

Docket Nos. 50-254/265

August 6, 1987

Mr. L. D. Butterfield, Jr.
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Butterfield:

SUBJECT: 4160 VAC BUS TIE TECHNICAL SPECIFICATIONS (TAC 61589 AND 61590)

Re: Quad Cities Nuclear Power Station, Units 1 and 2

The Commission has issued the enclosed Amendment Nos. 102 and 99 to Facility Operating License Nos. DPR-29 and DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2. The amendments are in response to your application dated May 14, 1986.

Operability and surveillance requirements for the 4160 VAC bus tie have been incorporated into Technical Specification 3.9/4.9, "Auxiliary Electrical Systems".

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

Original Signed by/

Thierry M. Ross, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

1. Amendment No. 102 to License No. DPR-29
2. Amendment No. 99 to License No. DPR-30
3. Safety Evaluation

cc w/enclosures:
See next page

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| Docket File | SNorris | JPartlow | NTrehan |
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| PDIII-2 Reading | DHagan | EButcher | PDIII-2 Plant File |
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SNorris
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OGC
OGC-Bethesda
MKarman
7/15/87
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Mr. L. D. Butterfield, Jr.
Commonwealth Edison Company

Quad Cities Nuclear Power Station
Units 1 and 2

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 102
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated May 14, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

8708190027 870806
PDR ADDCK 05000254
P PDR

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 102, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 6, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 102

FACILITY OPERATING LICENSE NO. DPR-29

DOCKET NO. 50-254

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.9/4.9 - 1*
3.9/4.9 - 2
3.9/4.9 - 3
3.9/4.9 - 4

INSERT

3.9/4.9 - 1*
3.9/4.9 - 2
3.9/4.9 - 3
3.9/4.9 - 4

* Page re-typed for editorial clarity only. No changes made.

QUAD-CITIES
DPR-29

3.9/4.9 AUXILIARY ELECTRICAL SYSTEMS

LIMITING CONDITIONS FOR OPERATION

Applicability:

Applies to the auxiliary electrical power system.

Objective:

To assure an adequate supply of electrical power during plant operation.

SURVEILLANCE REQUIREMENTS

Applicability:

Applies to the periodic testing requirement of the auxiliary electrical system.

Objective:

To verify the operability of the auxiliary electrical system

SPECIFICATIONS

A. Normal and Emergency A-C Auxiliary Power

The reactor shall not be made critical unless all the following requirements are satisfied.

1. The Unit diesel generator and the Unit 1/2 diesel generator shall be operable.

2. One 345-kV line, associated switchgear, and the reserve auxiliary power transformer capable of carrying power

A. Normal and emergency A-C Auxiliary Power

1. a. Each diesel generator shall be manually started and loaded once each month to demonstrate operational readiness. The test shall continue until both the diesel engine and the generator are at equilibrium conditions of temperature while full load output is maintained.
b. During the monthly generator test, the diesel-starting air compressor shall be checked for operation and its ability to recharge air receivers.
c. During the monthly generator test, the diesel fuel oil transfer pumps shall be operated.

2. The status of the 345-kV lines, associated switchgear, and the reserve auxiliary power transformer shall be

to the unit shall be available.

3. One other 345-kV line and unit reserve aux transformer capable of carrying auxiliary power to an essential electrical bus of the unit through the 4160-volt bus tie shall be available.
4. a. The Unit engineered safety features 4160-volt buses (13-1 and 14-1, Unit 1; 23-1 and 24-1, Unit 2) are energized.
b. The Unit engineered safety features 480-volt buses (18 and 19, Unit 1; 28 and 29, Unit 2) are energized.

B. Station Batteries

The unit 24/48-volt batteries, two station 125-volt batteries, the two station 250-volt batteries, and a battery charger for each required battery shall be operable before the reactor can be made critical.

C. Electric Power Availability

Whenever the reactor is in the Run mode or for startup from a hot shut-down condition, the availability of electric power shall be as specified in Specifications 3.9.A and 3.9.B except as stated in Specifications 3.9.C.1, 3.9.C.2, 3.9.C.3, and 3.9.E.

