

AUG 5 1974

Docket Nos. 50-254 and 50-265

Commonwealth Edison Company
ATTN: Mr. J. S. Abel
Nuclear Licensing Administrator -
Boiling Water Reactors
Post Office Box 767
Chicago, Illinois 60690

Gentlemen:

The Commission has issued the enclosed Amendment Nos. 12 and 6 to Facility Operating License Nos. DPR-29 and DPR-30, respectively. These amendments include Change No. 20 to the Technical Specifications, Appendix A, and are in response to your request dated March 11, 1974.

These amendments change the Technical Specifications of the subject licenses to permit sampling the reactor water and primary containment atmosphere, in the event of an occurrence resulting in primary containment isolation, to assess activity levels which initiated the isolation signal.

The Safety Evaluation and the Federal Register Notice relating to these actions are also enclosed.

Sincerely,

151
Karl R. Goller
Assistant Director
for Operating Reactors
Directorate of Licensing

GLainas
CLong
TJCanter
JIRiesland
CHEbron
BJones (8)
BScharf (15)
RMDiggs
SVarga
PCollins
SKari

Enclosures:

1. Amendments 12 and 6
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures. See next page

OFFICE >	CLORB #2	L:ORB #2	OGC	L:OR	KRG	
SURNAME >	JIRiesland	DLZiemann		KRGoller		Rg
DATE >	7/26/74	7/30/74	8/2/74	8/5/74		

AUG 5 1974

cc w/enclosures:

Mr. Charles Whitmore
President and Chairman
Iowa-Illinois Gas and
Electric Company
206 East Second Avenue
Davenport, Iowa 52801

Mr. Ed Vest
Environmental Protection Agency
1735 Baltimore Avenue
Kansas City, Missouri 64108

John W. Rowe, Esquire
Isham, Lincoln & Beale
Counselors at Law
One First National Plaza
Chicago, Illinois 60670

Anthony Z. Roisman, Esquire
Berlin, Roisman and Kessler
1712 N Street, N. W.
Washington, D. C. 20036

Mr. Robert W. Watts, Chairman
Rock Island County Board of
Supervisors
Rock Island County Courthouse
Rock Island, Illinois 61201

Moline Public Library

cc w/enclosures & cy of CE's filing
dtd 3/11/74:

Mr. Leroy Stratton
Bureau of Radiological Health
Illinois Department of Public Health
Springfield, Illinois 62706

Mr. Gary Williams
Federal Activities Branch
Environmental Protection Agency
1 N. Wacker Drive, Room 822
Chicago, Illinois 60606

COMMONWEALTH EDISON COMPANY AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY
DOCKET NO. 50-254
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12
License No. DPR-29

1. The Atomic Energy Commission (the Commission) has found that:
- A. The application for amendment by the Commonwealth Edison Company dated March 11, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public, and
 - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.
2. Accordingly, paragraph 3.3 of Facility License No. DPR-29 is hereby amended to read as follows:

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, attached to Facility Operating License No. DPR-29 are revised as indicated in the attachment to this license amendment. The Technical Specifications, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised."

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SURNAME➤						
DATE➤						

3. This license amendment is effective as of the date of its issuance.

FOR THE ATOMIC ENERGY COMMISSION

(S)
Karl R. Goller
Assistant Director
for Operating Reactors
Directorate of Licensing

Attachment:
Change No. 20 to Appendix A
Technical Specifications

Date of Issuance: AUG 5 1974

OFFICE >						
SURNAME >						
DATE >						

ATTACHMENT TO LICENSE AMENDMENT NO. 12

CHANGE NO. 20 TO APPENDIX A TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-29

Replace the existing pages 159 and 160 of the Technical Specifications with the enclosed revised pages bearing the same numbers.

The changed areas are reflected by marginal lines on each revised page.

QUAD CITIES TECHNICAL SPECIFICATIONS

TABLE 3.7.1

PRIMARY CONTAINMENT ISOLATION

Isolation Group	Valve Identification	Valve Number for Units 1 & 2	# of Power Operated Valves		Max. Operat. Time (Sec's)	Normal Operat. Position	Action on Initiating Signal
			In-Board	Out-Board			
<u>Main Steam Isolation</u>							
1	Mn Stm Isolation Valve	AO-203-1A,1B,1C,1D	4		*3<T<5	0	GC
1	Mn Stm Isolation Valve	AO-203-2A,2B,2C,2D		4	*3<T<5	0	GC
1	Mn Stm Drain Iso.Valve	MO-220-1	1		<35	C	SC
1	Mn Stm Drain Iso.Valve	MO-220-2		1	<35	C	SC
<u>Sampling</u>							
(1	Recirc.Sample Valve	AO-220-44	1		<5	0	GC
(1	Recirc.Sample Valve	AO-220-45		1	<5	0	GC
<u>RHR</u>							
2	RHR Discharge to Radwaste	MO-1001-20	1		<25	C	SC
2	RHR Discharge to Radwaste	MO-1001-21		1	<25	C	SC
2	Rx Shtdn Cooling Supply	MO-1001-47		1	<40	C	SC
2	Rx Shtdn Cooling Supply	MO-1001-50	1		<40	C	SC
2	Reactor Head Spray	MO-1001-60		1	<25	C	SC
2	Reactor Head Spray	MO-1001-63	1		<25	C	SC
<u>Pressure Suppression</u>							
2	Drywell Purge Valve	AO-1601-21	1		<10	C	SC
2	Vent Valve	AO-1601-22		1	<10	C	SC
2	Drywell Vent Valve	AO-1601-23	1		<10	C	SC
2	Vent to Rx Bldg Exh Sys.	AO-1601-24		1	<10	C	SC
2	Nitrogen Purge	AO-1601-55		1	<10	C	SC
2	Torus Purge Valve	AO-1601-56	1		<10	C	SC
2	Make Up Valve	MO-1601-57	1		<15	0	GC

3 | * Minimum

20 | Note 1: Valve can be reopened after isolation for sampling.

QUAD CITIES TECHNICAL SPECIFICATIONS

TABLE 3.7.1 (cont 'd)

Isolation Group	Valve Identification	Valve Number for Units 1 & 2	# of Power Operated Valves		Max. Operat. Time (Sec's)	Normal Operat. Position	Action on Initiating Signal
			In-Board	Out-Board			
2	Torus Make Up Valve	AO-1601-58		1	<15	O	GC
2	Drywell Make Up Valve	AO-1601-59		1	<15	O	GC
2	Torus Vent Valve	AO-1601-60	1		<10	C	SC
2	Torus 2" Vent Relief	AO-1601-61	1		<15	C	SC
2	Drywell 2" Vent Relief	AO-1601-62	1		<15	C	SC
2	Vent to S.G.T. System	AO-1601-63		1	<10	C	SC
2	Drywell Pneumatic Supply Isolation	AO-4720 AO-4721		2	<10	O	GC
<u>Radwaste</u>							
Note 1 (2)	Drywell Fl.Dr.Disch.	AO-2001-3	1		<20	O	GC
(2)	Drywell Fl.Dr.Disch.	AO-2001-4		1	<20	O	GC
(2)	Drywell Equip.Dr.Disch.	AO-2001-15	1		<20	O	GC
(2)	Drywell Equip.Dr.Disch.	AO-2001-16		1	<20	O	GC
<u>Oxygen Analyzer</u>							
2	Oxy.Analyzer Valve	AO-8801-A,B,C,D	4		<10	O	GC
2	Oxy.Analyzer Valve	AO-8802-A,B,C,D		4	<10	O	GC
2	Oxy.Analyzer Valve	AO-8803A	1		<10	O	GC
2	Oxy.Analyzer Valve	AO-8803B		1	<10	O	GC
<u>Traversing Incore Probe</u>							
2	On isolation signal, the T.I.P. detector is withdrawn if in use; five ball valves and one nitrogen purge are then closed.	T.I.P. Ball Valve 700-733 T.I.P. Purge Valve Assembly 700-743					

Note 1: Valve can be reopened after isolation for sampling.

COMMONWEALTH EDISON COMPANY AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. DPR-30

1. The Atomic Energy Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company dated March 11, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended, and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public, and
 - E. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.
2. Accordingly, paragraph 3.B of Facility License No. DPR-30 is hereby amended to read as follows:

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, attached to Facility Operating License No. DPR-30 are revised as indicated in the attachment to this license amendment. The Technical Specifications, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised."

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3. This license amendment is effective as of the date of its issuance.

FOR THE ATOMIC ENERGY COMMISSION

(S)

Karl R. Goller
Assistant Director
for Operating Reactors
Directorate of Licensing

Attachment:

Change No. 20 to Appendix A
Technical Specifications

Date of Issuance: AUG 5 1974

OFFICE >						
SURNAME >						
DATE >						

ATTACHMENT TO LICENSE AMENDMENT NO. 6

CHANGE NO. 20 TO APPENDIX A TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-30

Replace the existing pages 159 and 160 of the Technical Specifications with the enclosed revised pages bearing the same numbers.

The changed areas are reflected by marginal lines on each revised page.

QUAD CITIES TECHNICAL SPECIFICATIONS

TABLE 3.7.1

PRIMARY CONTAINMENT ISOLATION

Isolation Group	Valve Identification	Valve Number for Units 1 & 2	# of Power Operated Valves		Max. Operat. Time (Sec's)	Normal Operat. Position	Action on Initiating Signal
			In-Board	Out-Board			
<u>Main Steam Isolation</u>							
1	Mn Stm Isolation Valve	AO-203-1A,1B,1C,1D	4		*3<T<5	O	GC
1	Mn Stm Isolation Valve	AO-203-2A,2B,2C,2D		4	*3<T<5	O	GC
1	Mn Stm Drain Iso.Valve	MO-220-1	1		<35	C	SC
1	Mn Stm Drain Iso.Valve	MO-220-2		1	<35	C	SC
<u>Sampling</u>							
Note (1)	Recirc.Sample Valve	AO-220-44	1		<5	O	GC
1 (1)	Recirc.Sample Valve	AO-220-45		1	<5	O	GC
<u>RHR</u>							
2	RHR Discharge to Radwaste	MO-1001-20	1		<25	C	SC
2	RHR Discharge to Radwaste	MO-1001-21		1	<25	C	SC
2	Rx Shtdn Cooling Supply	MO-1001-47		1	<40	C	SC
2	Rx Shtdn Cooling Supply	MO-1001-50	1		<40	C	SC
2	Reactor Head Spray	MO-1001-60		1	<25	C	SC
2	Reactor Head Spray	MO-1001-63	1		<25	C	SC
<u>Pressure Suppression</u>							
2	Drywell Purge Valve	AO-1601-21	1		<10	C	SC
2	Vent Valve	AO-1601-22		1	<10	C	SC
2	Drywell Vent Valve	AO-1601-23	1		<10	C	SC
2	Vent to Rx Bldg Exh Sys.	AO-1601-24		1	<10	C	SC
2	Nitrogen Purge	AO-1601-55		1	<10	C	SC
2	Torus Purge Valve	AO-1601-56	1		<10	C	SC
2	Make Up Valve	MO-1601-57	1		<15	O	GC

3 * Minimum

Note 1: Valve can be reopened after isolation for sampling.

QUAD CITIES TECHNICAL SPECIFICATIONS

TABLE 3.7.1 (cont 'd)

Isolation Group	Valve Identification	Valve Number for Units 1 & 2	# of Power Operated Valves		Max. Operat. Time (Sec's)	Normal Operat. Position	Action on Initiating Signal
			In- Board	Out- Board			
2	Torus Make Up Valve	AO-1601-58		1	<15	O	GC
2	Drywell Make Up Valve	AO-1601-59		1	<15	O	GC
2	Torus Vent Valve	AO-1601-60	1		<10	C	SC
2	Torus 2" Vent Relief	AO-1601-61	1		<15	C	SC
2	Drywell 2" Vent Relief	AO-1601-62	1		<15	C	SC
2	Vent to S.G.T. System	AO-1601-63		1	<10	C	SC
2	Drywell Pneumatic Supply Isolation	AO-4720 AO-4721		2	<10	O	GC
<u>Radwaste</u>							
Note 1 (2)	Drywell Fl.Dr.Disch.	AO-2001-3	1		<20	O	GC
(2)	Drywell Fl.Dr.Disch.	AO-2001-4		1	<20	O	GC
(2)	Drywell Equip.Dr.Disch.	AO-2001-15	1		<20	O	GC
(2)	Drywell Equip.Dr.Disch.	AO-2001-16		1	<20	O	GC
<u>Oxygen Analyzer</u>							
2	Oxy.Analyzer Valve	AO-8801-A,B,C,D	4		<10	O	GC
2	Oxy.Analyzer Valve	AO-8802-A,B,C,D		4	<10	O	GC
2	Oxy.Analyzer Valve	AO-8803A	1		<10	O	GC
2	Oxy.Analyzer Valve	AO-8803B		1	<10	O	GC
<u>Traversing Incore Probe</u>							
2	On isolation signal, the T.I.P. detector is withdrawn if in use; five ball valves and one nitrogen purge are then closed.	T.I.P. Ball Valve 700-733 T.I.P. Purge Valve Assembly 700-743					

Note 1: Valve can be reopened after isolation for sampling.

SAFETY EVALUATION BY THE DIRECTORATE OF LICENSING

DOCKET NOS. 50-254 AND 50-265

COMMONWEALTH EDISON COMPANY

QUAD-CITIES UNITS 1 & 2

INTRODUCTION

In a letter dated March 11, 1974, Commonwealth Edison Company (CE) proposed a change to the technical specifications of Quad-Cities Units 1 and 2. These changes relate to a proposed modification of the facilities to install key-locked switches for operating the drywell and reactor water recirculating system sample valves. This equipment modification would provide means to open the sample valves to obtain appropriate samples without having to use emergency jumpers in the valve operating circuit when a primary containment isolation signal is present.

EVALUATION

In the analysis for the facility modification and technical specification change, CE states that safety to the public is not reduced by operation in the manner proposed. The basis for this statement, with which we agree, is that the keys for the switches will be under administrative control and thus operation of the valves by use of key-lock switches will be less prone to error than operation by the use of emergency jumpers.

We concur with the licensee's finding that with this modification, the probability of release to the environs is not increased because (1) if a bypass switch is left in the bypass position, the release of reactor recirculation water or drywell gases would be into their respective closed sample systems and no release to the environment would occur and (2) if either sample system fails, any release would be into the secondary containment and not into the environment surrounding the secondary containment structure.

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CONCLUSION

Based on the above, the staff concludes that (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration; and (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner.

151

John I. Riesland
Operating Reactors Branch #2
Directorate of Licensing

151

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Directorate of Licensing

Date: AUG 5 1974

OFFICE ➤						
SURNAME ➤						
DATE ➤						

UNITED STATES ATOMIC ENERGY COMMISSION

DOCKET NOS. 50-254 AND 50-265

COMMONWEALTH EDISON COMPANY AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY LICENSES

Notice is hereby given that the U. S. Atomic Energy Commission (the Commission) has issued Amendment Nos. 12 and 6 to Facility Operating License Nos. DPR-29 and DPR-30 (respectively) to the Commonwealth Edison Company (acting for itself and on behalf of the Iowa-Illinois Gas and Electric Company) which revised Technical Specifications for operation of the Quad-Cities Units 1 and 2 (located in Rock Island County, Illinois).

The amendments permit electrical circuit changes which allow convenient sampling of reactor water and primary containment atmosphere in the event of an occurrence which causes containment isolation.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations, and the Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I which are set forth in the license amendments.

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For further details with respect to these actions, see (1) the application for amendment dated March 11, 1974, (2) Amendment Nos. 12 and 6 to License Nos. DPR-29 and DPR-30, with any attachments, and (3) the letter to Commonwealth (transmitting Amendments 12 and 6) which includes an evaluation of the application. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Moline Public Library at 504 - 17th Street in Moline, Illinois 60265.

A copy each of items (2) and (3) may be obtained upon request addressed to the Atomic Energy Commission, Washington, D. C. 20545, Attention: Deputy Director for Reactor Projects, Directorate of Licensing - Regulation.

Dated at Bethesda, Maryland, this *5th* day of August 1974.

FOR THE ATOMIC ENERGY COMMISSION

LS

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Directorate of Licensing

OFFICE ➤						
SURNAME ➤						
DATE ➤						

Docket No. 50-254 & 50-265

Commonwealth Edison Company
ATTN: Mr. J. S. Abel
Nuclear Licensing Administrator -
Boiling Water Reactors
Post Office Box 767
Chicago, Illinois 60690

Gentlemen:

Your letter dated March 11, 1974, proposed a change to Appendix A (Technical Specifications) of Facility License Nos. DPR-29 and DPR-30 for the Quad-Cities Station. These changes relate to a proposed modification of the facilities to allow opening the reactor water recirculating system and drywell sample valves by the use of key-locked switches when a primary containment isolation signal is present. The proposed change to the technical specifications, with appropriate revisions to the PSAR, would allow sampling reactor water and primary containment atmosphere, in the event of an occurrence resulting in primary containment isolation, to assess activity levels which initiated the isolation signal.

In your analysis for the proposed changes, you state that the safety of operation of the facilities is not changed during normal operation because sampling capability exists both with and without bypass switches. The bypass switches will be key-locked switches under administrative controls. This modification replaces the emergency jumpers, the present procedures, which is prone to an error that might result in opening of other than these sample valves. With this modification you have found and we concur that the probability of release to the environs is not increased because (1) if a bypass switch is left in the bypass position the release of recirculating water or drywell air would be into their respective closed sample systems and no release to the environment would occur and (2) if either sample system fails, any release would be into the secondary containment and not into the surrounding environment.

Based on our review of your design criteria and evaluation of failure consequences associated with the modification, we have concluded that the proposed amendment does not involve significant new safety information of a type not considered by any previous Commission safety review of the facility; potentially involve a significant increase in the probability or consequence of an accident considered in a previous Commission safety review of the facility; or involve a potentially significant decrease in the margin of safety during normal plant operations, anticipated operational occurrence, or postulated accidents considered in any previous Commission safety review of the facility and, therefore, does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this change.

Accordingly, Amendment Nos. 12 and 6 to Facility Operating Licenses Nos. DPR-29 and DPR-30, respectively, are enclosed revising the Technical Specifications as proposed. A copy of a notice which is being forwarded to the Office of the Federal Register for publication relating to this action also is enclosed for your information.

Sincerely,

Karl R. Goller
Assistant Director
for Operating Reactors
Directorate of Licensing

Enclosures:

1. Amendment No. 12 to License No. DPR-29
2. Amendment No. 6 to License No. DPR-30
3. Federal Register Notice

cc w/enclosures:

See next page

Distribution

Docket File	WOMiller
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