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April 24, 1981

Docket Nos. 50-254 and 265

Mr. J. S. Abel  
Director of Nuclear Licensing  
Commonwealth Edison Company  
P. O. Box 767  
Chicago, Illinois 60690

Dear Mr. Abel:

**REGULATORY DOCKET FILE COPY**

The Commission has issued the enclosed Amendment Nos. 69 and 63 to Facility Operating License Nos. DPR-29 and DPR-30 for Quad Cities Station, Units 1 and 2. The amendments are in response to your letter of May 22, 1978.

The amendments remove reactor water cleanup isolation valve MO-1201-80 from Table 3.7-1 of the Technical Specifications and exclude the valve from the surveillance requirement as described in Section 4.7.D.

A copy of the Safety Evaluation and Notice of Issuance are also enclosed.

Sincerely,

Original Signed by  
J. A. Ippolito

Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Enclosures:

1. Amendment No. 69 to DPR-29
2. Amendment No. 63 to DPR-30
3. Safety Evaluation
4. Notice

cc w/enclosures:  
See next page

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F.R. NOTICE  
AMENDMENT  
ONLY

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SURNAME	SNorris	RBevan/cb	TIppolito	TWak	KARMAN		
DATE	4/16/81	4/17/81	4/10/81	4/20/81	4/21/81		



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

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Thomas A. Ippolito, Chief  
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cc w/enclosures:  
See next page

8105140392

Mr. J. S. Abel  
Commonwealth Edison Company

cc:

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Cordova, Illinois 61242

Resident Inspector  
U. S. Nuclear Regulatory Commission  
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Moline, Illinois 61265

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1035 Outer Park Drive  
5th Floor  
Springfield, Illinois 62704

Mr. Marcel DeJaegher, Chairman  
Rock Island County Board  
of Supervisors  
Rock Island County Court House  
Rock Island, Illinois 61201

Director, Criteria and Standards  
Division  
Office of Radiation Programs (ANR-460)  
U. S. Environmental Protection Agency  
Washington, D. C. 20460

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Federal Activities Branch  
Region V Office  
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Susan N. Sekuler  
Assistant Attorney General  
Environmental Control Division  
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Chicago, Illinois 60601



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69  
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated May 22, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-29 is hereby amended to read as follows:


3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 69, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 24, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 69

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix "A" Technical Specifications as follows:

Remove

3.7/4.7-21

Replace

3.7/4.7-21

**QUAD-CITIES  
DPR-29**

**TABLE 3.7-1 (Cont'd)**

Isolation Group	Valve Identification	Valve Number for Units 1 and 2	Number of Power-Operated Valves Inboard Outboard	Maximum Operating Time (sec)	Normal Operating Position	Action on Initiating Signal
<b>Radwaste</b>						
2	Drywell floor drain discharge	AO-2001-3	1	≤20	0	GC
2	Drywell floor drain discharge	AO-2001-4	1	≤20	0	GC
2	Drywell equipment drain discharge	AO-2001-15	1	≤20	0	GC
2	Drywell equipment drain discharge (NOTE: Valve can be reopened after isolation for sampling)	AO-2001-16	1	≤20	0	GC
<b>Oxygen Analyzer</b>						
2	Oxygen analyzer valve	AO-8801-A, B, C, D	4	≤10	0	GC
2	Oxygen analyzer valve	AO-8802-A, B, C, D	4	≤10	0	GC
2	Oxygen analyzer valve	AO-8803A	1	≤10	0	GC
2	Oxygen analyzer valve	AO-8803B	1	≤10	0	GC
<b>Traversing Incore Probe</b>						
2	On isolation signal, the TIP detector is withdrawn if in use; five ball valves and one nitrogen purge are closed.	TIP Ball Valve 700-733				
		TIP Purge Valve Assembly 700-743				
<b>Reactor Water Cleanup</b>						
3	Pump suction isolation valve	MO-1201-2	1	≤30	0	GC
3	Pump suction isolation valve	MO-1201-5	1	≤30	0	GC
<b>HPCI</b>						
4	Steam isolation valve	MO-2301-4	1	≤50	0	GC
4	Steam isolation valve	MO-2301-5	1	≤50	0	GC
<b>RCIC</b>						
5	Turbine steam supply	MO-1301-16	1	≤25	0	GC
5	Turbine steam supply	MO-1301-17	1	≤25	0	GC



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 63  
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment by the Commonwealth Edison Company (the licensee) dated May 22, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility Operating License No. DPR-30 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: April 24, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix "A" Technical Specifications as follows:

Remove

3.7/4.7-21

Replace

3.7/4.7-21

**QUAD-CITIES  
DPR-30**

**TABLE 3.7-1 (Cont'd)**

Isolation Group	Valve Identification	Valve Number for Units 1 and 2	Number of Power-Operated Valves Inboard Outboard	Maximum Operating Time (sec)	Normal Operating Position	Action on Initiating Signal
<b>Radwaste</b>						
2	Drywell floor drain discharge	AO-2001-3	1	≤20	0	GC
2	Drywell floor drain discharge	AO-2001-4		≤20	0	GC
2	Drywell equipment drain discharge	AO-2001-15	1	≤20	0	GC
2	Drywell equipment drain discharge (NOTE: Valve can be reopened after isolation for sampling)	AO-2001-16		≤20	0	GC
<b>Oxygen Analyzer</b>						
2	Oxygen analyzer valve	AO-8801-A, B,C,D	4	≤10	0	GC
2	Oxygen analyzer valve	AO-8802-A, B,C,D		≤10	0	GC
2	Oxygen analyzer valve	AO-8803A	1	≤10	0	GC
2	Oxygen analyzer valve	AO-8803B		≤10	0	GC
<b>Traversing Incore Probe</b>						
2	On isolation signal, the TIP detector is withdrawn if in-use; five ball valves and one nitrogen purge are closed.	TIP Ball Valve 700-733				
		TIP Purge Valve Assembly 700-743				
<b>Reactor Water Cleanup</b>						
3	Pump suction isolation valve	MO-1201-2	1	≤30	0	GC
3	Pump suction isolation valve	MO-1201-5		≤30	0	GC
<b>NPCL</b>						
4	Steam isolation valve	MO-2301-4	1	≤50	0	GC
4	Steam isolation valve	MO-2301-5		≤50	0	GC
<b>RCIC</b>						
5	Turbine steam supply	MO-1301-16	1	≤25	0	GC
5	Turbine steam supply	MO-1301-17		≤25	0	GC



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. DPR-29, AND  
AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY  
AND  
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES NUCLEAR POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-254 AND 265

Introduction

By letter dated May 22, 1978, Commonwealth Edison Company (the licensee) proposed that the Quad Cities Unit 1 and 2 Technical Specifications be amended to remove reactor water cleanup system isolation valve MO-1201-80 from Table 3.7-1, Primary Containment Isolation, and exclude the valve from the surveillance described in Section 4.7.D.

Evaluation

We have reviewed the reactor water cleanup (RWCU) system to determine if the return valve MO-1201-80 should be included in Table 3.7-1 as one of the listed primary containment isolation valves (PCIV's). Table 5.2.4 of the FSAR indicates that the RWCU return line enters the primary containment (PC) thru penetration X-15, provided for that purpose. According to the licensee, the original plant design called for valve 1201-80 (the valve under consideration) to be located in the return line just outboard of the PC, and check valve 1201-81 to be located just inboard of the PC. Valve MO-1201-80 would then be Class A PCIV. It is presently listed as such in Table 5.2.4 of the FSAR and is listed in Table 3.7-1, Primary Containment Isolation, in the Technical Specifications for each unit.

As the plant is built, however, and as shown in drawings M-47 and M-15 of the P and ID's for the Quad Cities units, the RWCU return line does not penetrate containment thru penetration X-15, but instead is routed via valve 1201-80, check valve 1201-81, and manual valve 1201-82 into the "A" feedwater line. (This configuration has been confirmed by discussions with the Resident Inspector). The RWCU return line then enters the PC via check valve 220-62A, penetration X-9, check valve 220-58A, and then into the reactor via feedwater spargers. It is seen that MO-1201-80 is not a PCIV and should not be listed as such in the Technical Specifications.

No modification will be made to the automatic function for valve MO-1201-80. Valve closure will still be initiated by low water level, SBLC initiation, or high temperature in the non-regenerative heat exchanger outlet. The special surveillance requirements to be performed for PCIV's will however not apply to valve MO-1201-80. On the basis of the foregoing evaluation, the licensee's proposal to exclude valve MO-1201-80 from Table 3.7-1 of the Technical Specifications is acceptable.

#### Environmental Considerations

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: April 24, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-254 AND 50-265COMMONWEALTH EDISON COMPANYANDIOWA-ILLINOIS GAS AND ELECTRIC COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 69 to Facility Operating License No. DPR-29, and Amendment No. 63 to Facility Operating License No. DPR-30, issued to Commonwealth Edison Company and Iowa-Illinois Gas and Electric Company, which revised the Technical Specifications for operation of the Quad Cities Nuclear Power Station, Unit Nos. 1 and 2, located in Rock Island County, Illinois. The amendments are effective as of the date of issuance.

The amendments remove reactor water cleanup isolation valve MO-1201-80 from Table 3.7-1 of the Technical Specifications and exclude the valve from the surveillance requirement as described in Section 4.7.D.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

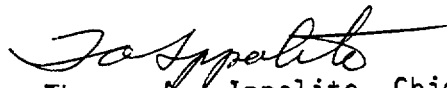
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The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR Section 51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated May 22, 1978, (2) Amendment No. 69 to License No. DPR-29 and Amendment No. 63 to License No. DPR-30, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, NW., Washington, D. C., and at the Moline Public Library, 504-17th Street, Moline, Illinois. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 24th day of April 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief  
Operating Reactors Branch #2  
Division of Licensing