

March 26, 1999

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. MA4296 AND MA4297)

Dear Mr. Kingsley:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 186 to Facility Operating License No. DPR-29 and Amendment No. 183 to Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The amendments are in response to your application dated November 30, 1998.

The amendments change the technical specifications (TSs) by decreasing the Allowed Outage Time (AOT) from 67 days to 14 days for the Safe Shutdown Makeup Pump (SSMP).

A copy of the related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIG. SIGNED BY
Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-254 and 50-265

Enclosures: 1. Amendment No.186to DPR-29
2. Amendment No.183to DPR-30
3. Safety Evaluation

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in black ink, appearing to read "Robert M. Pulsifer", is written over a horizontal line.

Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-254 and 50-265

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3. Safety Evaluation

cc w/encl: See next page

O. Kingsley
Commonwealth Edison Company

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 186
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated November 30, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

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B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 186⁷, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 26, 1999



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 183
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated November 30, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 183 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert M. Pulsifer, Project Manager
Project Directorate III-2
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 26, 1999

ATTACHMENT TO LICENSE AMENDMENT NOS. 186 AND 183

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET NOS. 50-254 AND 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4.8-24
B3/4.8-5

INSERT

3/4.8-24
B3/4.8-5

3.8 LIMITING CONDITIONS FOR OPERATION**J. Safe Shutdown Makeup Pump**

The Safe Shutdown Makeup Pump (SSMP) shall be OPERABLE.

APPLICABILITY:

OPERATIONAL MODE(s) 1, 2 and 3 with reactor steam dome pressure greater than 150 psig.

ACTION:

1. With the SSMP system inoperable, restore the inoperable SSMP system to OPERABLE status within 14 days, or be in at least HOT SHUTDOWN within the next 12 hours and in COLD SHUTDOWN within the following 24 hours.

4.8 - SURVEILLANCE REQUIREMENTS**J. Safe Shutdown Makeup Pump**

The SSMP system shall be demonstrated OPERABLE:

1. At least once per 31 days by:
 - a. Verifying that each valve, manual, power operated or automatic in the flow path that is not locked, sealed or otherwise secured in position, is in its correct position.
 - b. Verifying that the pump flow controller is in the correct position.
2. At least once per 92 days by verifying that the SSMP develops a flow of greater than or equal to 400 gpm against a system head corresponding to reactor vessel pressure of greater than 1120 psig.

BASES

header. The flow rate of the SSMP system is approximately equal to the reactor water boil-off rate 15 minutes after shutdown.

The SSMP system is required to be OPERABLE when either Unit 1 or Unit 2 is in OPERATIONAL MODE(s) 1, 2 or 3 with reactor steam dome pressure greater than 150 psig. With the SSMP system inoperable, a 14-day allowable out-of-service (AOT) is provided to restore the inoperable system to OPERABLE status before the Unit(s) must be shut down.

The surveillance requirements provide adequate assurance that the SSMP system will be OPERABLE when required. A design flow test can be performed during plant operation using a full flow test return line to the CCST.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 186 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 183 TO FACILITY OPERATING LICENSE NO. DPR-30
COMMONWEALTH EDISON COMPANY
AND
MIDAMERICAN ENERGY COMPANY
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
DOCKET NOS. 50-254 AND 50-265

1.0 INTRODUCTION

The Safe Shutdown Makeup Pump (SSMP) is an alternate system to the Reactor Core Isolation Cooling (RCIC) system pursuant to 10 CFR Part 50, Appendix R, paragraph III.G.3. Both the SSMP and the RCIC systems are used to provide reactor makeup water during post-fire safe shutdown activities. However, the Quad Cities Technical Specifications (TSs) presently have a 67 day Allowed Outage Time (AOT) for the SSMP system and 14 day AOT for the RCIC system. This amendment changes TS Section 3/4.8.J and Bases Section 3.8.J for the SSMP AOT from 67 days to 14 days to make it consistent with the RCIC AOT in order to assure equivalent pump availability.

2.0 EVALUATION

The Safe Shutdown Analysis (SSA) for the Quad Cities facility utilizes two methods, either the use of the SSMP system or the RCIC system to provide make up inventory to the reactor depending on the fire area involved. The SSMP system consists of one pump which serves both Units 1 and 2. The Updated Final Safety Analysis Report (UFSAR) Section 5.4.6.5.1 states that the SSMP system is to provide cooling water to Units 1 and 2 reactor core in the event that the reactor becomes isolated from the main condenser simultaneously with a loss of the feedwater system. To achieve this purpose, the SSMP system was designed with the same capacity as the RCIC system. UFSAR Section 5.4.6.5.2 further states that the SSMP system was installed as a common backup to Units 1 and 2 RCIC systems to satisfy the requirements of 10 CFR Part 50, Appendix R, Section III.G, "Fire Protection of Safe Shutdown Capability."

Although the TSs for the RCIC system, 3/4.5.D, "Reactor Core Isolation Cooling," states that the RCIC system when INOPERABLE must be restored to OPERABLE status within 14 days, TS Section 3/4.8.J, "Safe Shutdown Makeup Pump," states that the SSMP system when INOPERABLE must be restored to OPERABLE status within 67 days. On July 2, 1998, the

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NRC staff issued inspection report 50-254/98011; 50-265/98011 which stated that, "Since the SSMP and RCIC had the same post-fire shutdown functionality goals to provide reactor water makeup, it appeared that it was the NRC's expectation that the SSMP would have a 14 day AOT in its TS." Neither the SSMP nor RCIC has an Emergency Core Cooling System (ECCS) function. They are primarily used in the event the core is isolated due to loss of feedwater. The Quad Cities SSA, depending on the location of the fire in the plant, relies on either the RCIC or the SSMP to establish core cooling during the initial phase of maintaining hot shutdown.

On May 13, 1998, the NRC staff met with the licensee where the licensee committed to change the SSMP AOT to 14 days. This change meets that commitment and should ensure that the non-unit specific post-fire safe shutdown equipment functions will be maintained OPERABLE under all unit shutdown and operating conditions. Therefore, to provide consistency regarding the AOT times for these systems, changing the SSMP AOT from 67 days to 14 days is conservative and appropriate. This change is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (64 FR 2246). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Robert M. Pulsifer

Date: March 26, 1999