

April 9, 1996

Mr. Roger O. Anderson, Director
Licensing and Management Issues
Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT - ISSUANCE OF AMENDMENT RE:
REVISION OF TECHNICAL SPECIFICATION 4.7.D.4 FOR REPLACEMENT OF SEAT
SEALS FOR THE DRYWELL AND SUPPRESSION CHAMBER PURGE AND VENT VALVES
(TAC NO. M94309)

Dear Mr. Anderson:

The Commission has issued the enclosed Amendment No. 96 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The amendment consists of changes to the Technical Specifications (TS) in response to your application dated March 1, 1996, which superseded your December 11, 1995, application.

The amendment modifies TS Section 4.7, Surveillance Requirements for Primary Containment Automatic Isolation Valves. Specifically, the amendment revises TS Surveillance Requirement 4.7.D.4, which requires replacement of the seat seals for the drywell and suppression chamber purge and vent valves every 5 years, to change the replacement frequency to every six operating cycles.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By:

Tae Kim, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosures: 1. Amendment No. 96 to DPR-22
2. Safety Evaluation

cc w/encl: See next page

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*SEE PREVIOUS CONCURRENCE

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NAME	CJamerson		TJKim		CBerlinger	EHolder	JHannon
DATE	3/15/96		3/15/96		03/07/96	3/25/96	3/15/96

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DATED: April 9, 1996.

AMENDMENT NO. 96 TO FACILITY OPERATING LICENSE NO. DPR-22-MONTICELLO

Docket File
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The amendment modifies TS Section 4.7, Surveillance Requirements for Primary Containment Automatic Isolation Valves. Specifically, the amendment revises TS Surveillance Requirement 4.7.D.4, which requires replacement of the seat seals for the drywell and suppression chamber purge and vent valves every 5 years, to change the replacement frequency to every six operating cycles.

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OFFICE	LA:PD31	E	PM:PD31	E	BC:SCSB	E	OGC		D:PD31
NAME	CJamerson		TJKim		CBerlinger				JHannon
DATE	3/11/96				3/17/96				3/15/96

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 9, 1996

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Northern States Power Company
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Minneapolis, MN 55401

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Sincerely,

A handwritten signature in cursive script, appearing to read "Tae Kim".

Tae Kim, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosures: 1. Amendment No. 96 to DPR-22
2. Safety Evaluation

cc w/encl: See next page

Mr. Roger O. Anderson, Director
Northern States Power Company

Monticello Nuclear Generating Plant

cc:

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January 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 96
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated March 1, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 96 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Tae Kim, Project Manager
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: April 9, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 96

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

REMOVE

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INSERT

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3.0 LIMITING CONDITIONS FOR OPERATION

2. In the event any Primary Containment automatic isolation valve becomes inoperable, reactor operation in the run mode may continue provided at least one valve in each line having an inoperable valve is closed.
3. If Specification 3.7.D.1 and 3.7.D.2 cannot be met, initiate normal orderly shutdown and have reactor in the cold shutdown condition within 24 hours.

3.7/4.7

4.0 SURVEILLANCE REQUIREMENTS

- d. At least once per week the main steam-line power-operated isolation valves shall be exercised by partial closure and subsequent reopening.
2. Whenever a Primary Containment automatic isolation valve is inoperable, the position of at least one fully closed valve in each line having an inoperable valve shall be recorded daily.
3. Deleted
4. The seat seals of the drywell and suppression chamber 18-inch purge and vent valves shall be replaced at least once every six operating cycles. If periodic Type C leakage testing of the valves performed per surveillance requirement 4.7.A.2.b identifies a common mode test failure attributable to seat seal degradation, then the seat seals of all drywell and suppression chamber 18-inch purge and vent valves shall be replaced.

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Amendment No. 71, 77, 96



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 96 TO FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY
MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

By application dated March 1, 1996, the Northern States Power Company (the licensee) requested an amendment to the Technical Specifications (TS) appended to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. This application superseded the licensee's previous application dated December 11, 1995. The proposed amendment would modify TS Section 4.7, Surveillance Requirements for Primary Containment Automatic Isolation Valves, to revise Surveillance Requirement 4.7.D.4, which requires replacement of the seat seals for the drywell and suppression chamber purge and vent valves every 5 years, to change the seat replacement frequency to every six operating cycles. Further, the amendment would add a new requirement to replace the seat seals of all drywell and suppression chamber 18-inch purge and vent valves if local leak rate testing identified a common mode test failure attributable to seat seal degradation.

2.0 EVALUATION

Surveillance Requirement 4.7.D.4 was established in the Monticello TS by Amendment No. 64, dated May 10, 1989. The changes incorporated by Amendment No. 64 provided resolution, in part, to the NRC's Multi-Plant Action (MPA) B-24, purge and vent valve operability. In response to the staff's position requiring accelerated local leakage testing of containment purge and vent valves due to the poor performance in the industry of butterfly valves, Monticello proposed to add the surveillance requirement to replace the seals of the 18-inch drywell and suppression chamber purge and vent valves every 5 years. In a letter dated November 22, 1985, the licensee indicated that the T-shaped seal valve design for the Monticello purge and vent valves is relatively free of the valve leakage problems found at other facilities and that the existing preventive maintenance program with periodic seal replacement would provide assurance that the valve would seal tightly. In the November 22, 1985, letter, the licensee provided valve leakage test data for the period 1970 to 1985 to support the problem-free performance of these valves.

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The licensee has since reviewed the maintenance history and the results of the leakage test program for the primary containment purge and vent valves for the period 1985 to present. The results of the leakage test program for the primary containment purge and vent valves was provided in the licensee's letters dated December 11, 1995, and March 1, 1996. The licensee has indicated that this data supports the conclusion that no significant improvement in valve leakage performance can be attributed to replacement of the valve's elastomer T-shaped seat seal. Furthermore, the results of the leakage tests performed between the elastomer T-shaped seat seal replacement during the 1986 outage and the 1991 outage and subsequent to the 1991 outage indicate no adverse change in valve seat leakage performance.

The licensee also indicated that the leakage test data for the torus to reactor building vacuum breaker isolation valves provide further indication for allowing the revision of Surveillance Requirement 4.7.D.4. The torus to reactor building vacuum breaker isolation valves are 20-inch air-operated butterfly valves of the same model and design as the primary containment purge and vent valves. These valves employ the same T-shaped elastomer seat seal as the purge and vent valves and are not subject to the 5-year seat seal replacement requirement. The T-shaped elastomer seat seals of these valves were replaced in 1986. Since 1986, the torus to reactor building vacuum breaker isolation valves have shown no degradation of valve seat leakage attributed to a failure of the T-shaped elastomer seat seal.

The licensee is not proposing any change to the existing requirements contained in the TS for the leak testing of the primary containment purge and vent valves per 10 CFR Part 50, Appendix J, which requires Type C tests to be "performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years."

This amendment adds a new requirement to replace the seat seals of all drywell and suppression chamber 18-inch purge and vent valves if Type C leakage test identified a common mode test failure attributable to seat seal degradation. This requirement will help to maintain a high level of assurance that the ability of the purge and vent valves is maintained, thus providing an added level of conservatism.

The staff has reviewed the licensee's submittal, including the data provided in support of the revision of Surveillance Requirement 4.7.D.4, and finds the proposed change to be acceptable. Based upon the licensee's operational experience, the fact that the primary containment purge and vent valves will still be local leak rate tested in accordance with 10 CFR Part 50, Appendix J, and the new seat seal replacement requirement, the staff finds that the revision of Surveillance Requirement 4.7.D.4 to require replacement of the purge and vent valve seat seals every six operating cycles instead of 5 years is acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration and there has been no public comment on such finding (61 FR 9504). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Janet Kennedy
James Pulsipher

Date: April 9, 1996