Docket

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 6, 1981

Docket Nos. 50-237
50-249
50-254
and 50-265

Mr. J. S. Abel
Director of Nuclear Licensing
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Abel:

The Commission has issued the enclosed Amendment No. 55 to Provisional Operating License No. DPR-19 for Dresden Station Unit No. 2 and Amendment Nos. 48, 62 and 56 to Facility Operating License Nos. DPR-25, DPR-29 and DPR-30 for Dresden Unit No. 3 and Quad Cities Station Unit Nos. 1 and 2, respectively. These amendments consist of changes to the Technical Specifications in response to your application dated September 18, 1980.

The changes incorporate certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements concern (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Integrity of Systems Outside Containment and (5) Iodine Monitoring. With respect to the Shift Technical Advisor (STA) requirements, the changes proposed in your application regarding the implementation date and title for the STA are under staff review.

Copies of our Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

T. A. Ippolito

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Enclosures:
1. Amendment No. 55 to DPR-19
2. Amendment No. 48 to DPR-25
3. Amendment No. 62 to DPR-29
4. Amendment No. 56 to DPR-30
5. Safety Evaluation
6. Notice

cc w/encl:
See next page

8102280131

Mr. J. S. Abel
Commonwealth Edison Company

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN STATION UNIT NO. 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 55
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated September 18, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changing paragraphs 3.B, 3.J and adding paragraph 3.K of Provisional Operating License No. DPR-19 to read as follows:

   3.B Technical Specifications

   The Technical Specifications contained in Appendix A as revised through Amendment No. 55 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3.J Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and

2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

3.K Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel;

2. Procedures for monitoring, and

3. Provisions for maintenance of sampling and analysis equipment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 55

PROVISIONAL OPERATING LICENSE NO. DPR-19

DOCKET NO. 50-237

Revise the Appendix "A" Technical Specifications as follows:

Remove	Replace
44	44
-	44a
45	45

TABLE 4.2.1 (cont)

Instrument Channel	Instrument Functional Test (2)	Calibration (2)	Instrument Check (2)
ISOLATION CONDENSER ISOLATION			
1. Steam Line High Flow	(1)	Once/3 Months	None
2. Condensate Line High Flow	(1)	Once/3 Months	None
HPCI ISOLATION			
1. Steam Line High Flow	(1)	Once/3 Months	None
2. Steam Line Area High Temperature	Refueling Outage	Refueling Outage	None
3. Low Reactor Pressure	(1)	Once/3 Months	None
REACTOR BUILDING VENTILATION SYSTEM VIOLATION AND STANDBY GAS TREATMENT SYSTEM INITIATION			
1. Ventilation Exhaust Duct Radiation Monitors	(1)	Once/3 Months	Once/Day
2. Refueling Floor Radiation Monitors	(1)	Once/3 Months	Once/Day
STEAM JET-AIR EJECTOR OFF-GAS ISOLATION			
1. Radiation Monitors	(1) (3)	Once/3 Months (4)	Once/Day
CONTAINMENT MONITORING			
1. Pressure			
a. -5 in. Hg to +5 psig Indicator	None	Once/3 Months	Once/Day
b. 0 to 75 psig Indicator	None	Once/3 Months	None
2. Temperature	None	Refueling Outage	Once/Day
3. Drywell-Torus Differential Pressure (5)(6) (0-3 psid)	None	Once/6 Months (two channels operable) Once/Month (one channel operable)	None
4. Torus Water Level (5)(6)	None	Once/6 Months	
a. +25 in. Wide Range Indicator			
b. ±18 in. Sight Glass			
SAFETY/RELIEF VALVE MONITORING			
1. Safety/Relief Valve Position Indicator (Acoustic Monitor) (8)	(7)	None	Once Per 31 Days

TABLE 4.2.1 (cont)

Instrument Channel	Instrument Functional Test (2)	Calibration (2)	Instrument Check (2)
2. Safety/Relief Valve Position Indicator (Temperature Monitor) (8)	None	Once every 18 months	Once Per 31 Days
3. Safety Valve Position Indicator (Acoustic Monitor) (8)	(7)	None	Once Per 31 Days
4. Safety Valve Position Indicator (Temperature Monitor) (8)	None	Once every 18 months	Once Per 31 Days

NOTES:

1. Initially once per month until exposure hours (M as defined on Figure 4.1.1) is 2.0×10^5; thereafter, according to Figure 4.1.1 with an interval not less than one month nor more than three months. The compilation of instrument failure rate data may include data obtained from other Boiling Water Reactors for which the same design instrument operates in an environment similar to that of Dresden Unit 3.

