

June 30, 1988

Docket Nos.: 50-254
and 50-265

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

DISTRIBUTION

Docket	JPartlow GRIMES
NRC & Local PDRs	TBarnhart (8)
PDI III-2 Rdg. File	WJones
GHolahan	EButcher
LLuther	ACRS (10)
TRoss	GPA/PA
OGC-Rockville	ARM/LFMB
DHagan	PDI III-2 Plant File
EJordan	

Dear Mr. Bliss:

SUBJECT: AMENDMENT OF TECHNICAL SPECIFICATIONS FOR RF RAD MONITORS AND ADMINISTRATIVE REQUIREMENT (TAC NOS. 66856 AND 66857)

The Commission has issued the enclosed Amendment Nos. 110 and 106 to Facility Operating License Nos. DPR-29 and DPR-30 for the Quad Cities Nuclear Power Station (QCNSP), Units 1 and 2. These Technical Specifications amendments were in response to your application dated December 22, 1987. Accordingly, we have revised the trip setpoint for refueling floor radiation monitors used at QCNSP, and elevated the level of management authorization required for approving changes to plant operating procedures.

A copy of our Safety Evaluation is also enclosed. Furthermore, a Notice of Issuance will be included in the Commission's biweekly Federal Register.

Sincerely,

Thierry Ross, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 110 to License No. DPR-29
2. Amendment No. 106 to License No. DPR-30
3. Safety Evaluation

cc: w/enclosures:
See next page

PDI III-2:PM
TRoss:cdd
6/13/88

PDI III-2:LA
LLuther
6/14/88

PDI III-2
LNordim
6/13/88

*Concur with
changes to SER as
indicated.
APH*

OGC APH
6/20/88

*Ated, appropriate
review done
to be
w/Concur APH*

*DF01
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TRoss:cdd/R
6/13/88

PDIII-2:LA
LLuther/L
6/14/88

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OGC APH
6/20/88

*Concur with
~~See~~ changes to SER as
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APH*

*Also appropriate
review board
needs to be
in concur with
APH*



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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and 50-265

Mr. Henry E. Bliss
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
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SUBJECT: AMENDMENT OF TECHNICAL SPECIFICATIONS FOR RF RAD MONITORS AND
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1. Amendment No. 110 to
License No. DPR-29
2. Amendment No 106 to
License No. DPR-30
3. Safety Evaluation

cc: w/enclosures:
See next page

Mr. Henry Bliss
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Units 1 and 2

cc:

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Vice President
Iowa-Illinois Gas and
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U. S. Nuclear Regulatory Commission
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Cordova, Illinois 61242

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Rock Island County Board
of Supervisors
1504 3rd Avenue
Rock Island County Office Bldg.
Rock Island, Illinois 61201

Mr. Michael E. Parker, Chief
Division of Engineering
Illinois Department of Nuclear Safety
1035 Outer Park Drive,
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 110
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated December 22, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-29 is hereby amended to read as follows:

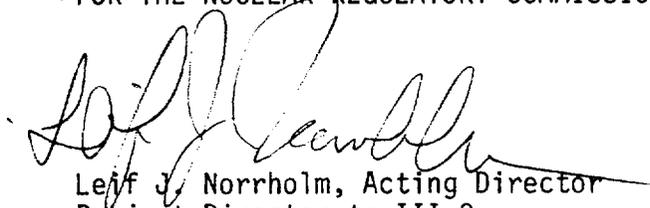
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B. Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 110, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Leif J. Norrholm, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 30, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 110

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.2/4.2-2

3.2/4.2-7

6.2.1

INSERT

3.2/4.2-2

3.2/4.2-7

6.2.1

QUAD-CITIES
DPR-29

D. Refueling Floor Radiation Monitors

1. Except as specified in Specification 3.2.D.2, the two refueling floor radiation monitors shall be operable whenever irradiated fuel or components are present in the fuel storage pool and during refueling or fuel movement operations.
2. One of the two refueling floor radiation monitors may be inoperable for 24 hours. If the inoperable monitor is not restored to service in this time, the reactor building ventilation system shall be isolated and the standby gas treatment operated until repairs are complete.
3. The trip setting for the refueling floor radiation monitors shall be set at a value of ≤ 100 mR/hr.
4. Upon loss of both refueling floor radiation monitors while in use, the reactor building ventilation system shall be isolated and the standby gas treatment operated.

E. Postaccident Instrumentation

The limiting conditions for operation for the instrumentation which is read out in the control room, required for postaccident monitoring are given in Table 3.2-4.

D. Refueling Floor Radiation Monitors

The two refueling floor radiation monitors shall be functionally tested and calibrated as indicated in Table 4.2-1. Reactor building ventilation isolation and standby gas treatment system initiation shall be performed at least each operating cycle.

E. Postaccident Instrumentation

Postaccident instrumentation shall be functionally tested and calibrated as indicated in Table 4.2-2.

QUAD-CITIES
DPR-29

The APRM rod block function is flow biased and prevents a significant reduction in MCPR, especially during operation at reduced flow. The APRM provides gross core protection, i.e., limits the gross withdrawal of control rods in the normal withdrawal sequence.

In the refuel and startup/hot standby modes, the APRM rod block function is set at 12% of rated power. This control rod block provides the same type of protection in the Refuel and Startup/Hot Standby modes as the APRM flow-biased rod block does in the Run mode, i.e., prevents control rod withdrawal before a scram is reached.

The RBM rod block function provides local protection of the core, i.e., the prevention of transition boiling in a local region of the core for a single rod withdrawal error from a limiting control rod pattern. The trip point is flow biased. The worst-case single control rod withdrawal error is analyzed for each reload to assure that, with the specific trip settings, rod withdrawal is blocked before the MCPR reaches the fuel cladding integrity safety limit.

Below 30% power, the worst-case withdrawal of a single control rod without rod block action will not violate the fuel cladding integrity safety limit. Thus the RBM rod block function is not required below this power level.

The IRM block function provides local as well as gross core protection. The scaling arrangement is such that the trip setting is less than a factor of 10 above the indicated level. Analysis of the worst-case accident results in rod block action before MCPR approaches the MCPR fuel cladding integrity safety limit.

A downscale indication on an APRM is an indication the instrument has failed or is not sensitive enough. In either case the instrument will not respond to changes in control rod motion, and the control rod motion is thus prevented. The downscale trips are set at 3/125 of full scale.

The SRM rod block with ≤ 100 CPS and the detector not full inserted assures that the SRM's are not withdrawn from the core prior to commencing rod withdrawal for startup. The scram discharge volume high water level block provide annunciation for operator action. The alarm setpoint has been selected to provide adequate time to allow determination of the cause of level increase and corrective action prior to automatic scram initiation.

For effective emergency core cooling for small pipe breaks the HPCI system must function since reactor pressure does not decrease rapidly enough to allow either core spray or LPCI to operate in time. The automatic pressure relief function is provided as a backup to the HPCI in the event the HPCI does not operate. The arrangement of the tripping contacts is such as to provide this function when necessary and minimize spurious operation. The trip settings given in the specification are adequate to assure the above criteria are met (reference SAR Section 6.2.6.3). The specification preserves the effectiveness of the system during periods of maintenance, testing or calibration and also minimizes the risk of inadvertent operation, i.e., only one instrument channel out of service.

Two radiation monitors are provided on the refueling floor which initiate isolation of the reactor building and operation of the standby gas treatment systems. The trip logic is one out of two. Trip settings of ≤ 100 mR/hr for the monitors on the refueling floor are based upon initiating normal ventilation isolation and standby gas treatment system operation

6.2 PLANT OPERATING PROCEDURES

- A. Detailed written procedures, including applicable checkoff lists covering items listed below shall be prepared, approved, and adhered to:
1. Normal startup, operation, and shutdown of the reactor, and other systems and components involving nuclear safety of the facility.
 2. Refueling operations.
 3. Actions to be taken to correct specific and foreseen potential malfunctions of systems or components, including responses to alarms, suspected primary system leaks, and abnormal radioactivity changes.
 4. Emergency conditions involving potential or actual release of radioactivity - "Generating Station Emergency Plan" and station emergency and abnormal procedures.
 5. Instrumentation operation which could have an effect on the safety of the facility.
 6. Preventive and corrective maintenance operations which could have an affect on the safety of the facility.
 7. Surveillance and testing requirements.
 8. Tests and experiments.
 9. Procedure to ensure safe shutdown of the plant
 10. Station Security Plan and implementation procedures.
 11. Fire Protection Program implementation.
 12. ODCM implementation.
 13. PCP implementation.
 14. Working hours of the Shift Engineer, Station Control Room Engineer, Shift Foreman and the Nuclear Station Operator job classifications such that the heavy use of overtime is not routinely required.
- B. Radiation control procedures shall be maintained, made available to all station personnel, and adhered to. The procedures shall show permissible radiation exposure and shall be consistent with the requirements of 10 CFR 20. This radiation protection program shall be organized to meet the requirements of 10 CFR 20.
- C.
1. Procedures for items identified in Specification 6.2.A and 6.2.B and any changes to such procedures shall be reviewed and approved by the Technical Staff Supervisor, the Assistant Superintendent, and department head responsible for the activity performed. At least one person approving each of the above procedures shall hold a valid senior operator's license. In addition, these procedures and changes thereto must have authorization by a Station Superintendent before being implemented.
 2. Work and instruction type procedures which implement approved maintenance or modification procedures shall be approved and authorized by the Asst. Supt. Maintenance where the written authority has been provided by a Station Superintendent. The "Maintenance Modification Procedure" utilized for safety related work shall be so approved only if procedures referenced in the "Maintenance Modification Procedure" have been approved as required by 6.2.A. Procedures which do not fall within the requirement of 6.2.A or 6.2.B may be approved by the Department Heads.
- D. Temporary changes to procedures 6.2.A and 6.2.B above may be made provided:
1. The intent of the original procedure is not altered.
 2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 106
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated December 22, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 106, are hereby incorporated in this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Leif J. Norrholm, Acting Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: June 30, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.106

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.2/4.2-2

3.2/4.2-7

6.2.1

INSERT

3.2/4.2-2

3.2/4.2-7

6.2.1

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3. The trip setting for the refueling floor radiation monitors shall be set at a value of ≤ 100 mR/hr.
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QUAD-CITIES
DPR-30

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Below 30% power, the worst-case withdrawal of a single control rod without rod block action will not violate the fuel cladding integrity safety limit. Thus the RBM rod block function is not required below this power level.

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QUAD-CITIES
DPR-30

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- B. Radiation control procedures shall be maintained, made available to all station personnel, and adhered to. The procedures shall show permissible radiation exposure and shall be consistent with the requirements of 10 CFR 20. This radiation protection program shall be organized to meet the requirements of 10 CFR 20.
- C.
1. Procedures for items identified in Specification 6.2.A and 6.2.B and any changes to such procedures shall be reviewed and approved by the Technical Staff Supervisor, the Assistant Superintendent, and department head responsible for the activity performed. At least one person approving each of the above procedures shall hold a valid senior operator's license. In addition, these procedures and changes thereto must have authorization by a Station Superintendent before being implemented.
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- D. Temporary changes to procedures 6.2.A and 6.2.B above may be made provided:
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 2. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Reactor Operator's license on the unit affected.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 110 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 106 TO FACILITY OPERATING LICENSE NO. DPR-30

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2

DOCKET NOS. 50-254 AND 50-265

1.0 INTRODUCTION

In a letter dated December 22, 1987, Commonwealth Edison Company (the licensee), submitted an application for license amendments to change the Technical Specifications (TS) for the Quad-Cities Nuclear Power Station (QCNPS). These amendment requests proposed revising TS 3.2.D.3 to change the refueling floor radiation monitors trip set point from 100 mR/hr to < 100 mR/hr, and TS 6.2.C.1 to upgrade the highest level of authorization (to the Assistant Superintendent level) for approving all procedures identified in TS 6.2.A and 6.2.B.

During the course of our review, the staff referred to Title 10 of the Code of Federal Regulations - Chapter I, and the Technical Specifications for QCNPS, Units 1 and 2.

2.0 EVALUATION

Proposed amendments to TS 3.2.D.3 would change the trip setpoint for the refueling floor radiation monitors from 100 mR/hr to ≤ 100 mR/hr. This change is consistent with the general philosophy of prescribing TS setpoints in terms of a limiting value rather than an absolute value. The use of a limiting value permits a setpoint margin to be established for accommodating instrument drift and calibration uncertainties. This provides assurance that the limiting value will not be exceeded. The staff has reviewed this change and concurs there is no adverse impact on safety. Therefore, this proposed change is acceptable. Proposed amendments to TS 6.2.C.1 raise the highest level of authorization for procedures, and changes to all procedures, identified in TS 6.2.A. and 6.2.B to the Assistant Superintendent level. Current TS requirements for review and approval varied the level of authorization dependent upon procedure type. This proposed change would provide a consistent level of authorization for approving procedures, and their changes, for all procedures identified by TS 6.2.A and 6.2.B. The staff determined this change is administrative in nature, has no adverse impact on safety, and, therefore, is acceptable.

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3.0 ENVIRONMENTAL CONSIDERATION

The amendment to TS 3.2.D.3 involves a change to the requirements for use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined this amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). The amendment to TS 6.2.C.1 concerns administrative procedures and, therefore, is eligible for categorical exclusion under 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The staff concluded, based upon considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: R.A. Hasse, Region III

Dated: June 30, 1988