1. From and after the date that incoming power is available from only one of the lines specified in 3.9.A, continued reactor operation is permissible only during the succeeding 7 days,

checked daily.

3. The status of the additional source of power via the 4160-volt bus tie shall be checked daily.
4. The Unit engineered safety features 4160-volt and 480-volt buses shall be checked daily.

B. Station Batteries

1. Every week the specific gravity and voltage of the pilot cell, the temperature of adjacent cell, and overall battery voltage shall be measured.
2. Every 3 months the measurement shall be made of the voltage of each cells to the nearest 0.01 volt, the specific gravity of each cells, and the temperature of every fifth cell.
3. Every refueling outage, the station batteries shall be subjected to a rated load discharge test. Specific gravity and voltage of each cell shall be determined after the discharge.

C. Electric Power Availability

The availability status of electric power shall be checked daily.

QUAD-CITIES
DPR-29

unless the second line is sooner made available, providing both the Unit and Unit 1/2 emergency diesel generators are operable.

2. From and after the date the incoming power is not available from any line, continued reactor operation is permissible providing both the Unit and Unit 1/2 emergency diesel generators are operating, all core and containment cooling systems are operable, reactor power level is reduced to 40% of rated, and the NRC is notified within 24 hours of the situation, the precautions to be taken during this period, and the plans for prompt restoration of incoming power.
3. From and after the date that one of the two 125/250-volt battery systems is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding 3 days unless such battery system is sooner made operable.

D. Diesel Fuel

There shall be a minimum of 10,000 gallons of diesel fuel supply on site for each diesel generator.

E. Diesel-Generator Operability

1. Whenever the reactor is in the Startup/Hot Standby or Run mode and the unit or shared diesel generators and/or their respective associated buses are made or found to be inoperable for any reason, except as specified in Specification 3.9.E.2 below, continued reactor operation is permissible only during the succeeding 7 days provided that all of the low-pressure core cooling and all loops of the containment cooling mode of the RHR system associated with the operable diesel generator shall be operable, and two offsite lines as specified in 3.9.A are available. If this requirement cannot be met, an orderly shutdown shall be initiated and the

D. Diesel Fuel

Once a month the quantity of diesel fuel available shall be logged.

Once a month a sample of diesel fuel shall be checked for quality.

E. Diesel-Generator Operability

1. When it is determined that either the unit or shared diesel generator is inoperable, all low-pressure core cooling systems and all loops of the containment cooling modes of the RHR system associated with the operable diesel generator shall be demonstrated to be operable immediately and daily thereafter. The operable diesel generator shall be demonstrated to be operable immediately and daily thereafter.
2. During each refueling outage, a simulated loss of off-site power in conjunction with an ECCS initiation signal test shall be performed on the 4160 volt emergency bus by:

reactor shall be in the cold shutdown condition within 24 hours.

2. Specification 3.9.E.1 shall not apply when a diesel generator has been made inoperable for a period not to exceed 1-1/2 hours for the purpose of conducting preventative maintenance. Additionally, preventative maintenance shall not be undertaken unless two off-site lines as specified in 3.9.A are available and the alternate diesel generator has been demonstrated to be operable.
3. When the reactor is in the Cold Shutdown or Refueling mode, a minimum of one diesel generator (either the Unit diesel generator or the Unit 1/2 diesel generator) shall be operable whenever any work is being done which has the potential for draining the vessel, secondary containment is required, or a core or containment cooling system is required.

F. REACTOR PROTECTION BUS POWER MONITORING SYSTEM

1. Two RPS electric power monitoring channels for each inservice RPS MG set or inservice alternate power source shall be OPERABLE except when the reactor is in the SHUTDOWN mode.
2.
 - a. With one RPS electric power monitoring channel for an inservice RPS MG set or inservice alternate power source inoperable, restore the inoperable channel to OPERABLE status within 72 hours or remove the associated RPS MG set or alternate power source from service.
 - b. With both RPS electric power monitoring channels for an inservice RPS MG set or inservice alternate power source inoperable, restore at least one channel to OPERABLE status within 30 minutes, or remove the associated RPS MG set or alternate power source from service.

- a. Verifying de-energization of the emergency buses, and load shedding from the emergency buses.
- b. Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency buses with permanently connected loads, energizes the auto-connected emergency loads through the load sequencer, and operates for greater than 5 minutes while its generator is loaded with the emergency loads.

F. REACTOR PROTECTION BUS POWER MONITORING SYSTEM

1. The RPS Bus power monitoring system instrumentation shall be determined OPERABLE:
 - a. At least once per 6 months by performing a channel functional test, and
 - b. At least once per operating cycle by demonstrating the operability of overvoltage, undervoltage, and underfrequency protective instrumentation by performance of a channel calibration including simulated automatic activation of the protective relays, tripping logic, and output circuit breakers, and verifying the following setpoints:
 - (1) overvoltage $126.5 \text{ V} = 2.5\%$
Min. 123.3 V
Max. 129.6 V
 - (2) undervoltage $108 \text{ V} = 2.5\%$
Min. 105.3 V
Max. 110.7 V
 - (3) underfrequency $56.0 \text{ Hz} \pm 1\%$
of 60 Hz
Min. 55.4 Hz
Max. 56.6 Hz



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 99
License No. DPR-30

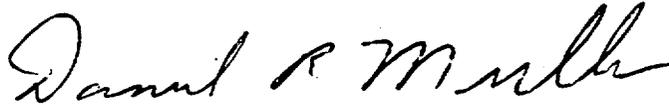
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated May 14, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 99, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 6, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 99

FACILITY OPERATING LICENSE NO. DPR-30

DOCKET NO. 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.9/4.9 - 1*
3.9/4.9 - 2
3.9/4.9 - 3
3.9/4.9 - 4

INSERT

3.9/4.9 - 1*
3.9/4.9 - 2
3.9/4.9 - 3
3.9/4.9 - 4

* Page re-typed for editorial clarity only. No changes made.

3.9/4.9 AUXILIARY ELECTRICAL SYSTEMS

LIMITING CONDITIONS FOR OPERATION

Applicability:

Applies to the auxiliary electrical power system.

Objective:

To assure an adequate supply of electrical power during plant operation.

SURVEILLANCE REQUIREMENTS

Applicability:

Applies to the periodic testing requirement of the auxiliary electrical system.

Objective:

To verify the operability of the auxiliary electrical system

SPECIFICATIONS

A. Normal and Emergency A-C Auxiliary Power

The reactor shall not be made critical unless all the following requirements are satisfied.

1. The Unit diesel generator and the Unit 1/2 diesel generator shall be operable.

2. One 345-kV line, associated switchgear, and the reserve auxiliary power transformer capable of carrying power

A. Normal and emergency A-C Auxiliary Power

1. a. Each diesel generator shall be manually started and loaded once each month to demonstrate operational readiness. The test shall continue until both the diesel engine and the generator are at equilibrium conditions of temperature while full load output is maintained.

- b. During the monthly generator test, the diesel-starting air compressor shall be checked for operation and its ability to recharge air receivers.

- c. During the monthly generator test, the diesel fuel oil transfer pumps shall be operated.

2. The status of the 345-kV lines, associated switchgear, and the reserve auxiliary power transformer shall be

to the unit shall be available.

3. One other 345-kV line and unit reserve aux transformer capable of carrying auxiliary power to an essential electrical bus of the unit through the 4160-volt bus tie shall be available.
4. a. The Unit engineered safety features 4160- volt buses (13-1 and 14-1, Unit 1; 23-1 and 24-1, Unit 2) are energized.
b. The Unit engineered safety features 480- volt buses (18 and 19, Unit 1; 28 and 29, Unit 2) are energized.

B. Station Batteries

The unit 24/48-volt batteries, two station 125-volt batteries, the two station 250-volt batteries, and a battery charger for each required battery shall be operable before the reactor can be made critical.

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Whenever the reactor is in the Run mode or for startup from a hot shut-down condition, the availability of electric power shall be as specified in Specifications 3.9.A and 3.9.B except as stated in Specifications 3.9.C.1, 3.9.C.2, 3.9.C.3, and 3.9.E.