2. Function test calibrations and instrument checks are not required when these instruments are not required to be operable or are tripped. Functional tests shall be performed before each startup with a required frequency not to exceed once per week. Calibrations shall be performed during each startup or during controlled shutdowns with a required frequency not to exceed once per week. Instrument checks shall be performed at least once per week. Instrument checks shall be performed at least once per day during those periods when the instruments are required to be operable.

3. This instrumentation is excepted from the functional test definition. The functional test will consist of injecting a simulated electrical signal into the measurement channel. See Note 4.

4. These instrument channels will be calibrated using simulated electrical signals once every three months. In addition, calibration including the sensors will be performed during each refueling outage.

5. A minimum of two channels is required.

6. From and after the date that one of these parameters (...either drywell-torus differential pressure or torus water level indication) is reduced to one indication, continued operation is not permissible beyond thirty days, unless such instrumentation is sooner made operable. In the event that all indications of these parameters (...either drywell-torus differential pressure or torus water level) is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition in twenty four hours.

NOTES:

7. Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular Safety/Relief Valve.

8. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however, if the reactor is in a shutdown condition, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be returned in thirty days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN STATION UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Commonwealth Edison Company (the licensee) dated September 18, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changing paragraphs 3.B and 3.I and adding paragraph 3.K of Facility Operating License No. DPR-25 to read as follows:

   3.B Technical Specifications

   The Technical Specifications contained in Appendix A as revised through Amendment No. 48 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3.I Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and

2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

3.K Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel;

2. Procedures for monitoring, and

3. Provisions for maintenance of sampling and analysis equipment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 48

FACILITY OPERATING LICENSE NO. DPR-25

DOCKET NO. 50-249

Revise the Appendix "A" Technical Specifications as follows:

Remove	Replace
44	44
-	44a
45	45

TABLE 4.2.1 (cont)

Instrument Channel	Instrument Functional Test (2)	Calibration (2)	Instrument Check (2)
ISOLATION CONDENSER ISOLATION			
1. Steam Line High Flow	(1)	Once/3 Months	None
2. Condensate Line High Flow	(1)	Once/3 Months	None
HPCI ISOLATION			
1. Steam Line High Flow	(1)	Once/3 Months	None
2. Steam Line Area High Temperature	Refueling Outage	Refueling Outage	None
3. Low Reactor Pressure	(1)	Once/3 Months	None
REACTOR BUILDING VENTILATION SYSTEM VIOLATION AND STANDBY GAS TREATMENT SYSTEM INITIATION			
1. Ventilation Exhaust Duct Radiation Monitors	(1)	Once/3 Months	Once/Day
2. Refueling Floor Radiation Monitors	(1)	Once/3 Months	Once/Day
STEAM JET-AIR EJECTOR OFF-GAS ISOLATION			
1. Radiation Monitors	(1) (3)	Once/3 Months (4)	Once/Day
CONTAINMENT MONITORING			
1. Pressure			
a. -5 in. Hg to +5 psig Indicator	None	Once/3 Months	Once/Day
b. 0 to 75 psig Indicator	None	Once/3 Months	None
2. Temperature	None	Refueling Outage	Once/Day
3. Drywell-Torus Differential Pressure (5)(6) (0-3 psid)	None	Once/6 Months (two channels operable) Once/Month (one channel operable)	None
4. Torus Water Level (5)(6)	None	Once/6 Months	
a. ±25 in. Wide Range Indicator			
b. 18 in. Sight Glass			
SAFETY/RELIEF VALVE MONITORING			
1. Safety/Relief Valve Position Indicator (Acoustic Monitor) (8)	(7)	None	Once Per 31 Days

Instrument Channel	Instrument Functional Test (2)	Calibration (2)	Instrument Check (2)
2. Safety/Relief Valve Position Indicator (Temperature Monitor) (8)	None	Once every 18 months	Once Per 31 Days
3. Safety Valve Position Indicator (Acoustic Monitor) (8)	(7)	None	Once Per 31 Days
4. Safety Valve Position Indicator (Temperature Monitor) (8)	None	Once every 18 months	Once Per 31 Days

NOTES:

1. Initially once per month until exposure hours (M as defined on Figure 4.1.1) is 2.0×10^5; thereafter, according to Figure 4.1.1 with an interval not less than one month nor more than three months. The compilation of instrument failure rate data may include data obtained from other Boiling Water Reactors for which the same design instrument operates in an environment similar to that of Dresden Unit 3.

2. Function test calibrations and instrument checks are not required when these instruments are not required to be operable or are tripped. Functional tests shall be performed before each startup with a required frequency not to exceed once per week. Calibrations shall be performed during each startup or during controlled shutdowns with a required frequency not to exceed once per week. Instrument checks shall be performed at least once per week. Instrument checks shall be performed at least once per day during those periods when the instruments are required to be operable.

3. This instrumentation is excepted from the functional test definition. The functional test will consist of injecting a simulated electrical signal into the measurement channel. See Note 4.

4. These instrument channels will be calibrated using simulated electrical signals once every three months. In addition, calibration including the sensors will be performed during each refueling outage.

5. A minimum of two channels is required.

6. From and after the date that one of these parameters (...either drywell-torus differential pressure or torus water level indication) is reduced to one indication, continued operation is not permissible beyond thirty days, unless such instrumentation is sooner made operable. In the event that all indications of these parameters (...either drywell-torus differential pressure or torus water level) is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition in twenty four hours.

NOTES:

7. Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular Safety/Relief Valve.

8. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however, if the reactor is in a shutdown condition, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be returned in thirty days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 62
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated September 18, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changing paragraphs 3.B and 3.G and adding paragraph 3.J of Facility Operating License No. DPR-29 to read as follows:

   3.B Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 62 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3.G Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and

2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

3.J Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel;

2. Procedures for monitoring, and

3. Provisions for maintenance of sampling and anslysis equipment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 62

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix "A" Technical Specifications as follows:

Remove	Replace
3.2/4/2-15	3.2/4.2-15
3.2/4.2-15a	3.2/4.2-15a
3.2/4.2-18	3.2/4.2-18

TABLE 3.2.4

POSTACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS[2]

Minimum Number of Operable Channels[1] [3]	Parameter	Instrument Readout Location Unit 1	Number Provided	Range
1	Reactor pressure	901-5	1 2	0-1500 psig 0-1200 psig
1	Reactor water level	901-3	2	-243 inches-+57 inches
1	Torus water temperature	901-21	2	0-200° F
1	Torus air temperature	901-21	2	0-600° F
2[4]	Torus water level, indicator	901-3	1	-25 inches – + 25 inches
	Torus water level, sight glass		1	18 inch range
1	Torus pressure	901-3	1	-5 inches Hg to 5 psig
1	Drywell pressure	901-3	1	-5 inches Hg to 5 psig 0 to 75 psig
2	Drywell temperature	901-21	6	0-600° F
2	Neutron monitoring	901-5	4	$0.1\text{-}10^5$ CPS
2[4]	Torus to drywell differential pressure		2	0-3 psid
2/valve[5]	Main Steam RV position, acoustic monitor	901-21	1 per valve	NA
	Main Steam RV position, temperature monitor	901-21	1 per valve	0-600°F
2/valve[5]	Main Steam SV position, acoustic monitor	901-21	1 per valve	NA
	Main Steam SV position, temperature monitor	901-21	1 per valve	0-600°F

Notes

1. Instrument channels required during power operation to monitor postaccident conditions.

2. Provisions are made for local sampling and monitoring of drywell atmosphere.

3. In the event any of the instrumentation becomes inoperable for more than 7 days during reactor operation, initiate an orderly shutdown and be in the cold shutdown condition within 24 hours.

4. From and after the date that one of these parameters is reduced to one indication, continued operation is not permissible beyond thirty days, unless such instrumentation is sooner made operable. In the event that all indication of these parameters is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition in twenty four (24) hours.

5. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however, if the reactor is in a shutdown condition, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be restored in 30 days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.

TABLE 4.2-2

POSTACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

Minimum Number of Operable Channels*	Parameter	Instrument Readout Location Unit 1	Calibration	Instrument Check
1	Reactor pressure	901-5	Once every 3 months	Once per day
1	Reactor water level	901-3	Once every 3 months	Once per day
1	Torus water temperature	901-21	Once every 3 months	Once per day
1	Torus air temperature	901-21	Once every 3 months	Once per day
2	Torus water level (indicator)	901-3	Once every 3 months	Once per day
	Torus water level (sight glass)		N/A	None
1	Torus pressure	901-3	Once every 3 months	Once per day
1	Drywell pressure	901-3	Once every 3 months	Once per day
2	Drywell temperature	901-21	Once every 3 months	Once per day
2	Neutron monitoring	901-5	Once every 3 months	Once per day
2	Torus to drywell differential pressure		Once every 6 months	None
2/valve	Main Steam RV Position acoustic monitor	901-21	**	Once per 31 days
	Main Steam RV Position, temperature monitor	901-21	Once every 18 months	Once per 31 days
2/valve	Main Steam SV Position, acoustic monitor	901-21	**	Once per 31 days
	Main Steam SV Position, temperature monitor	901-21	Once every 18 months	Once per 31 days

*Instrument channels required during power operation to monitor postaccident conditions.

**Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular safety or relief valve.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 56
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Commonwealth Edison Company (the licensee) dated September 18, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changing paragraph 3.B and adding paragraphs 3.H and 3.I of Facility Operating License No. DPR-30 to read as follows:

   3.B Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 56 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3.H Systems Integrity

The licensee shall implement a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and

2. Leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

3.I Iodine Monitoring

The licensee shall implement a program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

1. Training of personnel;

2. Procedures for monitoring, and

3. Provisions for maintenance of sampling and analysis equipment.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 6, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 56

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix "A" Technical Specifications as follows:

Remove	Replace
3.2/4.2-15	3.2/4.2-15
3.2/4.2-15a	3.2/4.2-15a
3.2/4.2-18	3.2/4.2-18

TABLE 3.2.4

POSTACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS[2]

Minimum Number of Operable Channels[1] [3]	Parameter	Instrument Readout Location Unit 2	Number Provided	Range
1	Reactor pressure	902-5	1 2	0-1500 psig 0-1200 psig
1	Reactor water level	902-3	2	-243 inches-+57 inches
1	Torus water temperature	902-21	2	0-200° F
1	Torus air temperature	902-21	2	0-600° F
2[4]	Torus water level, indicator	902-3	1	-25 inches – +25 inches
	Torus water level, sight glass		1	18 inch range
1	Torus pressure	902-3	1	-5 inches Hg to 5 psig
1	Drywell pressure	902-3	1	-5 inches Hg to 5 psig 0 to 75 psig
2	Drywell temperature	902-21	6	0-600° F
2	Neutron monitoring	902-5	4	$0.1\text{-}10^6$ CPS
2[4]	Torus to drywell differential pressure		2	0-3 psid
2/valve[5]	Main Steam RV position, acoustic monitor	902-21	1 per valve	NA
	Main Steam RV position, temperature monitor	902-21	1 per valve	0-600°F
2/valve[5]	Main Steam SV position, acoustic monitor	902-21	1 per valve	NA
	Main Steam SV position, temperature monitor	902-21	1 per valve	0-600°F

Notes

1. Instrument channels required during power operation to monitor postaccident conditions.
2. Provisions are made for local sampling and monitoring of drywell atmosphere.
3. In the event any of the instrumentation becomes inoperable for more than 7 days during reactor operation, initiate an orderly shutdown and be in the cold shutdown condition within 24 hours.
4. From and after the date that one of these parameters is reduced to one indication, continued operation is not permissible beyond thirty days, unless such instrumentation is sooner made operable. In the event that all indication of these parameters is disabled and such indication cannot be restored in six (6) hours, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition in twenty four (24) hours.
5. If the number of position indicators is reduced to one indication on one or more valves, continued operation is permissible; however, if the reactor is in a shutdown condition, it may not be started up until all position indication is restored. In the event that all position indication is lost on one or more valves and such indication cannot be restored in 30 days, an orderly shutdown shall be initiated, and the reactor shall be depressurized to less than 90 psig in 24 hours.

