NOV 2 5 1975

Docket Nos. 50-254/265

Commonwealth Edison Company ATTN: Mr. R. L. Bolger Assistant Vice President Post Office Box 767 Chicago, Illinois 60690

Gentlemen:

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The Commission has issued the enclosed Amendment Nos. 21 and 17 to Facility Operating License Nos. DPR-29 and DPR-30 for the Quad Citles Units 1 and 2, respectively. These amendments consist of Change No. 33 in the Technical Specifications in accordance with your request dated November 17, 1975.

These amendments incorporate into the Quad Cities Units 1mand 2 Technical Specifications a new definition of "surveillance interval" that permits the licensee to extend the interval between required surveillance by up to 25% provided that the total combined interval time for any three consecutive surveillance intervals does not exceed 3.25 times the specified surveillance interval.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Original signed by Dennis L. Ziemann Dennis L. Ziemann, Chief Operating Reactors Branch #2 Division of Reactor Licensing

Enclosures:

- 1. Amendment Nos. 21 and 17
- 2. Safety Evaluation
- 3. Pederal Register Notice

Notified N. Kalevianakan Of C flowt Super on 11/26/15 hat this amendment was signed

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Commonwealth Edison Company

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cc w/enclosures: Mr. Charles Whitmore President and Chairman Iowa-Illinois Gas and Electric Company 206 East Second Avenue Davenport, Iowa 52801

John W. Rowe, Esquire Isham, Lincoln & Beale Counselors at Law One First National Plaza Chicago, Illinois 60670

Anthony Z. Roisman, Esquire Berlin, Roisman and Kessler 1712 N Street, N. W. Washington, D. C. 20036

Moline Public Library 504 - 17th Street Moline, Illinois 61265

Mr. Robert W. Watts, Chairman Rock Island County Board of Supervisors Rock Island County Court House Rock Island, Illinois 61201

cc w/enclosures and cy of CECo's
 filing dtd. 11/17/75:
Mr. Leroy Stratton
Bureau of Radiological Health
Illinois Department of Public Health
Springfield, Illinois 62706

UNITED STATES

WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY AND IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NOS. 50-254 AND 50-265

QUAD CITIES UNITS'1 AND 2

AMENDMENT TO FACILITY OPERATING LICENSES

Amendment No. 21 & 19 License No. DPR-29 DPR-30

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated November 17, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
- Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No.DPR-29 & -30 is hereby amended to read as follows:



"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, As revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 33."

3. These license amendments become effective as of their date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original signed by Dennis L. Ziemann

Dennis L. Ziemann, Chief Operating Reactors Branch #2 Division of Reactor Licensing

Attachment:

Change No. 33 to the Technical Specifications

Date of Issuance: NOV 2 5 1975

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ATTACHMENT TO LICENSE AMENDMENT NOS. 21 AND 19

CHANGE NO. 33 TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET 105. 50-254 AND 50-265

Replace the existing page 5 of the Technical Specifications with the enclosed revised page 5. The changed area is reflected by memarginal line on the revised page.

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may not in itself result in serious consequences but it indicates an operational deficiency subject to regulatory review.

- Y. <u>Secondary Containment Integrity</u> Secondary containment integrity means that the reactor building is intact and the following conditions are met:
 - 1. At least one door in each access opening is closed.
 - 2. The standby gas treatment system is operable.
 - 3. All reactor building automatic ventilation system isolation valves are operable or are secured in the isolated position.
- Z. <u>Shutdown</u> The reactor is in a shutdown condition when the reactor mode switch is in the shutdown mode position and no core alterations are being performed.
 - Hot Shutdown means conditions as above with reactor coolant temperature greater than 212°F.
 - 2. Cold Shutdown means conditions as above with reactor coolant temperature equal to or less than 212°F.
- AA. <u>Simulated Automatic Actuation</u> Simulated automatic actuation means applying a simulated signal to the sensor to actuate the circuit in question.

- BB. Total Peaking Factor The Total Peaking Factor (TPF) is the highest product of radial, axial, and local peaking factors simultaneously operative at any segment of fuel rod.
- CC. <u>Transition Boiling</u> Transition boiling means the boiling regime between nucleate and film boiling. Transition boiling is the regime in which both nucleate and film boiling occur intermittently with neither type being completely stable.
- DD. <u>Critical Power Ratio (CPR)</u> The critical power ratio is the ratio of that assembly power which causes some point in the assembly to experience transition boiling to the assembly power at the reactor condition of interest as calculated by application of the GEXL correlation. (Reference NEDO-10958)
- EE. <u>Minimum Critical Power Ratio (MCPR)</u> The minimum in-core critical power ratio corresponding to the most limiting fuel assembly in the core.
- FF <u>Surveillance Interval</u> Each surveillance requirement shall be performed within the specified surveillance interval with:
 - a. A maximum allowable extension not to exceed 25% of the surveillance interval.
 - b. A total maximum combined interval time for any 3 consecutive surveillance intervals not to exceed 3.25 times the specified surveillance interval.

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 21 AND 19 TO FACILITY LICENSE NOS. DPR-29 AND DPR-30

CHANGE NO. 33 TO THE TECHNICAL SPECIFICATIONS

COMMONWEALTH EDISON COMPANY AND IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

QUAD CITIES UNITS 1 AND 2

DOCKET NOS. 50-254 AND 50-265

INTRODUCTION

By letter dated November 17, 1975, the Commonwealth Edison Company (CECo) requested an amendment to Facility License Nos. DPR-29 and DPR-30 for Quad Cities Station Units 1 and 2. The proposed request involves revisions to the Technical Specifications with regard to the addition of a new definition of "surveillance interval" that would permit CECo to extend the interval between required surveillance by up to 25% provided that the total combined interval time for any three consecutive surveillance intervals does not exceed 3.25 times the specified surveillance interval.

DISCUSSION

Some of the surveillance requirements contained in the technical specifications of licensed facilities cannot be performed with the reactor operating. In some cases reactors have been required to shut down to complete a surveillance requirement before a surveillance interval had passed.

The NRC staff has recognized that the surveillance intervals established in the technical specifications have been conservatively chosen based upon statistical analyses and contain a substantial margin of tolerance beyond any expected failure interval.

Because of the tolerance factor built into these surveillance intervals, the NRC staff has developed a standard surveillance tolerance which permits a licensee to extend a surveillance interval by up to 25% provided that the total combined interval time for any three consecutive intervals does not exceed 3.25 times the specified surveillance interval. This tolerance is incorporated in the standard technical specifications that have been developed by the NRC staff and are being issued with all new operating licenses.

EVALUATION

We have reviewed the proposed change to the Technical Specifications proposed by CECo. We have concluded that the proposed change does not result in an increase of the nominal interval during which the specified surveillance must be performed. The change in the specification is administrative in nature and reflects the staff's position that minor adjustments to the nominal surveillance intervals established in the technical specifications are acceptable, provided that these adjustments do not degrade the reliability associated with the surveillance activity significantly beyond that obtained from the nominal specific interval.

In the CECo proposal, the allowed extension to the nominal surveillance interval of 25% is further restricted by requiring that over a period of three consecutive surveillance intervals, the cumulative extension of the nominal surveillance interval must be no greater than 3.25 times the specified surveillance interval.

The numerical values of the surveillance interval extensions requested by CECo are the same as those currently being issued by the NRC on newly licensed facilities. Therefore, this change is a further step in seeking uniformity and clarity in the Technical Specifications for operating reactors. We conclude that this change does not modify the safety provisions of the Technical Specifications and is therefore acceptable.

CONCLUSION

We have concluded, based on the considerations above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: NOV 2 5 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION DOCKET NOS. 50-254 AND 50-265 <u>COMMONWEALTH EDISON COMPANY</u> <u>AND</u> IOWA-ILLINOIS GAS AND ELECTRIC COMPANY NOTICE OF ISSUANCE OF AMENDMENTS TO

FACILITY OFERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 21 and 19 to Facility Operating License Nos. DPR-29 and DPR-30, issued to the Commonwealth Edison Company (acting for itself and on behalf of the Iowa-Illinois Gas and Electric Company), which revised Technical Specifications for operation of the Quad Cities Units 1 and 2 located in Rock Island County, Illinois. These amendments are effective as of their date of issuance.

These amendments add a new definition of the term "Surseillance Interval" to the Technical Specifications for Quad Cities Units 1 and 2 that permits the licensee to extend the interval between required surveillance by up to 25% provided that the total combined interval time for any three consecutive surveillance intervals does not exceed 3.25 times the specified surveillance interval.

The application for these amendments dated November 17, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments is not required since these amendments do not involve a significant hazards consideration.

For further details with respect to this action, see (1) the application for these amendments dated November 17, 1975, (2) Amendment Nos. 21 and 19 to License Nos. DPR-29 and DPR-30, with Change No. 33, and (3) the Commission's concurrently issued related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C., and at the Moline Public Library, at 504 - 17th Street in Moline, Illinois 60265.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 25th day of Morember, 1925.

FOR THE NUCLEAR REGULATORY COMMISSION Original signed by Dennis L. Ziemann

Dennis L. Ziemann, Chief Operating Reactors Branch #2 Division of Reactor Licensing

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