

February 17, 1988

Docket Nos.: 50-254
and 50-265

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Mr. L. D. Butterfield, Jr.
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Butterfield:

SUBJECT: DRYWELL PRESSURE INSTRUMENTATION (TAC NOS. 66410 AND 66411)

Re: Quad Cities Nuclear Power Station, Units 1 and 2

The Commission has issued the enclosed Amendment Nos. 105 and 101 to Facility Operating License Nos. DPR-29 and DPR-30 for Quad Cities Nuclear Power Station (QCNPS), Units 1 and 2. These amendments are in response to your application dated October 16, 1987. Table 3.2-4 of Appendix A, Technical Specifications, was revised to specify an expanded range for drywell pressure recorders.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

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Thierry M. Ross, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 105 to
License No. DPR-29
2. Amendment No 101 to
License No. DPR-30
3. Safety Evaluation

cc w/enclosures:
See next page

PDIII-2:LA
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Mr. L. D. Butterfield, Jr.
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Units 1 and 2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 105
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated October 16, 1987 as supplemented by November 24, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-29 is hereby amended to read as follows:

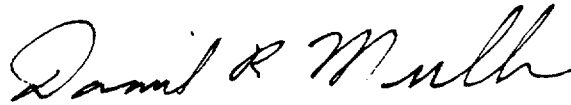
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P PDR

B. Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 105, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 17, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 105

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.2/4.2-15

INSERT

3.2/4.2-15

QUAD-CITIES
DPR-29

TABLE 3.2-4

POSTACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS^[2]

Minimum Number of Operable Chan- nels	Parameter	Instrument Readout Location Unit 1	Number Provided	Range
1	Reactor pressure	901-5	1 2	0-1500 psig 0-1200 psig
1	Reactor water level	901-3	2	-243 inches+57 inches
1	Torus water temperature	901-21	2	0-200°F
1	Torus air temperature	901-21	2	0-600°F
	Torus water level indicator	901-3	1	-5 inches +5 inches (narrow range)
2[6]	Torus water level indicator	901-3	2	0-30 feet (wide range)
	Torus water level sight glass		1	19 inch range (narrow range)
1	Torus pressure	901-3	1	-5 inches Hg to 5 psig
2	Drywell pressure	901-3	1 2	-5 inches Hg to 5 psig -10 inches Hg to 70 psig 0 to 250 psig
2	Drywell temperature	901-21	6	0-600°F
2	Neutron monitoring	901-5	4	0.1-10 ⁸ CPS
2[4]	Torus to drywell differential pressure		2	0-3 psid
1(8)	Drywell Hydrogen concentration	901-55, 56	2	0-4%
2(7)	Drywell radiation monitor	901-55, 56	2	1 to 10 ⁸ R/hr
2/valve[5]	Main Steam RV posi- tion, acoustic monitor	901-21	1 per valve	NA
	Main Steam RV position, temperature monitor	901-21	1 per valve	0-600°F
2/valve[5]	Main Steam SV posi- tion, acoustic monitor	901-21	1 per valve	NA
	Main Steam SV position, temperature monitor	901-21	1 per valve	0-600°F



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 101
License No. DPR-30

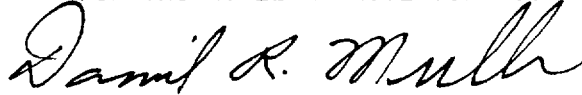
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated October 16, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 101, are hereby incorporated in this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 17, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 101

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.2/4.2-15

INSERT

3.4/4.2-15

QUAD-CITIES
DPR-30

TABLE 3.2-4

POSTACCIDENT MONITORING INSTRUMENTATION REQUIREMENTS^[2]

Minimum Number of Operable Chan- nels	Parameter	Instrument Readout Location Unit 2	Number Provided	Range
1	Reactor pressure	902-5	1 2	0-1500 psig 0-1200 psig
1	Reactor water level	902-3	2	-243 inches+57 inches
1	Torus water temperature	902-21	2	0-200°F
1	Torus air temperature	902-21	2	0-600°F
	Torus water level indicator	902-3	1	-5 inches +5 inches (narrow range)
2[6]	Torus water level indicator	902-3	2	0-30 feet (wide range)
	Torus water level sight glass		1	18 inch range (narrow range)
1	Torus pressure	902-3	1	-5 inches Hg to 5 psig
2	Drywell pressure	902-3	1	-5 inches Hg to 5 psig -10 inches Hg to 70 psig
			2	0 to 250 psig
2	Drywell temperature	902-21	6	0-600°F
2	Neutron monitoring	902-5	4	0.1-10 ⁸ CPS
2[4]	Torus to drywell differential pressure		2	0-3 psid
1(8)	Drywell Hydrogen concentration	902-55, 56	2	0-4%
2(7)	Drywell radiation monitor	902-55, 56	2	1 to 10 ⁸ R/hr
2/valve[5]	Main Steam RV posi- tion, acoustic monitor	902-21	1 per valve	NA
	Main Steam RV position, temperature monitor	902-21	1 per valve	0-600°F
2/valve[5]	Main Steam SV posi- tion, acoustic monitor	902-21	1 per valve	NA
	Main Steam SV position, temperature monitor	902-21	1 per valve	0-600°F



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 105 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 101 TO FACILITY OPERATING LICENSE NO. DPR-30
COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
DOCKET NOS. 50-254 AND 50-265

1.0 INTRODUCTION

In order to comply with an NRC Regulatory Guide (RG) 1.97 commitment to expand the indicating range of drywell pressure recorders, Commonwealth Edison Company (CECo, the licensee) replaced affected scales and recalibrated corresponding instruments. However, these scale modifications were inconsistent with Quad Cities Nuclear Power Station (QCNPS), Units 1 and 2, Technical Specifications (TS). Consequently, on October 16, 1987 CECo submitted an amendment request to revise TS appropriately.

2.0 EVALUATION

TS Table 3.2-4 specifies three ranges, and four channels, for drywell pressure indication: "-5 inches Hg to 5 psig", "0 to 75 psig", and "0 to 250 psig". One of these ranges, "0 to 75 psig", was expanded to "-10 inches Hg to 70 psig" by rescaling the recorder and respanning applicable instrumentation. This modification was accomplished, without hardware instrument changes, to provide for negative drywell pressure indication on a wide range scale. Those indicators which display up to 250 psig are also present on front panels in the control room, and will continue to provide for drywell pressures in excess of 70 psig. Furthermore, the drywell pressure boundary at QCNPS was designed for a maximum internal pressure of 62 psig, which is still within the "-10 inches Hg to 70 psig" pressure range. Thus, no meaningful information capability was lost by this modification. To the contrary, a broader range of negative drywell pressure indication is now available to the operators without altering any modes of equipment or instrument operations. The only physical equipment change actually made was to replace the control room scale display of recorder 8740-12 to allow for visual quantification of the expanded instrument range. This modification made was in accordance with CECo commitments to meet the post-accident monitoring requirements of RG 1.97.

Consequently, the NRC staff has concluded that CEC's amendment request of October 16, 1987 does accurately correct the applicable TS to incorporate the revised drywell pressure indication range and is therefore, acceptable.

An additional TS change was proposed for Quad Cities Unit 2, DPR-30, which corrects a typographical error on Table 3.2-4 where reference is made to Unit 1 instrumentation rather than Unit 2. This is simply an administrative clarification, and is acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to the use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types of any effluents that may be released offsite and there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: T. Ross

Dated: February 17, 1988