TABLE 4.2-2

POSTACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

Minimum Number of Operable Channels*	Parameter	Instrument Readout Location Unit 2	Calibration	Instrument Check
1	Reactor pressure	902-5	Once every 3 months	Once per day
1	Reactor water level	902-3	Once every 3 months	Once per day
1	Torus water temperature	902-21	Once every 3 months	Once per day
1	Torus air temperature	902-21	Once every 3 months	Once per day
2	Torus water level (indicator)	902-3	Once every 3 months	Once per day
	Torus water level (sight glass)		N/A	None
1	Torus pressure	902-3	Once every 3 months	Once per day
1	Drywell pressure	902-3	Once every 3 months	Once per day
2	Drywell temperature	902-21	Once every 3 months	Once per day
2	Neutron monitoring	902-5	Once every 3 months	Once per day
2	Torus to drywell differential pressure		Once every 6 months	None
2/valve	Main Steam RV Position acoustic monitor	902-21	**	Once per 31 days
	Main Steam RV Position, temperature monitor	902-21	Once every 18 months	Once per 31 days
2/valve	Main Steam SV Position, acoustic monitor	902-21	**	Once per 31 days
	Main Steam SV Position, temperature monitor	902-21	Once every 18 months	Once per 31 days

*Instrument channels required during power operation to monitor postaccident conditions.

**Functional tests will be conducted before startup at the end of each refueling outage or after maintenance is performed on a particular safety or relief valve.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 55 TO PROVISIONAL OPERATING LICENSE NO. DPR-19

AMENDMENT NO. 48 TO FACILITY OPERATING LICENSE NO. DPR-25, AND

AMENDMENT NO. 62 TO FACILITY OPERATING LICENSE NO. DPR-29, AND

AMENDMENT NO. 56 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DRESDEN STATION UNIT NO. 2

DRESDEN STATION UNIT NO. 3

QUAD CITIES STATION UNIT NO. 1

QUAD CITIES STATION UNIT NO. 2

DOCKET NOS. 50-237, 50-249, 50-254, 50-265

I. INTRODUCTION

By letter dated September 18, 1980, Commonwealth Edison Company (CECo, the licensee) proposed amendments to Appendix A, Technical Specifications for Operating License Nos. DPR-19, DPR-25, DPR-29 and DPR-30 for Dresden Unit Nos. 2 and 3 and Quad Cities Unit Nos. 1 and 2, respectively. The amendments involve the incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. The licensee's request is in direct response to the NRC staff's letter dated July 2, 1980.

II. BACKGROUND INFORMATION

By our letter dated September 13, 1979, we issued to all operating nuclear power plants requirements established as a result of our review of the TMI-2 accident. Certain of these requirements, designated Lessons Learned Category "A" requirements, were to have been completed by the licensee prior to any operation subsequent to January 1, 1981. Our evaluation of the licensee's compliance with these Cagetory "A" items was attached to our letter to CECo dated March 5, 1980.

In order to provide reasonable assurance that operating reactor facilities are maintained within the limits determined acceptable following the implementation of the TMI-2 Lessons Learned Category "A" items, we requested that licensees amend their Technical Specifications to incorporate additional Limiting Conditions of Operation and Surveillance Requirements, as appropriate. This request was transmitted to all licensees on July 2, 1980. Included therein were model specifications that we had determined to be acceptable. The licensee's application is in direct response to our request. Each of the issues identified by the NRC staff and the licensee's response is discussed in the following Evaluation.

III. EVALUATION

1. Emergency Power Supply/Inadequate Core Cooling

As applicable to Boiling Water Reactors (BWRs), we indicated that instrumentation is important to post-accident monitoring and that surveillance of this instrumentation should be performed. The licensee's response to this request stated that their current surveillance requirements for the reactor water level instrumentation are in adequate agreement with our requirements.

We have reviewed the current specifications (Tables 3.2.1 and 4.2.1 for Dresden 2 and 3 and Tables 3.2-1 and 4.2-1 for Quad Cities 1 and 2) and determined that water level instrumentation is included. The specifications provide ACTION statements for inoperable instrument channels. Surveillance requirements for instrument checks and calibration are also included. The frequency of surveillance meets or exceeds our guidelines. Based on this review, we conclude that no changes are required to satisfy our request.

2. Valve Position Indication

Our requirements for installation of a reliable position indicating system for relief and safety valves was based on the need to provide the operator with a diagnostic aid to reduce the ambiguity between indications that might indicate either an open relief/ safety valve or a small line break. Such a system did not need to be safety grade provided that backup methods of determining valve positions are available.

The licensee's request would add both acoustic monitors and temperature monitors to the specifications. Actions have been specified for the condition of an inoperable channel and for inoperability of both detector channels. Additionally, surveillance requirements have been included. Based on our review, we find that the licensee's recommended changes satisfy our guidelines and are acceptable.

3. Containment Isolation

Our request indicated that the specifications should include a Table of Containment Isolation Valves which reflect the diverse isolation signal requirement of this Lessons Learned issue. The licensee's response to this request stated that their current specifications are in adequate agreement with our requirements.

We have reviewed the current specifications (Table 3.7.1 for Dresden 2 and 3 and Table 3.7-1 for Quad Cities 1 and 2) and conclude that no changes are required to satisfy our request.

4. Integrity of Systems Outside Containment

Our request indicated that licensees should be required to periodically conduct a System Integrity Measurements Program to prevent the release of significant amounts of radioactivity to the environment via leakage from engineered safety systems and auxiliary systems which are located outside reactor containment. The licensee's program includes provisions for a preventive maintenance program and periodic visual inspections. The program also includes system leak test measurements at frequencies not to exceed refueling cycle intervals.

Based on our review we find that inclusion of this requirement as a license condition satisfies our requirement and is acceptable.

5. Iodine Monitoring

We requested that the licensees implement a program which will ensure the capability to determine the airborne iodine concentration in areas requiring personnel access under accident conditions. The licensee's program includes training of personnel, procedures for monitoring and provisions for maintenance of sampling and analysis equipment.

Based on our review we find that inclusion of this requirement as a license condition satisfies our requirement and is acceptable.

IV. ENVIRONMENTAL CONSIDERATIONS

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and eivnronmental impact appraisal need not be prepared in connection with the issuance of these amendments.

V. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: February 6, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-237, 50-249, 50-254 AND 50-265

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 55 to Provisional Operating License No. DPR-19, and Amendment No. 48 to Facility Operating License No. DPR-25, issued to Commonwealth Edison Company, which revised the licenses and their appended Technical Specifications for operation of the Dresden Nuclear Power Station, Unit Nos. 2 and 3, located in Grundy County, Illinois. The Commission has also issued Amendment No. 62 to Facility Operating License No. DPR-29, and Amendment No. 56 to Facility Operating License No. DPR-30, issued to Commonwealth Edison Company and Iowa-Illinois Gas and Electric Company, which revised the licenses and their appended Technical Specifications for operation of the Quad Cities Nuclear Power Station, Unit Nos. 1 and 2, located in Rock Island County, Illinois. The amendments are effective as of the date of issuance.

The revisions to the licenses and the changes to the Technical Specifications involve incorporation of certain of the TMI-2 Lessons Learned Category "A" requirements. These requirements concern (1) Emergency Power Supply/Inadequate Core Cooling, (2) Valve Position Indication, (3) Containment Isolation, (4) Integrity of Systems Outside Containment and (5) Iodine Monitoring.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate

findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated September 18, 1980, (2) Amendment No. 55 to License No. DPR-19, Amendment No. 48 to License No. DPR-25, Amendment No. 62 to License No. DPR-29 and Amendment No. 56 to License No. DPR-30, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D.C., and at the Morris Public Library, 604 Liberty Street, Morris, Illinois, for Dresden 2 and 3 and at the Moline Public Library, 504-17th Street, Moline, Illinois, for Quad Cities 1 and 2. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 6th day of February 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing