

OFFICE	SURNAME	DATE
OR:ORB #2	RMDiggs	2/2/76
OR:ORB #2	RSilver:ro	2/13/76
OR:ORB #2	PWO'Connor	2/11/76
OEID	S.WANNEN	2/19/76
OR:ORB #2	DLZiemann	2/24/76
OR:ORB #2	RLPaer	2/11/76

OR:AD
K.Gollen
2/1/76

- Enclosures:
1. Amendment No. 15 to License DPR-19
 2. Amendment No. 13 to License DPR-25
 3. Amendment No. 23 to License DPR-29
 4. Amendment No. 22 to License DPR-39
 5. Safety Evaluation
 6. Federal Register Notice

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Sincerely,
Dennis L. Ziemann

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

In response to your request dated October 29, 1975 and in accordance with telephone conversations with your staff, the Commission has issued the enclosed Amendment Nos. 15 and 13 to Facility Operating License Nos. DPR-19 and DPR-25 for Units 2 and 3 of the Dresden Nuclear Power Station and Amendment Nos. 23 and 22 to Facility Operating License Nos. DPR-29 and DPR-39 for Units 1 and 2 of the Quad Cities Nuclear Power Station. These amendments consist of changes in the Technical Specifications that limit reactor operation to a period of 24 hours with one recirculation loop out of service, and deletes a license condition for Dresden 3 and Quad Cities 2 regarding single loop operation.

Gentlemen:
Commonwealth Edison Company
ATTN: Mr. R. L. Bolger
Assistant Vice President
Post Office Box 767
Chicago, Illinois 60690

- DISTRIBUTION
- OPA (Clare Miles)
 - VStello
 - TBabernathy
 - JRBuchanan
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 - KRGoller
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 - PWO'Connor
 - OEID
 - OIGB (3)
 - Bjomes (16)
 - Bschart (15)
 - JMCGough
 - ACRS (16)

Packet Nos. 50-257, 50-249
50-254, 50-265



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 26, 1976

Docket Nos. 50-237, 50-249
~~50-254~~, 50-265

Commonwealth Edison Company
ATTN: Mr. R. L. Bolger
Assistant Vice President
Post Office Box 767
Chicago, Illinois 60690

Gentlemen:

In response to your request dated October 29, 1975 and in accordance with telephone conversations with your staff, the Commission has issued the enclosed Amendment Nos. 15 and 13 to Facility Operating License Nos. DPR-19 and DPR-25 for Units 2 and 3 of the Dresden Nuclear Power Station and Amendment Nos. 23 and 22 to Facility Operating License Nos. DPR-29 and DPR-30 for Units 1 and 2 of the Quad Cities Nuclear Power Station.

These amendments consist of changes in the Technical Specifications that limit reactor operation to a period of 24 hours with one recirculation loop out of service, and deletes a license condition for Dresden 3 and Quad Cities 2 regarding single loop operation.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Dennis L. Ziemann
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosures:

1. Amendment No. 15 to License DPR-19
2. Amendment No. 13 to License DPR-25
3. Amendment No. 23 to License DPR-29
4. Amendment No. 22 to License DPR-30
5. Safety Evaluation
6. Federal Register Notice

February 26, 1976

cc w/enclosures:

Mr. Charles Whitmore
President and Chairman
Iowa-Illinois Gas and
Electric Company
206 East Second Avenue
Davenport, Iowa 52801

John W. Rowe, Esquire
Isham, Lincoln & Beale
Counselors at Law
One First National Plaza
Chicago, Illinois 60603

Anthony Z. Roisman, Esquire
Berlin, Roisman and Kessler
1712 N Street, N. W.
Washington, D. C. 20036

Moline Public Library
504 - 17th Street
Moline, Illinois 61265

Morris Public Library
604 Liberty Street
Morris, Illinois 60451

Mr. William Waters
Chairman, Board of Supervisors
of Grundy County
Grundy County Courthouse
Morris, Illinois 60450

Mr. Robert W. Watts, Chairman
Rock Island County Board of
Supervisors
Rock Island County Courthouse
Rock Island, Illinois 61201

cc w/enclosures and cy of CECO's
filing dtd. 10/29/75:

Mr. Leroy Stratton
Bureau of Radiological Health
Illinois Department of Public Health
Springfield, Illinois 62706



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION UNIT 2

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 15
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated October 29, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date Of Issuance: February 26, 1976

ATTACHMENT TO LICENSE AMENDMENT NOS. 15 AND 13

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Replace the existing pages 91a and 99a of the Technical Specifications with the attached revised pages bearing the same numbers. Changes on these pages are shown by marginal lines.

