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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DO NOT REMOVE

Docket Nos. 50-237
50-249
50-254
50-265

July 9, 1982

Posted
Amdt. 80
to DPR-29

Mr. L. DelGeorge
Director of Nuclear Licensing
Commonwealth Edison Company
P. O. Box 767
Chicago, Illinois 60690

Dear Mr. DelGeorge:

The Commission has issued the enclosed Amendment No. 73 to Provisional Operating License No. DPR-19 for Dresden Nuclear Power Station, Unit 2 and Amendment Nos. 65, 80 and 74 for Facility Operating License Nos. DPR-25, DPR-29 and DPR-30 for Dresden Nuclear Power Station Unit 3 and Quad Cities Nuclear Power Station, Units 1 and 2, respectively. These amendments consist of changes to the Technical Specifications (TS) in response to your letter dated April 29, 1982 and, for Dresden 3 only, by letter dated May 25, 1982.

The amendments authorize deletion of the surveillance requirement to demonstrate the operability of the automatic pressure relief subsystem on a daily basis when the high pressure coolant injection (HPCI) system is inoperable. The existing TS requirement to demonstrate the operability of the automatic pressure relief subsystem immediately upon declaring the HPCI inoperable is not affected by this change.

The amendments also correct a typographical error previously issued in Amendment No. 59 to DPR-25 for Dresden Station, Unit 3 pertaining to a reactor protection system bypass permissive setpoint, and correct the specified surveillance frequency pertaining to auto-initiation of relief valve testing for Quad Cities Station, Units 1 and 2.

Copies of our Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Joseph D. Hegner, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

1. Amendment No. 73 to DPR-19
2. Amendment No. 65 to DPR-25
3. Amendment No. 80 to DPR-29
4. Amendment No. 74 to DPR-30
5. Safety Evaluation
6. Notice

cc w/encls:
See next page

Mr. L. DelGeorge

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Mr. L. DelGeorge

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237..

DRESDEN NUCLEAR POWER STATION UNIT 2
AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 73
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 29, 1982 complies with standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Provisional Operating License No. DPR-19 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 73, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the
Technical Specifications

Date of Issuance: July 9, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 73

PROVISIONAL OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Remove

p. 77

Insert

p. 77

3.5 LIMITING CONDITION FOR OPERATION

containment cooling subsystem, both core spray subsystems and both diesel generators required for operation of such components if no external source of power were available, shall be operable.

4. If the requirements of 3.5.B cannot be met an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown condition within 24 hours.

C. HPCI Subsystem

1. Except as specified in 3.5.C.2 below, the HPCI subsystem shall be operable whenever the reactor pressure is greater than 90 psig and irradiated fuel is in the reactor vessel.
2. From and after the date that the HPCI subsystem is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such subsystem is sooner made operable, provided that during such seven days all active components of the Automatic Pressure Relief Subsystem, the core spray subsystems, LPCI subsystem, and isolation cooling system are operable.
3. If the requirements of 3.5.C cannot be met an orderly shutdown shall be initiated and the reactor pressure shall be reduced to 90 psig within 24 hours.

4.5 SURVEILLANCE REQUIREMENT**C. Surveillance of HPCI Subsystem shall be performed as follows:**

1. HPCI Subsystem Testing shall be as specified in 4.5.A.1.a, b, c, d, and f, except that the HPCI pump shall deliver at least 5000 gpm against a system head corresponding to a reactor vessel pressure of 1150 psig to 150 psig.
2. When it is determined that HPCI subsystem is inoperable, the LPCI subsystem, both core spray subsystems, the automatic pressure relief subsystem, and the motor operated isolation valves and shell side make-up system for the isolation condenser system shall be demonstrated to be operable immediately. The motor operated isolation valves and shell side make-up system of the isolation condenser shall be demonstrated to be operable daily thereafter. Daily demonstration of the automatic pressure relief subsystem operability is not required provided that two feedwater pumps are operating at power levels above 300 MWe; and one feedwater pump is operating as normally required with one additional feedwater pump operable at power levels less than 300 MWe.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 65
License No. DPR-25

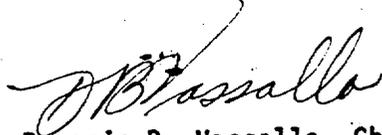
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 29, 1982, as supplemented by letter dated May 25, 1982 complies with standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 65, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 9, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 65

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-249

Remove

p. 24
p. 77

Insert

p. 24
p. 77

Minimum of Operable Inst. Channels per Trip (1) System	Trip Function	Trip Level Setting	Modes in Which Function Must be Operable			Action*
			Refuel (7)	Startup/Hot Standby	Run	
2	Turbine Condenser Low Vacuum	>23 in. Hg Vacuum	X(3)	X(3)	X	A or C
2	Main Steam Line High Radiation	<3 X Normal Full Power Background	X(3)	X(3)	X	A or C
4(6)	Main Steam Line Isolation Valve Closure	<10% Valve Closure	X(3)	X(3)	X	A or C
2	Generator Load Rejection	****	X(4)	X(4)	X(4)	A or C
2	Turbine Stop Valve Closure	<10% Valve Closure	X(4)	X(4)	X(4)	A or C
2	Turbine Control - Loss of Control Oil Pressure	>900 psig	X	X	X	A or C

Notes:

1. There shall be two operable or tripped trip systems for each function.
2. Permissible to bypass, with control rod block, for reactor protection system reset in refuel and shutdown positions of the reactor mode switch.
3. Permissible to bypass when reactor pressure is <600 psig.
4. Permissible to bypass when first stage turbine pressure less than that which corresponds to 45% rated steam flow.
5. IRM's are bypassed when APRM's are onscale and the reactor mode switch is in the run position.
6. The design permits closure of any one valve without a scram being initiated.
7. When the reactor is subcritical and the reactor water temperature is less than 212°F, only the following trip functions need to be operable:
 - a. Mode Switch in Shutdown
 - b. Manual Scram
 - c. High Flux IRM
 - d. Scram Discharge Volume High Level
8. Not required to be operable when primary containment integrity is not required.
9. Not required while performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed 5 MW(t).

3.5 LIMITING CONDITION FOR OPERATION

containment cooling subsystem, both core spray subsystems and both diesel generators required for operation of such components if no external source of power were available, shall be operable.

4. If the requirements of 3.5.B cannot be met an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown condition within 24 hours.

C. HPCI Subsystem

1. Except as specified in 3.5.C.2 below, the HPCI subsystem shall be operable whenever the reactor pressure is greater than 90 psig and irradiated fuel is in the reactor vessel.
2. From and after the date that the HPCI subsystem is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such subsystem is sooner made operable, provided that during such seven days all active components of the Automatic Pressure Relief Subsystem, the core spray subsystems, LPCI subsystem, and isolation cooling system are operable.
3. If the requirements of 3.5.C cannot be met an orderly shutdown shall be initiated and the reactor pressure shall be reduced to 90 psig within 24 hours.

4.5 SURVEILLANCE REQUIREMENT**C. Surveillance of HPCI Subsystem shall be performed as follows:**

1. HPCI Subsystem Testing shall be as specified in 4.5.A.1.a, b, c, d, and f, except that the HPCI pump shall deliver at least 5000 gpm against a system head corresponding to a reactor vessel pressure of 1150 psig to 150 psig.
2. When it is determined that HPCI subsystem is inoperable, the LPCI subsystem, both core spray subsystems, the automatic pressure relief subsystem, and the motor operated isolation valves and shell side make-up system for the isolation condenser system shall be demonstrated to be operable immediately. The motor operated isolation valves and shell side make-up system of the isolation condenser shall be demonstrated to be operable daily thereafter. Daily demonstration of the automatic pressure relief subsystem operability is not required provided that two feedwater pumps are operating at power levels above 300 MWe; and one feedwater pump is operating as normally required with one additional feedwater pump operable at power levels less than 300 MWe.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
AND
IOWA ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 29, 1982 complies with standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-29 is hereby amended to read as follows:

3. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 80, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "D. B. Vassallo".

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 9, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 80

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Remove

p. 3.5/4.5-5

Insert

p. 3.5/4.5-5

provided that during such 7 days all active components of the automatic pressure relief subsystems, the core spray subsystems, LPCI mode of the RHR system, and the RCIC system are operable.

3. If the requirements of Specification 3.5.C cannot be met, an orderly shutdown shall be initiated, and the reactor pressure shall be reduced to 90 psig within 24 hours.

D. Automatic Pressure Relief Subsystems

1. The automatic pressure relief subsystem shall be operable whenever the reactor pressure is greater than 90 psig, irradiated fuel is in the reactor vessel and prior to reactor startup from a cold condition.
2. From and after the date that one of the five relief valves of the automatic pressure relief subsystem is made or found to be inoperable when the reactor is pressurized above 90 psig with irradiated fuel in the reactor vessel, reactor operation is permissible only during the succeeding 7 days unless repairs are made and provided that during such time the HPCI subsystem is operable.
 - 2.a Plant operation shall be in accordance with 3.5.D.2 above except that, for the current operating cycle 5, four of the five relief valves of the ADS are required to be operable. In subsequent operating cycles, operation shall be in accordance with 3.5.D.2.
3. If the requirements of Specification 3.5.D cannot be met, an orderly shutdown shall be initiated and the reactor pressure shall be reduced to 90 psig within 24 hours.

operable immediately. The RCIC system shall be demonstrated to be operable daily thereafter. Daily demonstration of the automatic pressure relief subsystem operability is not required provided that two feedwater pumps are operating at levels above 300 MWe; and one feedwater pump is operating as normally required with one additional feedwater pump operable at power levels less than 300 MWe.

D. Automatic Pressure Relief Subsystems

Surveillance of the automatic pressure relief subsystems shall be performed as follows:

1. The following surveillance shall be carried out on a six-month surveillance interval:
 - a. With the reactor at pressure each relief valve shall be manually opened. Relief valve opening shall be verified by a compensating turbine bypass valve or control valve closure.
2. A logic system functional test shall be performed each refueling outage.
3. A simulated automatic initiation which opens all pilot valves shall be performed each refueling outage.
4. When it is determined that one relief valve of the automatic pressure relief subsystem is inoperable, the HPCI shall be demonstrated to be operable immediately and weekly thereafter.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
AND
IOWA ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES STATION UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 29, 1982 complies with standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-30 is hereby amended to read as follows:
 - B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 74, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "D. Vassallo", written in black ink.

Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 9, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Remove

p. 3.5/4.5-5

Insert

p. 3.5/4.5-5

QUAD-CITIES
DPR-30

provided that during such 7 days all active components of the automatic pressure relief subsystems, the core spray subsystems, LPCI mode of the RHR system, and the RCIC system are operable.

3. If the requirements of Specification 3.5.C cannot be met, an orderly shut-down shall be initiated, and the reactor pressure shall be reduced to 90 psig within 24 hours.

D. Automatic Pressure Relief Subsystems

1. The automatic pressure relief subsystem shall be operable whenever the reactor pressure is greater than 90 psig, irradiated fuel is in the reactor vessel and prior to reactor startup from a cold condition.
2. From and after the date that one of the five relief valves of the automatic pressure relief subsystem is made or found to be inoperable when the reactor is pressurized about 90 psig with irradiated fuel in the reactor vessel, reactor operation is permissible only during the succeeding 7 days unless repairs are made and provided that during such time the HPCI subsystem is operable.
3. If the requirements of Specification 3.5.D cannot be met, an orderly shut-down shall be initiated and the reactor pressure shall be reduced to 90 psig within 24 hours.

operable immediately. The RCIC system shall be demonstrated to be operable daily thereafter. Daily demonstration of the automatic pressure relief subsystem operability is not required provided that two feedwater pumps are operating at levels above 300 MWe; and one feedwater pump is operating as normally required with one additional feedwater pump operable at power levels less than 300 MWe.

D. Automatic Pressure Relief Subsystems

Surveillance of the automatic pressure relief subsystems shall be performed as follows:

1. The following surveillance shall be carried out on a 6 month surveillance interval:
 - a. With the reactor at pressure each relief valve shall be manually opened. Relief valve opening shall be verified by a compensating turbine bypass valve or control valve closure.
2. A logic system functional test shall be performed each refueling outage.
3. A simulated automatic initiation which opens all pilot valves shall be performed each refueling outage.
4. When it is determined that one relief valve of the automatic pressure relief subsystem is inoperable, the HPCI shall be demonstrated to be operable immediately and weekly thereafter.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 73 TO PROVISIONAL OPERATING LICENSE NO. DPR-19
AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. DPR-25, AND
AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. DPR-29, AND
AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DRESDEN NUCLEAR POWER STATION, UNIT 2

DRESDEN NUCLEAR POWER STATION, UNIT 3

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

DOCKET NOS. 50-237, 50-249, 50-254, 50-265

Author: J. Hegner

1. Introduction

By letter dated April 29, 1982 the Commonwealth Edison Company (the licensee) proposed a Technical Specification (TS) change to the Dresden Station, Units 2 and 3 and Quad Cities Station, Units 1 and 2 TS to delete that portion of the surveillance requirements that requires the automatic pressure relief subsystem be demonstrated operable daily upon declaring the high pressure coolant injection (HPCI) system inoperable.

2. Background

The Dresden and Quad Cities Stations' TS currently require that certain ECCS systems, including the automatic pressure relief subsystem, be demonstrated operable immediately and daily thereafter when it is determined that the HPCI is inoperable. The change requested is to delete only the repetitive daily requirement to cycle each automatic pressure relief subsystem valve for the remaining six days in which the affected reactor unit is permitted by the Technical Specifications to operate with the HPCI inoperable.

On two previous occasions (Reference: Amendment No. 65 to DPR-19 for Dresden Unit 2 and Amendment No. 57 to DPR-25 for Dresden Unit 3) the licensee has been granted temporary relief from the provisions of this TS on an emergency basis.

3. Evaluation

The licensee has proposed that the portion of the surveillance test requirement that requires daily testing of the automatic pressure relief subsystem be deleted. The licensee shall still be required to demonstrate the operability of the automatic pressure relief subsystem immediately upon declaring the HPCI inoperable.

Based on a review of BWR operating experience, it appears that the original TS surveillance requirement is overly restrictive in that the review identified certain undesirable consequences that can occur as a result of repeated testing of the automatic pressure relief subsystem. There have been instances in the past in which relief valves have failed to reseal after opening, and the testing of these valves causes repeated significant thermal and mechanical shocks to the reactor vessel and suppression chamber.

Based on this review, we have concluded that performance of the surveillance test immediately upon declaring the HPCI inoperable adequately demonstrates the operability of the automatic pressure relief subsystem. We have also concluded that the increased probability of having one of the pressure relief valves stick open during the repeated daily testing outweighs the benefits gained by such testing and that the safety of the facility will not be adversely affected by deleting the requirement to test these valves daily after the initial demonstration of their operability.

As a compensatory measure for the inoperable HPCI system, the licensee has proposed and will be required to have the feedwater pumps operable during the same period of permitted operation. The operability of the feedwater pumps will provide assurance, as long as offsite power is available, that a high pressure makeup capability is available for transients that do not cause feedwater isolation.

Based on this evaluation and on similar evaluations conducted by the staff in issuing Amendment No. 65 to DPR-19 for Dresden Unit 2, dated September 14, 1981 and Amendment No. 57 to DPR-25 for Dresden Unit 3, dated December 23, 1981, we find the licensee's proposed TS change to be acceptable.

4. RPS Bypass Permissive Setpoint Error (Dresden Unit 3)

By letter dated May 25, 1982 the licensee informed the staff that Note No. 3 on page 24 to the TS for DPR-25 (changed in Amendment No. 59) was incorrect due to a typographical error in the page provided by the licensee in its March 18, 1982 submittal. Specifically, the note should read: "Permissible to bypass when reactor pressure is < 600 psig." The note was issued as "... < 100 psig." The staff has confirmed that 600 psig is the correct setpoint and will amend the note accordingly.

5. Surveillance Frequency Error for ADS testing (Quad Cities 1 and 2)

The licensee informed the staff by letter dated April 29, 1982 that TS 3.5.D.1 to DPR-29 and 30 incorrectly specifies a surveillance test frequency pertaining to auto-initiation of the ADS valves every six months. The test is intended to be performed during refueling outages. We have confirmed that each refueling outage is the correct frequency and will amend the specifications accordingly.

6. Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

7. Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 9, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSION
DOCKET NOS. 50-237, 50-249, 50-254 AND 50-265

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO
OPERATING LICENSES

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 73 to Provisional Operating License No. DPR-19, and Amendment No. 65 to Facility Operating License No. DPR-25, issued to Commonwealth Edison Company, which revised the Technical Specifications for operation of the Dresden Nuclear Power Station, Unit Nos. 2 and 3, located in Grundy County, Illinois. The Commission has also issued Amendment No. 80 to Facility Operating License No. DPR-29, and Amendment No. 74 to Facility Operating License No. DPR-30, issued to Commonwealth Edison Company and Iowa-Illinois Gas and Electric Company, which revised the Technical Specifications for operation of the Quad Cities Nuclear Power Station, Unit Nos. 1 and 2, located in Rock Island County, Illinois. The amendments are effective as of the date of issuance.

The amendments authorize deletion of the surveillance requirement to demonstrate the operability of the automatic pressure relief subsystem on a daily basis when the high pressure coolant injection system (HPCI) is inoperable. The existing requirement to demonstrate the operability of the automatic pressure relief subsystem immediately upon declaring the HPCI inoperable remains unchanged. The amendments also correct several previously amended pages which contained typographical and editorial errors.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the

Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated April 29, 1982, and, for Dresden Unit 3 only, a letter dated May 25, 1982 (2) Amendment No. 73 to License No. DPR-19, Amendment No. 65 to License No. DPR-25, Amendment No. 80 to License No. DPR-29 and Amendment No. 74 to License No. DPR-30, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the the Morris Public Library, 604 Liberty Street, Morris, Illinois, for Dresden 2 and 3 and at the Moline Public Library, 504-17th Street, Moline, Illinois, for Quad Cities 1 and 2. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 9th day of July 1982.

FOR THE NUCLEAR REGULATORY COMMISSION



Domenic B. Vassallo, Chief
Operating Reactors Branch #2
Division of Licensing