

December 14, 1982

Docket Nos. 50-254
and 50-265

Mr. L. DelGeorge
Director of Nuclear Licensing
Commonwealth Edison Company
P. O. Box 767
Chicago, Illinois 60690

Dear Mr. DelGeorge:

Re: Quad Cities Nuclear Power Station, Units 1 and 2

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By letter dated November 2, 1982, we transmitted Amendment Nos. 82 and 76 to Facility Operating License Nos. DPR-29 and DPR-30 for Quad Cities Nuclear Power Station Units 1 and 2.

We have determined that page 3.7/4.7-4 of the amendment for each unit contained erroneous information. Please substitute the enclosed corrected page 3.7/4.7-4 to the amendment for each unit.

Sincerely,

ORIGINAL SIGNED BY

Roby Bevan, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosure:
As Stated

cc: w/enclosure
See next page

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PDR ADOCK 05000254
P PDR

[Handwritten signature]

OFFICE	ORB #2	ORB #2 <i>abb</i>	ORB #2				
SURNAME	Norris	Bevan/tcm	Vassallo				
DATE	12/9/82	12/10/82	12/10/82				

Mr. L. DelGeorge
Commonwealth Edison Company

cc:

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Electric Company
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Mr. Marcel DeJaegher, Chairman
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Rock Island County Court House
Rock Island, Illinois 61201

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U.S. Environmental Protection Agency
Region V Office
Regional Radiation Representative
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Chicago, Illinois 60604

Susan N. Sekuler
Assistant Attorney General
Environmental Control Division
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Suite 2315
Chicago, Illinois 60601

The Honorable Tom Corcoran
United States House of Representatives
Washington, D.C. 20515

QUAD CITIES
DPR-29

e. With the measured leakage rate exceeding 11.5 scf per hour for any one main steam isolation valve, restore the leakage rate to \leq 11.5 scf per hour for any one main steam isolation valve prior to increasing the reactor coolant temperature above 212°F.

2) Main steam isolation valves, which shall be leak tested at least once per 18 months at a pressure of 25 psig.

3) Bolted double-gasketed seals which shall be tested at a pressure of 48 psig whenever the seal is closed after being opened and each operating cycle.

e. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurements system.

3. Pressure Suppression Chamber-
Reactor Building Vacuum Breakers

a. The pressure suppression chamber-reactor building vacuum

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Reactor Building Vacuum Breakers

a. Except as specified in Specification 3.7.A.3.b below, two pressure sup-

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PDR ADDCK 05000254
PDR

Amendment No. 68, 82

3.7/4.7-4

QUAD CITIES
DPR-30

e. With the measured leakage rate exceeding 11.5 scf per hour for any one main steam isolation valve, restore the leakage rate to ≤ 11.5 scf per hour for any one main steam isolation valve prior to increasing the reactor coolant temperature above 212°F.

2) Main steam isolation valves, which shall be leak tested at least once per 18 months at a pressure of 25 psig.

3) Bolted double-gasketed seals which shall be tested at a pressure of 48 psig whenever the seal is closed after being opened and each operating cycle.

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