3.6 LIMITING CONDITION FOR OPERATION

H. Recirculation Pump Flow Mismatch

1. Whenever both recirculation pumps are in operation, pump speeds shall be maintained within 10% of each other when power level is greater than 80% and within 15% of each other when power level is less than 80%.
2. If specification 3.6.H.1 cannot be met, one recirculation pump shall be tripped.
3. The reactor shall not be operated with one recirculation loop out of service for more than 24 hours. With the reactor operating, if one recirculation loop is out of service the plant shall be placed in a hot shutdown condition within 24 hours unless the loop is sooner returned to service.
4. Whenever one pump is operable and the remaining pump is in the tripped position, the operable pump shall be at a speed less than 65% before starting the inoperable pump.

4.6 SURVEILLANCE REQUIREMENT

3. The baseline data required to evaluate the conditions in Specifications 4.6.G.1 and 4.6.G.2 will be acquired each operating cycle.

H. Recirculation Pump Flow Mismatch

Recirculation pumps speed shall be checked daily for mismatch.

H. Recirculation Pump Flow Mismatch

The LPCI loop selection logic has been described in the Dresden Nuclear Power Station Units 2 and 3 FSAR, Amendments 7 and 8. For some limited low probability accidents with the recirculation loop operating with large speed differences, it is possible for the logic to select the wrong loop for injection. For these limited conditions the core spray itself is adequate to prevent fuel temperatures from exceeding allowable limits. However, to limit the probability even further, a procedural limitation has been placed on the allowable variation in speed between the recirculation pumps.

The licensee's analyses indicate that above 80% power the loop select logic could not be expected to function at a speed differential of 15%. Below 80% power the loop select logic would not be expected to function at a speed differential of 20%. This specification provides a margin of 5% in pump speed differential before a problem could arise. If the reactor is operating on one pump, the loop select logic trips that pump before making the loop selection.

In addition, during the start-up of Dresden Unit 2 it was found that a flow mismatch between the two sets of jet pumps caused by a difference in recirculation loops could set up a vibration until a mismatch in speed of 27% occurred. The 10% and 15% speed mismatch restrictions provide additional margin before a pump vibration problem will occur.

ECCS performance during reactor operation with one recirculation loop out of service has not been analyzed. Therefore, sustained reactor operation under such conditions is not permitted.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated October 29, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.G of the license is deleted in its entirety.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 26, 1976

ATTACHMENT TO LICENSE AMENDMENT NOS. 15 AND 13

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Replace the existing pages 91a and 99a of the Technical Specifications with the attached revised pages bearing the same numbers. Changes on these pages are shown by marginal lines.

3.6 LIMITING CONDITION FOR OPERATION

H. Recirculation Pump Flow Mismatch

1. Whenever both recirculation pumps are in operation, pump speeds shall be maintained within 10% of each other when power level is greater than 80% and within 15% of each other when power level is less than 80%.
2. If specification 3.6.H.1 cannot be met, one recirculation pump shall be tripped.
3. The reactor shall not be operated with one recirculation loop out of service for more than 24 hours. With the reactor operating, if one recirculation loop is out of service the plant shall be placed in a hot shutdown condition within 24 hours unless the loop is sooner returned to service.
4. Whenever one pump is operable and the remaining pump is in the tripped position, the operable pump shall be at a speed less than 65% before starting the inoperable pump.

4.6 SURVEILLANCE REQUIREMENT

3. The baseline data required to evaluate the conditions in Specifications 4.6.G.1 and 4.6.G.2 will be acquired each operating cycle.

H. Recirculation Pump Flow Mismatch

Recirculation pumps speed shall be checked daily for mismatch.

H. Recirculation Pump Flow Mismatch

The LPCI loop selection logic has been described in the Dresden Nuclear Power Station Units 2 and 3 FSAR, Amendments 7 and 8. For some limited low probability accidents with the recirculation loop operating with large speed differences, it is possible for the logic to select the wrong loop for injection. For these limited conditions the core spray itself is adequate to prevent fuel temperatures from exceeding allowable limits. However, to limit the probability even further, a procedural limitation has been placed on the allowable variation in speed between the recirculation pumps.

The licensee's analyses indicate that above 80% power the loop select logic could not be expected to function at a speed differential of 15%. Below 80% power the loop select logic would not be expected to function at a speed differential of 20%. This specification provides a margin of 5% in pump speed differential before a problem could arise. If the reactor is operating on one pump, the loop select logic trips that pump before making the loop selection.

In addition, during the start-up of Dresden Unit 2 it was found that a flow mismatch between the two sets of jet pumps caused by a difference in recirculation loops could set up a vibration until a mismatch in speed of 27% occurred. The 10% and 15% speed mismatch restrictions provide additional margin before a pump vibration problem will occur.

