated into the FSAR. Also, the licensee has proposed that the existing licensing conditions on the Fire Protection Program be replaced with the standard condition noted above in accordance with GL 86-10.

Also, GL 88-12 requested that any specifications related to the capability for safe shutdown following a fire are to be retained in the TS. In the original submittal the licensee stated that Quad Cities, Units 1 and 2, TS do not contain requirements concerning the capability for safe shutdown following a fire. However, after some discussion with the staff, by letter dated March 23, 1993, the licensee proposed to submit the TS for the safe shutdown makeup pump as part of the ongoing Technical Specification Upgrade Program. The staff finds the licensee's proposal to be acceptable.

On the bases of its review of the above items, the staff concludes that the licensee has met the guidance of GL 88-12 and GL 86-10. Therefore, the staff finds the proposed changes acceptable.

The proposed change in the August 29, 1991, submittal deletes Section 3.11/4.11. Modifications were completed in 1979 which would acceptably mitigate the consequences of high energy line breaks outside containment and eliminate the need for Section 3.11/4.11. Therefore, the staff also finds this proposed change acceptable.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public

comment on such finding (57 FR 37561). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

On the considerations discussed above, the staff concludes that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Date: May 13, 1993