OCT 1 9 1973

Docket Nos. 50-254 and 50-265

Commonwealth Edison Company
ATTN: Mr. J. S. Abel
Nuclear Licensing Administrator -
Boiling Water Reactors
Post Office Box 767
Chicago, Illinois 60690

Change No. 10
Operating Licenses Nos.
DPR-29 and DPR-30

Gentlemen:

Your letter dated November 6, 1972, proposed changes to Sections 3.3 and 4.3 of the Quad-Cities Units 1 and 2 Technical Specifications of Facility Operating Licenses Nos. DPR-29 and DPR-30. In response to our request dated July 24, 1973, you provided supplemental information by letter dated August 30, 1973. On October 19, 1973, you requested exemption from the present control rod drive surveillance requirements until the final change to the Technical Specifications is issued.

Our review consisted of a statistical analysis of control rod drive scram time data. We have concluded that the frequency of scram time measurements can be reduced provided that following each series of measurements for 50 percent of the control rod drives an analysis be made to verify performance or detect degradation which affect scram time.

During our review of the proposed changes to Sections 3.3 and 4.3, we informed your staff that certain modifications to your proposed changes were necessary to meet regulatory requirements. These modifications have been made.

On the basis of our review of your proposed Sections 3.3 and 4.3, as modified, we have concluded that these changes do not present significant hazards considerations and that there is reasonable assurance that the health and safety of the public will not be endangered by operation of these units in the manner proposed.

OCT 19 1973

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specifications appended to Facility Licenses Nos. DPR-29 and DPR-30 are hereby changed by replacing the existing pages 75 through 77 with the enclosed pages 75 through 77.

This change is effective as of October 19, 1973.

Sincerely,

Original Signed by:
Donald J. Skovholt

Donald J. Skovholt
Assistant Director for
Operating Reactors
Directorate of Licensing

Enclosures:
Revised pages 75 through 77

cc w/enclosures:
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SURNAME ▶	JIRiesland:sjh	RMDiggs	DLZiemann	DJSkovholt		
DATE ▶	10/19/73	10/19/73	10/19/73	10/19/73		

3.3 LIMITING CONDITION FOR OPERATION

5. During operation with limiting control rod patterns, as determined by the nuclear engineer, either:

   a. Both RBM channels shall be operable; or

   b. Control rod withdrawal shall be blocked; or

   c. The operating power level shall be limited so that the MCHFR will remain above 1.0 assuming a single error that results in complete withdrawal of any single operable control rod.

C. Scram Insertion Times

1. The average scram insertion time, based on the de-energization of the scram pilot valve solenoids at time zero, of all operable control rods in the reactor power operation condition shall be no greater than:

% Inserted From Fully Withdrawn	Avg. Scram Insertion Times (sec)
10	0.70
50	2.05
90	5.00

4.3 SURVEILLANCE REQUIREMENT

5. When a limiting control rod pattern exists, an instrument functional test of the RBM shall be performed prior to withdrawal of the designated rod(s) and daily thereafter.

C. Scram Insertion Times

3 | 1. After refueling outage and prior to operation above 30% power, with reactor pressure above 800 psig, all control rods shall be subject to scram-time measurements from the fully withdrawn position. The scram times shall be measured without reliance on the control rod drive pumps.

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3.3 LIMITING CONDITION FOR OPERATION	4.3 SURVEILLANCE REQUIREMENT

The average of the scram insertion times for the three fastest control rods of all groups of four control rods in a two by two array shall be no greater than:

% Inserted From Fully Withdrawn	Avg. Scram Insertion Times (sec)
10	0.74
50	2.17
90	5.30

2. The maximum scram insertion time for 90% insertion of any operable control rod shall not exceed 7.00 seconds.

3. If Specification 3.3.A.1 cannot be met, the reactor shall not be made supercritical; if operating, the reactor shall be shut down immediately upon determination that average scram time is deficient.

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4. If Specification 3.3.A.2 cannot be met, the deficient control rod shall be considered inoperable, fully inserted into the core, and electrically disarmed.

2. Following a controlled shutdown of the reactor, but not more frequently than 16 weeks nor less frequently than 32 weeks intervals, 50% of the control rod drives in each quadrant of the reactor core shall be measured for scram times specified in Specification 3.3.C. All control rod drives shall have experienced scram-time measurements each year. Whenever 50% of the control rod drives scram times have bee measured, an evaluation shall be made to provide reasonable assurance that proper control rod drives performance is being maintained. The results of measurements performed on the control rod drives shall be submitted in the semiannual operating report to the AEC.

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3.3 LIMITING CONDITION FOR OPERATION	4.3 SURVEILLANCE REQUIREMENT

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