ECCS performance during reactor operation with one recirculation loop out of service has not been analyzed. Therefore, sustained reactor operation under such conditions is not permitted.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-254

QUAD CITIES UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 23
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Commonwealth Edison Company (the licensee) dated October 29, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 26, 1976

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY
AND
IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

DOCKET NO. 50-265

QUAD CITIES UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 22
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated October 29, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance, (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 3.E of the license is deleted in its entirety.
3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 26, 1976

ATTACHMENT TO AMENDMENT NOS. 23 AND 22

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET NOS. 50-254 AND 50-265

Replace the existing pages 121 and 142 of the Technical Specifications contained in Appendix A with the attached revised pages bearing the same numbers. Changes on these revised pages are shown by marginal lines.

3.6 LIMITING CONDITION FOR OPERATION

II. Recirculation Pump Flow Mismatch

1. Whenever both recirculation pumps are in operation, pump speeds shall be maintained within 10% of each other when power level is greater than 80% and within 15% of each other when power level is less than 80%.
2. If specification 3.6.II.1 cannot be met, one recirculation pump shall be tripped.
3. The reactor shall not be operated with one recirculation loop out of service for more than 24 hours. With the reactor operating, if one recirculation loop is out of service the plant shall be placed in a hot shutdown condition within 24 hours unless the loop is sooner returned to service.

I. Hydraulic Snubbers

1. During all modes of operation except Cold Shutdown and Refuel, all hydraulic snubbers listed in Table 3.6-1 shall be operable except as noted in 3.6.I.2 through 3.6.I.4 below.
2. From and after the time that a hydraulic snubber is determined to be inoperable, continued reactor operation is permissible only during the succeeding 72 hours unless the snubber is sooner made operable.

4.6 SURVEILLANCE REQUIREMENT

3. The baseline data required to evaluate the conditions in Specifications 4.6.G.1 and 4.6.G.2 will be acquired each operating cycle.

H. Recirculation Pump Flow Mismatch

Recirculation pumps speed shall be checked daily for mismatch.

I. Hydraulic Snubbers

The following surveillance requirements apply to all hydraulic snubbers listed in Table 3.6-1.

1. All hydraulic snubbers whose seal material has been demonstrated by operating experience, lab testing or analysis to be compatible with the operating environment shall be visually inspected to verify their operability in accordance with the following schedule:

without an initial nozzle riser system failure.

- II. The LPCI loop selection logic is described in the SAR, Section 6.2.4.2.5. For some limited low probability accidents with the recirculation loop operating with large speed differences, it is possible for the logic to select the wrong loop for injection. For these limited conditions the core spray itself is adequate to prevent fuel temperatures from exceeding allowable limits. However, to limit the probability even further, a procedural limitation has been placed on the allowable variation in speed between the recirculation pumps.

The licensee's analyses indicate that above 80% power the loop select logic could not be expected to function at a speed differential of 15%. Below 80% power the loop select logic would not be expected to function at a speed differential of 20%. This specification provides a margin of 5% in pump speed differential before a problem could arise. If the reactor is operating on one pump, the loop select logic trips that pump before making the loop selection.

ECCS performance during reactor operation with one recirculation loop out of service has not been analyzed. Therefore, sustained reactor operation under such conditions is not permitted.

I. Hydraulic Snubbers

Snubbers are designed to prevent unrestrained pipe motion under dynamic loads as might occur during an earthquake or severe transient, while allowing normal thermal motion during startup and shutdown. The consequence of an inoperable snubber is an increase in the probability of structural damage to piping as a result of a seismic or other event initiating dynamic loads. It is therefore

required that all hydraulic snubbers required to protect the primary coolant system or any other safety system or component be operable during reactor operation.

Because the snubber protection is required only during relatively low probability events, a period of 72 hours is allowed for repairs or replacements. In case a shutdown is required, the allowance of 36 hours to reach a cold shutdown condition will permit an orderly shutdown consistent with standard operating procedures. Since plant startup should not commence with knowingly defective safety related equipment, Specification 3.6.1.4 prohibits startup with inoperable snubbers.

All safety related hydraulic snubbers are visually inspected for overall integrity and operability. The inspection will include verification of proper orientation, adequate hydraulic fluid level and proper attachment of snubber to piping and structures.