1. From and after the date that incoming power is available from only one of the lines specified in 3.9.A, continued reactor operation is permissible only during the succeeding 7 days,

checked daily.

3. The status of the additional source of power via the 4160-volt bus tie shall be checked daily.
4. The Unit engineered safety features 4160-volt and 480-volt buses shall be checked daily.

B. Station Batteries

1. Every week the specific gravity and voltage of the pilot cell, the temperature of adjacent cell, and overall battery voltage shall be measured.
2. Every 3 months the measurement shall be made of the voltage of each cells to the nearest 0.01 volt, the specific gravity of each cells, and the temperature of every fifth cell.
3. Every refueling outage, the station batteries shall be subjected to a rated load discharge test. Specific gravity and voltage of each cell shall be determined after the discharge.

C. Electric Power Availability

The availability status of electric power shall be checked daily.

unless the second line is sooner made available, providing both the Unit and Unit 1/2 emergency diesel generators are operable.

2. From and after the date the incoming power is not available from any line, continued reactor operation is permissible providing both the Unit and Unit 1/2 emergency diesel generators are operating, all core and containment cooling systems are operable, reactor power level is reduced to 40% of rated, and the NRC is notified within 24 hours of the situation, the precautions to be taken during this period, and the plans for prompt restoration of incoming power.
3. From and after the date that one of the two 125/250-volt battery systems is made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding 3 days unless such battery system is sooner made operable.

D. Diesel Fuel

There shall be a minimum of 10,000 gallons of diesel fuel supply on site for each diesel generator.

E. Diesel-Generator Operability

1. Whenever the reactor is in the Startup/Hot Standby or Run mode and the unit or shared diesel generators and/or their respective associated buses are made or found to be inoperable for any reason, except as specified in Specification 3.9.E.2 below, continued reactor operation is permissible only during the succeeding 7 days provided that all of the low-pressure core cooling and all loops of the containment cooling mode of the RHR system associated with the operable diesel generator shall be operable, and two offsite lines as specified in 3.9.A are available. If this requirement cannot be met, an orderly shutdown shall be initiated and the

D. Diesel Fuel

Once a month the quantity of diesel fuel available shall be logged.

Once a month a sample of diesel fuel shall be checked for quality.

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1. When it is determined that either the unit or shared diesel generator is inoperable, all low-pressure core cooling systems and all loops of the containment cooling modes of the RHR system associated with the operable diesel generator shall be demonstrated to be operable immediately and daily thereafter. The operable diesel generator shall be demonstrated to be operable immediately and daily thereafter.
2. During each refueling outage, a simulated loss of off-site power in conjunction with an ECCS initiation signal test shall be performed on the 4160 volt emergency bus by:

QUAD-CITIES
DPR-30

- reactor shall be in the cold shutdown condition within 24 hours.
2. Specification 3.9.E.1 shall not apply when a diesel generator has been made inoperable for a period not to exceed 1-1/2 hours for the purpose of conducting preventative maintenance. Additionally, preventative maintenance shall not be undertaken unless two off-site lines as specified in 3.9.A are available and the alternate diesel generator has been demonstrated to be operable.
 3. When the reactor is in the Cold Shutdown or Refueling mode, a minimum of one diesel generator (either the Unit diesel generator or the Unit 1/2 diesel generator) shall be operable whenever any work is being done which has the potential for draining the vessel, secondary containment is required, or a core or containment cooling system is required.

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1. Two RPS electric power monitoring channels for each inservice RPS MG set or inservice alternate power source shall be OPERABLE except when the reactor is in the SHUTDOWN mode.
2.
 - a. With one RPS electric power monitoring channel for an inservice RPS MG set or inservice alternate power source inoperable, restore the inoperable channel to OPERABLE status within 72 hours or remove the associated RPS MG set or alternate power source from service.
 - b. With both RPS electric power monitoring channels for an inservice RPS MG set or inservice alternate power source inoperable, restore at least one channel to OPERABLE status within 30 minutes, or remove the associated RPS MG set or alternate power source from service.

- a. Verifying de-energization of the emergency buses, and load shedding from the emergency buses.
- b. Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency buses with permanently connected loads, energizes the auto-connected emergency loads through the load sequencer, and operates for greater than 5 minutes while its generator is loaded with the emergency loads.

F. REACTOR PROTECTION BUS POWER MONITORING SYSTEM

1. The RPS Bus power monitoring system instrumentation shall be determined OPERABLE:
 - a. At least once per 6 months by performing a channel functional test, and
 - b. At least once per operating cycle by demonstrating the operability of overvoltage, undervoltage, and underfrequency protective instrumentation by performance of a channel calibration including simulated automatic activation of the protective relays, tripping logic, and output circuit breakers, and verifying the following setpoints:

| | |
|--------------------|--|
| (1) overvoltage | 126.5 V = 2.5% Min. 123.3 V Max. 129.6 V |
| (2) undervoltage | 108 V = 2.5% Min. 105.3 V Max. 110.7 V |
| (3) underfrequency | 56.0 Hz \pm 1% of 60 Hz Min. 55.4 Hz Max. 56.6 Hz |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 102 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 99 TO FACILITY OPERATING LICENSE NO. DPR-30
COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
DOCKET NOS. 50-254/265

1.0 INTRODUCTION

By letter dated May 14, 1986 Commonwealth Edison Co (CECo, the licensee) proposed to amend the Quad Cities Nuclear Power Station (Units 1 and 2) Technical Specifications (TS). The amendments would incorporate operability and surveillance requirements for the common unit 4-KV Bus Cross-tie into TS 3.9/4.9 (Auxiliary Electrical Systems). The necessity for these additional TS requirements came as result of an event at Dresden 2.

During the August 16, 1985 Dresden 2 event, normal offsite AC power was lost due to tripping of the Reserve Auxiliary Transformer (RAT) which was being fed by the 138-KV switchyard. Due to a transfer circuit deficiency, the two auxiliary power buses normally fed by this transformer failed to automatically transfer to the Unit Auxiliary Transformer (UAT) which was the second normal supply of AC power (from output of Main Generator). This failure to transfer ultimately led to a reactor scram and loss of Main Generator AC power to the UAT, constituting a complete loss of offsite AC power. As designed, both diesel generators automatically started providing AC power to essential equipment. During this event, another independent source of offsite power was available (but not utilized) via the Unit 2/3 4-KV Bus Cross-tie.

2.0 EVALUATION

GDC 17 of 10 CFR 50 Appendix A requires that electric power from the transmission network to the onsite electric distribution system shall be supplied by two physically independent circuits. By letter dated December 13, 1985, the licensee stated that the RAT of the other Dresden unit, supplied through a bus tie between corresponding safety-related buses of units 2/3, was the second offsite source of electric power mandated by GDC-17. Although this specific back-up source of offsite

power is not automatically transferred, it could be aligned quickly from the control room. Furthermore, offsite power capability via the cross-tie has been designed with sufficient capacity to operate one plant's total auxiliary loads during normal operation, while supplying adequate power to safely shutdown the other plant and also support all necessary engineered safeguards during an accident. This electrical configuration is essentially the same at Quad Cities 1/2.

However, existing TS 3.9.A only requires the 4-KV cross-tie, for Quad Cities 1/2, to be available if normal offsite power to one of the two units is lost. In a response dated December 13, 1985, to address concerns raised by the staff about compliance with GDC 17, the licensee committed to amend TS 3.9 for Dresden and Quad Cities. Subsequently by letter dated May 14, 1986, the licensee proposed changes to TS 3.9 which would establish operability and surveillance requirements for the common unit 4-KV cross-tie and thereby assure an alternative source of offsite AC power would be available.

In conclusion, the staff has reviewed the proposed TS and considers them acceptable requirements for ensuring availability of a second independent back-up source of offsite power via the 4-KV bus cross-tie that is consistent with GDC 17.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to the operability and surveillance requirements of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding of no significant hazards consideration for these amendments and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N. Trehan

Dated: August 6, 1987