The inspection frequency is based upon maintaining a constant level of snubber protection. Thus the required inspection interval varies inversely with the observed snubber failures. The number of inoperable snubbers found during a required inspection determines the time interval for the next required inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NOS. 15, 13, 23 AND 22 TO

FACILITY LICENSE NOS. DPR-19, DPR-25, DPR-29 AND DPR-30

COMMONWEALTH EDISON COMPANY

DRESDEN NUCLEAR POWER STATION UNITS 2 AND 3

QUAD CITIES NUCLEAR POWER STATION UNITS 1 AND 2

DOCKET NOS. 50-237, 50-249, 50-254 AND 50-265

INTRODUCTION

By letter dated October 29, 1975, Commonwealth Edison Company requested an amendment to Facility License Nos. DPR-25 and DPR-30 for the Dresden Nuclear Power Station Unit 3 and the Quad Cities Nuclear Power Station Unit 2. The request involves restrictions on operation with a single recirculation loop. This report also evaluates an identical amendment for Dresden Unit 2 and Quad Cities Unit 1.

DISCUSSION

By Amendment Nos. 5 and 12 to DPR-25 and DPR-30 issued August 29, 1975 and April 21, 1975, respectively, a restriction was placed in the license of each reactor stating that "the reactor shall not be operated with one recirculation loop out of service." As explained in the Safety Evaluation related to Amendment Nos. 5 and 12, an evaluation had not been provided for ECCS performance during reactor operation with one (and only one) recirculation loop out of service. Therefore, we stated that reactor operation under such conditions should not be authorized until the necessary analyses have been performed, evaluated, and determined to be acceptable, and included this restriction in the operating license.

As explained in the licensee's letter of October 29, 1975, this restriction, as presently worded, does not clearly state what action should be taken in the event that one recirculation pump should trip or otherwise become unavailable while the reactor is in operation. The restriction also eliminates the opportunity for recovery from the loss of a single recirculation pump while the reactor is operating.

EVALUATION

In reviewing the licensee's request for change, we agree that certain clarifications and modifications of this restriction are necessary and desirable.

As stated previously, the restriction was inserted into the operating license because an evaluation of ECCS performance during reactor operation with one recirculation loop out of service has not been provided. The intent of the existing restriction on single loop operation was (and is), therefore, to prohibit sustained operation in this mode, pending receipt of further analyses in support of such operation. It was never our intent to prohibit single loop operation for brief periods for good reason, such as for necessary testing or for recovery from a single pump trip. The requested change provides this clarification. This type of time allowance is similar to Technical Specifications both for this facility and others which allow operation for a limited period of time with systems inoperable to allow time for effecting repairs. Such a 24-hour period allows a reasonable length of time to restore an inoperable recirculation loop to service, and avoids undesirable and unnecessary plant transients, such as a manual scram or rapid plant shutdown which otherwise would be required. Additionally, the proposed amendment is desirable because it would make clear the action that is required in the event a recirculation pump trips during operation.

We have reviewed the single loop mode of operation from a thermal-hydraulic safety standpoint, and conclude that in such a mode of operation the reactors will be operated within the bounds of the thermal analysis, will be protected from fuel damage limits resulting from anticipated transients, and is acceptable.

The Dresden Unit 2 and Quad Cities Unit 1 licenses do not restrict operation with a recirculation loop out of service. However, ECCS performance during operation with one loop has not been evaluated. Therefore, the restriction should also apply to Dresden 2 and Quad Cities 1. We have discussed placing a restriction in the Technical Specifications for these facilities which is identical to that in Dresden Unit 3 and Quad Cities Unit 2 with the licensee. They are in agreement with the change.

ENVIRONMENTAL CONSIDERATION

We have determined that these amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the changes do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the changes do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

DATE: February 26, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-237, 50-249, 50-254 AND 50-265

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS AND ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 15, 13, 23 and 22 to Facility Operating License Nos. DPR-19, DPR-25, DPR-29 and DPR-30 (respectively), issued to the Commonwealth Edison Company (and in the matter of License Nos. DPR-29 and DPR-30, the Iowa-Illinois Gas and Electric Company), which revised Technical Specifications for operation of Dresden Nuclear Power Station Units 2 and 3 (located in Grundy County, Illinois) and Units 1 and 2 of the Quad Cities Nuclear Power Station (located in Rock Island County, Illinois). These amendments are effective as of their date of issuance.

The amendments limit reactor operation for each of the facilities to 24 hours with one recirculation loop out of service.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of the amendments.

For further details with respect to this action, see (1) the application for amendments dated October 29, 1975, (2) Amendment Nos. 15 and 13 to License Nos. DPR-19 and DPR-25, and Amendment Nos. 23 and 22 to License Nos. DPR-29 and DPR-30 and (3) the Commission's concurrently issued related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N. W., Washington, D. C. and for those items relating to Dresden Units 2 and 3 at the Morris Public Library, 604 Liberty Street, Morris, Illinois 60450, and for those items relating to Quad Cities Units 1 and 2 at the Moline Public Library, 504 - 17th Street, Moline, Illinois 60625.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 26th day of February, 1976.

FOR THE NUCLEAR REGULATORY COMMISSION

Dennis L. Ziemann
Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors