

ATTACHMENT 1

Fourth Ten-Year Snubber Testing Interval

Relief Request SP-1

Defined Test Plan Grouping (DTPG)

Of Hydraulic Shock Suppressors

**RELIEF REQUEST SP-1**  
**DEFINED TEST PLAN GROUPING (DTPG)**  
**OF HYDRAULIC SHOCK SUPPRESSORS**

Pursuant to 10 CFR 50.55a(f)(6)(i), PBNP requests an alternative to the ASME Code for Operation and Maintenance of Nuclear Power Plants, Subsection ISTD 7.5.3 requirements for defined test plan grouping of hydraulic snubbers as specified in the 1995 Edition, with 1996 Addenda.

Identification

Point Beach Nuclear Plant Units 1 and 2

Code Requirement:

OM Code for Operation and Maintenance of Nuclear Power Plants, 1995 Edition with 1996 Addenda ISTD 7.5 Defined Test Group Plan (DTPG); ISTD 7.5.3 Additional DTPG Requirements for Pressurized Water Reactors. "Snubbers attached to the steam generator and snubbers attached to the reactor coolant pump shall be at least one, separate DTPG."

Relief Requested:

Relief is requested from the requirement to make the snubbers attached to the steam generators one, separate DTPG. It requested that the snubbers attached to the main steam (MS) line and the snubbers attached to the steam generator be made one DTPG for each unit.

Basis for Relief:

PBNP is a two-loop Westinghouse pressurized water reactor with two steam generators per unit. There are 10 Anker-Holth snubbers for each unit. These are installed as follows:

- Three (3) 12" bore snubbers attached to each steam generator (SG) – six (6) snubbers total per unit
- Two (2) 16" bore snubbers attached to each main steam line just downstream from each steam generator nozzle – four (4) snubbers total per unit

All Anker-Holth snubbers are installed to perform the same design basis accident function. The 12" and 16" Anker-Holth snubbers have similar nominal ratings and have identical control blocks. It is a labor intensive activity to remove and functionally test these large bore snubbers. A seal study has established a common rebuild schedule for both snubbers, based on analysis of seals from a 16" and 12" bore snubber. This provides additional basis for treating this snubber group as one DTPG for functional testing.

Alternative Documentation and Requirements:

The DTPG's for Unit 1 and Unit 2 will be documented in the Fourth Interval Snubber Testing Program.

Justification of Relief:

The snubbers on the main steam line and the steam generator are part of the steam generator support system and are designed to resist forces generated by a postulated main steam line break (MSLB). Three Anker-Holth snubbers are installed per steam generator in each of Units 1 and 2. In addition to these snubbers, two Anker-Holth snubbers are installed on each main steam line to resist MSLB loads. The snubber for each steam generator are installed adjacent to each other in the same plane; thus, each snubber is required to resist one-third the MSLB load. The main steam line snubbers are installed in the same plane, 90 degrees apart, thus, each snubber must resist the full MSLB load.

The snubbers attached to the main steam line are in close proximity to the steam generator and serve the same function as the snubbers attached to the steam generators. The snubbers are Anker-Holth and have identical control blocks. Due to these similarities they should be considered one separate DTPG. Periodic testing of snubbers in accordance with the ISTD will be a representative of the sample of the steam generator support system. Inservice degradation will be identified with the proposed testing DTPG. The proposed DTPG provides an acceptable level of quality and safety in accordance with 10 CFR 50.55a(a)(i).

Implementation Schedule:

This relief request will be applicable to the Fourth Inservice Test Interval and will be implemented with the Fourth Interval Snubber Testing Program. The fourth interval snubber testing program is 120 months and begins on September 1, 2002, for Unit 1 and on October 1, 2002, for Unit 2.

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Point Beach Nuclear Plant  
Class 1, 2, and 3  
Inservice Inspection Program

Point Beach Nuclear Plant  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Commercial Service Dates  
Unit 1: December 21, 1970  
Unit 2: October 1, 1972

USNRC Docket Numbers  
Unit 1: 50-266  
Unit 2: 50-301

Effective Date: July 1, 2002

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Record of Revision

Revision No.	Date	Reason for Revision
0	3/11/2002	Original Issue

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Abbreviations

Listed below are the abbreviations utilized in this document:

89	1989 Edition of ASME Section XI with no Addenda
92	1992 Edition of ASME Section XI with no Addenda
95A95	1995 Edition of ASME Section XI with Addenda through 1995
98A00	1998 Edition of ASME Section XI with Addenda through 2000
ANII	Authorized Nuclear Inservice Inspector
AOV	Air Operated Valve
ASME	American Society of Mechanical Engineers
B&PV	Boiler and Pressure Vessel Code
CBD	Code Boundary Drawing (see P&ID)
CCW	Component Cooling Water
CRDM	Control Rod Drive Mechanism
CH	Charging System
Code, the	Refers to ASME Section XI
CV	Control Valve
CVC	Chemical Volume Control System
ECCS	Emergency Core Cooling System
ECT	Eddy Current Testing (Examination)
FW	Feedwater System
HPSI	High Pressure Safety Injection
HS	High Stress
ITE	Inspection and Test Engineering
LD	Letdown System
LPSI	Low Pressure Safety Injection
L or LS	Longitudinal Seam Weld
MOV	Motor Operated Valve
MSIV	Main Steam Isolation Valve
MS	Main Steam System
MT	Magnetic Particle Testing (Examination)
N/A	Not Applicable
NDE	Nondestructive Examination
NMC	Nuclear Management Company, LLC
NPS	Nominal Pipe Size
P&ID	Piping and Instrumentation Diagram
PBNP	Point Beach Nuclear Plant
PT	Liquid Penetrant Testing (Examination)
QC	Quality Control
PWR	Pressurized Water Reactor
PZR	Pressurizer
RCS	Reactor Coolant System
RCP	Reactor Coolant Pump
RWST	Reactor Water Storage Tank
RHE	Regenerative Heat Exchanger

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RPV	Reactor Pressure Vessel
SF	Spent Fuel System
SG	Steam Generator
SI	Safety Injection System
T	Thickness of Component, Pipe, etc.
FSAR	Final Safety Analysis Report
USNRC	United States Nuclear Regulatory Commission
WEPCo	Wisconsin Electric Power Company
UT	Ultrasonic Testing (Examination)
VT	Visual Testing (Examination)

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Abstract

This document describes the Class 1, 2, and 3 Inservice Inspection (ISI) Program for Point Beach Nuclear Plant (PBNP), Units 1 and 2, which are owned by Wisconsin Electric Power Company. The Nuclear Management Company, LLC, is the Licensee and Administrator of the plant.

This document addresses the requirements for the fourth interval Inservice Inspection Program for both PBNP Units, which begins on July 1, 2002, and will end on June 30, 2012. These dates were approved by the United States Nuclear Regulatory Commission (USNRC) through a relief request on June 18, 2001, and will be effective for the remainder of the plant life.

This program meets the requirements of the American Society of Mechanical Engineers (ASME), Boiler and Pressure Vessel Code, Section XI, 1998 Edition with Addenda through 2000, and 10 CFR 50.55a (proposed rule of August 3, 2001) for Class 1, 2, and 3 systems. This Code Edition and Addenda was approved for use at PBNP by the USNRC through a relief request on November 6, 2001, and will be effective for the fourth 10-year interval. The Nuclear Regulatory Commission (NRC) placed limitations on the use of this Code Edition and Addenda. PBNP will use the provisions and limitations in the proposed rule for 10 CFR 50.55a published in the Federal Register (fr03au01-14) on August 3, 2001. When the final rule is issued, PBNP will be required to evaluate what additional actions are necessary. It is subject to the limitations and modifications of 10 CFR 50.55a(b)(2), except the design and access provisions and preservice examination requirements.

Where applicable, ASME Code Cases were incorporated. The Code Cases used are either approved through publication in 10 CFR 50.55a, NRC Regulatory Guide 1.147, or are included in a relief request.

Other alternatives to the Code requirements have been included as relief requests, or they reference specific NRC regulations. Areas where Code compliance is not possible are also included as relief requests, along with proposed alternatives.

This document implements a Risk Informed selection criterion for Examination Category B-F, B-J, C-F-1, and C-F-2 piping welds. This alternative to the requirements of Section XI will be submitted to the NRC under separate cover.

The ISI Programs for Containment and Snubber Examinations are covered under separate documents. General requirements for these programs are included for completeness.

Additional requirements for augmented examinations are addressed. The ISI Program does not require these examinations, but rather are included and administered at the request of the plant. The ISI Coordinator performs the planning and the tracking of each item.

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1.0 Introduction

PBNP Units 1 and 2 are two-loop Westinghouse Pressurized Water Nuclear Power Plants. The Wisconsin Electric Company of Milwaukee, Wisconsin, owns the plants. The Nuclear Management Company (NMC) LLC is the Licensee for the plant and has the operating authority. For purposes of the ISI Program, the NMC is considered the Owner in accordance with the definition in IWA-9000.

1.1 ISI Program Development

This document details the Inservice Inspection Program of Class 1, 2, and 3 components for PBNP, Units 1 and 2.

On April 6, 2001, PBNP requested relief from using the 1995 Edition of ASME Section XI with Addenda through 1996. This exemption was granted in an SER dated November 6, 2001. This authorized PBNP to update the ISI Program to the 1998 Edition of Section XI with Addenda through 2000. The authorization required PBNP to follow the proposed rule issued on August 3, 2001. When the final rule is issued, PBNP will review the rule and modify the ISI Program as appropriate.

The plan and schedules for the Units are located in the Point Beach Nuclear Plant Units 1 and 2, Class 1, 2, and 3 Inservice Inspection Program Schedules. These schedules are separate controlled documents.

1.2 Other ISI Programs

This document does not address every aspect of Inservice Inspection. The following details the examination and testing requirements of those parts covered by other documents. The QA Record of this document can be found in EDMS under Licensing Documents Records.

The program for Inservice Testing of Class 1, 2, and 3 Pumps and Valves is covered by the Point Beach Inservice Testing (IST) Program, which is submitted and approved separately.

The Steam Generator (SG) Eddy Current Testing (ET) Program is controlled by Technical Specifications and is administered separately.

The Snubber Testing Program controls the functional testing of snubbers. This document is submitted under separate cover.

The Containment Examination Program controls the examination of Containment Components under Subsections IWE and IWL. On September 9, 1996, 10CFR50.55a was amended to include sections IWE and IWL of the 1992 Edition with Addenda through 1992 of Section XI. This amendment requires PBNP to implement and complete the initial period examination requirements by September 9, 2001.

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1.3 Construction Permit

The Construction permit for PBNP 1 was CPPR-32 and was issued July 19, 1966.

The Construction permit for PBNP 2 was CPPR-47 and was issued July 25, 1967. Wisconsin Electric Power Company (WEPCo) is the Owner of the plant.

1.4 Commercial Service Date

The commercial service date for PBNP 1 is December 21, 1970. The license will expire on October 5, 2010.

The commercial service date for PBNP 2 is October 1, 1972. The license will expire on March 8, 2013.

1.5 Background

PBNP has been examined to the requirements of many different editions and addenda of Section XI. The applicable Edition of Section XI was mandated by changes to 10 CFR 50.55a at 10-year intervals or earlier.

Appendix A contains a listing of refuel outage dates. This list shows how refuel outages correspond to ISI Intervals, Periods, and Outages.

Preservice Examinations

Preservice examinations were performed to the October 1968 Draft ASME Code for Inservice Inspection of Nuclear Reactor Coolant Systems (N-45).

First Inservice Inspection Interval

The first interval for PBNP Unit 1 began on December 21, 1970, and ended on December 20, 1980; and for PBNP Unit 2, began on October 1, 1972, and ended on September 30, 1982. The first and second Inservice Inspection periods were conducted in accordance with the 1971 Edition with Addenda through Summer 1971 Addenda of the ASME Boiler and Pressure Vessel (B&PV) Code, Section XI. The third period was conducted in accordance with the 1974 Edition with Addenda through Summer 1975 of ASME B&PV Code, Section XI. These requirements were called out in the plant Technical Specifications at that time.

During the first interval, first two periods, PBNP did not have an official ISI Program. Plans and schedules were performed following the Codes in affect at that time, and in accordance with Technical Specifications, which contained the ISI examination schedule.

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Second Inservice Inspection Interval

The second 10-year Inservice Inspection Interval for both Units was conducted in accordance with the 1977 Edition with Addenda through Summer 1979 of Section XI. The second interval for PBNP Unit 1 began on December 21, 1980, and ended on December 20, 1990. PBNP Unit 2 began on October 1, 1982, and ended on September 30, 1992.

Third Inservice Inspection Interval

The Third Inservice Inspection Interval was conducted in accordance with the 1986 Edition of Section XI, no Addenda. The interval ran from December 1, 1990, through June 30, 2002, for PBNP Unit 1. For PBNP Unit 2, the interval ran from December 1, 1992, through June 30, 2002. PBNP submitted a relief request to change the end of the third interval of both Units to June 30, 2002. This was authorized in an SER dated June 18, 2001 (Reference 14.36).

Fourth Inservice Inspection Interval

The fourth interval will be conducted in accordance with the 1998 Edition of ASME Section XI with Addenda through 2000 as modified by 10 CFR 50.55a. This edition and addenda was approved for use by the NRC in a Safety Evaluation Report dated November 16, 2001.

The fourth interval has been changed to allow both Units to have the same start of July 1, 2002, and an end date of June 30, 2012. This was authorized by the USNRC in a Safety Evaluation Report dated June 18, 2001 (Reference 14.37).

The dates of the fourth interval and periods for both Units are scheduled as follows:

	<b>Begin</b>	<b>End</b>
4th Interval	7/1/2002	6/30/2012
1st Period	7/1/2002	6/30/2005
2nd Period	7/1/2005	6/30/2009
3rd Period	7/1/2009	6/30/2012

**Table 1 - Fourth Interval Schedule**

1.6 Applicable Editions and Addenda to Section XI

In accordance with the relief to 10 CFR 50.55a(b)(2), the Inservice Inspection Requirements for the fourth Inservice Inspection interval applicable to Class 1, 2, and 3 components at PBNP are based on the 1998 Edition with Addenda through 2000 of ASME Section XI. This was approved by the USNRC in a Safety Evaluation Report dated November 6, 2001. PBNP agreed to follow the requirements of the proposed rule for 10 CFR 50.55a dated August 3, 2001 (Reference 14.37).

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Portions of the ISI Program are based on other editions and addenda of Section XI, relief requests, the Code of Federal Regulations, Regulatory Guides, and plant technical specifications and commitments. Where this has occurred, it is documented within this Program.

## 1.7 System Classification

At the beginning of the first interval, the Code boundaries were established on the best information available. It was not until sometime during the first interval that the Code Boundaries were defined to their current state. This has resulted in the examination of some components that would not be performed for subsequent intervals under the current rules. See the ISI Program Basis Document (Reference 4.31) for details on how the current boundaries are established.

### 1.7.1 System Classifications

Class 1 system boundaries are developed based on 10 CFR 50.2(v), and the Point Beach Plant FSAR. Class 2 and 3 system boundaries are developed based on Regulatory Guide 1.26 and the Point Beach Plant FSAR. Vessels, piping, pumps, and valves were built and installed according to the rules of ASME Sections III and VIII, and USA Standard B31.1 for those systems classified as Class 1, 2, and 3.

The Code required boundaries for all Class 1, 2, and 3 systems are identified on Classification Boundary Drawings (CBD). For a listing of applicable CBD's, see the ISI Program Basis Document.

### 1.7.2 Optional Construction

Optional construction of a component within a system boundary to a classification higher than the minimum Class established in the component design specification does not affect the overall system classification by which the applicable rules of Section XI are determined.

### 1.7.3 Containment Penetrations

Portions of piping penetrating the containment vessel which are required to be constructed to Class 1 or 2 rules for piping and which may differ from the classification of the balance of the piping system, may not affect the overall system classification that determines the applicable rules of Section XI.

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1.8 Inspection Program

Examinations for Class 1, 2, and 3 components are scheduled in accordance with Inservice Inspection Program B (Subarticle IWB-2412 and Table IWB-2412-1). Examinations of components are scheduled based upon previous 10-year intervals to the extent practical and within the limits of Code Case N-624 (Reference Relief Request No. 9). Piping weld examinations are based on the Risk Informed selection criteria (Reference Relief Request No. 8).

In order to be consistent in determining percentages, PBNP chose to multiply the number of examination areas by the minimum or maximum percentages of Program B and rounding to the nearest whole number. Where fractions the would result in a non-conservative number, the number was rounded in the conservative direction. This will be the number PBNP uses for determining how many welds/components will be examined each period.

The Inservice Inspection Plan Tables for Inservice examinations of the major components of PBNP are located in the ISI Plan and Schedule. These tables have brief descriptions of each component subject to examination. They include all required Code references and any other pertinent information that is useful for determining examination requirements.

A complete description of the format of the examination tables can be found in the Unit 1 and 2 Class 1, 2, and 3 ISI Program Schedule.

1.9 Regulatory Guides

The Regulatory Guides determined to be applicable to PBNP for purposes of this ISI Program are listed below:

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USNRC Regulatory Guides	
R.G. Number	Description of Regulatory Guide
1.26	Quality Group Classifications
1.65	Materials and Inspections for Reactor Vessel Closure Studs
1.147	Section XI Code Case Acceptability
1.150 Rev. 1	UT of RPV Welds During PSI and ISI (for the Head to Flange and Flange to Shell Welds Only)

**Table 2 - USNRC Regulatory Guides**

1.10 ASME Section XI Code Cases

Section XI Code Cases applicable to the ISI Program are shown below. Each Code Case has been approved and listed in USNRC Regulatory Guide 1.147, listed in 10 CFR 50.55a as acceptable for use, or is the subject of a relief request. Other Code Cases used that are listed in the Regulatory Guide, but not included in this listing, will be included in the next revision.

Applicable Code Cases	
Number	Description
N-460	Alternate Examination Coverage for Class 1 and Class 2 Welds, Section XI, Division 1
N-498-1	Alternative Rules for 10-Year Hydrostatic Testing for Class 1, 2, and 3 Systems, Section XI, Division 1 (Used for Class 3 Only)
N-513	Evaluation Criteria for Temporary Acceptance of Flaws in Class 3 Piping, Revision 0 (See Notes 1 and 3)
N-522	Pressure Testing of Containment Penetration Piping, Section XI, Division 1
N-523-1	Mechanical Clamping Devices for Class 2 and 3 Piping (See Notes 2 and 3)
N-532-1	Alternative Requirements to Repair and Replacement Documentation Requirements and Inservice Summary Report Preparation and Submission as Required by IWA-4000 and IWA-6000, Section XI, Division 1 (See Relief Request No. 4)
N-533-1	Alternative Requirements for VT-2 Visual Examination of Class 1 Insulated Pressure-Retaining Bolted Connection, Section XI, Division 1 (See Relief Request No. 5)
N-566-1	Corrective Action for Leakage Identified at Bolted Connections, Section XI, Division 1 (See Relief Request No. 6)
N-616	Alternative Requirements for VT-2 Visual Examination of Classes 1, 2, and 3 Insulated Pressure Retaining Bolted Connections, Section XI, Division 1 (See Relief Request No. 7)
N-624	Use of Code Case N-624, Successive Inspections (See Relief Request No. 8)

**Table 3 - Applicable Code Cases**

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- Notes: 1. When implementing Code Case N-513, the specific safety factors in paragraph 4.0 must be satisfied. Code Case N-513 may not be applied to:
- (a) Components other than pipe and tube, such as pumps, valves, expansion joints, and heat exchangers;
  - (b) leakage through a flange gasket;
  - (c) threaded connections employing nonstructural seal welds for leakage prevention (through seal weld leakage is not a structural flaw, thread integrity must be maintained); and
  - (d) degraded socket welds.
2. When choosing to apply Code Case N-523-1, all of its provisions shall be applied.
3. NRC acceptance of Code Cases N-513 and N-523-1 are found in 10 CFR 50.55a(b)(2)(xiii).

1.11 Branch Technical Position MEB 3-1

Branch Technical Position MEB 3-1 required PBNP to determine where piping failure in fluid systems outside of the containment would have an adverse effect on other components and equipment. PBNP completed a detailed engineering analysis of the Residual Heat Removal, Chemical Volume Control, Main Steam, Main Feedwater, Auxiliary Feedwater, and Steam Generator Blowdown systems.

The engineering analysis determined that portions of the Main Steam and Feedwater systems needed to be included in the ISI examination schedule. These included those supports and welds to the first support beyond the Class 2 boundary. Appropriate welds and supports will be examined.

1.12 Replacement Steam Generators

PBNP 1 replaced the steam generators during the 1984 refueling outage. The replacements were baseline examined and scheduled for Inservice examination as per the applicable categories.

PBNP 2 replaced the steam generators during the 1997 refueling outage. The replacements were baseline examined, and scheduled for Inservice examination as per the applicable categories.

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1.13 Successive Examinations

The sequence of component examinations will be modified in accordance with the requirements of ASME Code Case N-624 (Reference Relief Request No. 8). This allows PBNP to alter the sequence of examinations to allow the examination of several components in an area during one outage instead of over several outages. This will reduce costs and radiation exposure. The percentage requirements of IWB-2412, IWC-2412, IWD-2412, and IWF-2410 (Program B) will be satisfied. Due to the implementation of the Risk Informed ISI Program, the scheduling of piping weld examinations will not follow Code requirements, as the selection criteria are different. However, the percentage requirements of IWB-2412 and IWC-2412 for those components selected will be maintained. The modified examination schedule is designed to optimize the performance of work within the plant to reduce radiation dose, eliminate interference with other work, and reduce costs.

1.14 Plant Life Extension

The ISI Program will implement a PBNP commitment to manage the effects of aging for systems/structures/components within the scope of license renewal. As this commitment is finalized, it will be incorporated into the ISI Program.

1.15 Alternative Examinations

The provisions of IWA-2240, which allows the substitution of alternative examination methods, a combination of methods, or newly developed techniques for the methods specified in the Construction Code may not be applied (Reference 10 CFR 50.55a(b)(2)(xix)).

2.0 Risk Informed (RI) ISI Requirements

PBNP is implementing a Risk Informed Inservice Inspection (RI ISI) Program for Class 1 and 2 piping welds. This will begin with the start of the fourth Inservice Inspection interval. The guidelines of ASME Code Case N-578, Risk Informed Requirements for Class 1, 2, and 3 Piping, Method B, will be used. For PBNP, only Class 1 and 2 piping systems will be effected (Reference Relief Request No. 3).

2.1 Exemption Criteria

The exemption criteria for Class 1 and 2 piping welds is as specified in the Code.

2.2 Probabilistic Risk Analysis

The RI ISI Program depends on the Probabilistic Risk Analysis (PRA) model for determination of the areas where risk is most significant. When the PRA model is changed, this change will be reviewed and incorporated into the RI ISI Program as appropriate.

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2.3 Degradation Mechanisms

The RI ISI Program depends on the degradation mechanism for determination of the areas where piping is most susceptible to damage. When a degradation mechanism change is identified, this change will be reviewed and incorporated into the RI ISI Program as appropriate.

2.4 Component Examination Basis

Piping welds are subjected to a thorough analysis to determine their risk significance. Each weld is then placed into a matrix to determine its risk category based on combinations of consequence and failure potential. In those areas where the risk is high, 25% of the welds are selected for examination. For medium risk areas, 10% of the welds are selected. For low risk areas, no welds were selected. The pressure test requirements of examination categories B-P and C-H are still applicable.

Examination areas for each degradation mechanism are defined by the 1989 Edition of Section XI.

Refer to the Units 1 and 2 Class 1, 2, and 3 ISI Program Schedules for a complete listing of welds subject to examination and the proposed examination schedule.

Examination Category R-A, Risk Informed Piping Systems,

R1.10 More-Safety-Significant Piping Structural Elements

R1.11 Elements Subject to Thermal Fatigue

Volumetric examination to include the circumferential and those portions of the longitudinal welds within the examination boundary of the circumferential weld. The examination area shall be extended to 1/2 inch beyond each side of the base metal thickness transition or counterbore.

R1.12 Elements Subject to High Cycle Mechanical Fatigue

Visual VT-2 examination to be performed during a system pressure test or a pressure test specific to that component/element.

There are no welds at PBNP that are subject to this degradation mechanism.

R1.13 Elements Subject to Corrosive, Erosive, or Cavitation Wastage

Volumetric examination for internal wastage or a surface examination for external wastage.

There are no welds at PBNP that are subject to this degradation mechanism.

R1.14 Elements Subject to Crevice Corrosion Cracking

Volumetric examination of the base metal, weld, heat affected zone, and crevice region

There are no welds at PBNP that are subject to this degradation mechanism.

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- R1.15 Elements Subject to Primary Water Stress Corrosion Cracking (PWSCC)  
Examination is to be performed on the base metal, weld, heat affected zone, and crevice region. Visual VT-2 examination to be performed during a system pressure test or a pressure test specific to that component/element.  
There are no welds at PBNP Unit 1 that are subject to this degradation mechanism.
- R1.16 Elements Subject to Intergranular Stress Corrosion Cracking (IGSCC)  
Volumetric examination of the weld and base material in accordance with Figure IWB-2500-8(c) or IWB-2500-9, 10, or 11 (figures are found in the 1989 Edition of ASME Section XI).
- R1.17 Elements Subject to Microbiologically Influenced Corrosion (MIC)  
Visual VT-3 examination on internal surfaces if accessible or,  
Volumetric examination:  
Carbon or low alloy steel - base metal, weld, and heat affected zone austenitic steel - weld and heat affected zone  
There are no welds at PBNP that are subject to this degradation mechanism.
- R1.18 Elements Subject to Flow Accelerated Corrosion (FAC)  
Examinations will be performed in accordance with the Flow Accelerated Corrosion Program (Secondary ISI) requirements.
- R1.19 Elements Subject to External Chloride Stress Corrosion Cracking (ECSCC)  
Surface examination ½" beyond the toe of the weld. There are no welds at PBNP that are subject to this degradation mechanism.  
The item number was adopted from later versions of the Code Case N-578 as it does not cover components subject to this degradation mechanism.
- R1.20 Elements not Subject to a Damage Mechanism  
Surface or volumetric examination to include the circumferential and those portions of the longitudinal welds within the examination boundary of the circumferential weld. The examination area shall be extended to ½" beyond each side of the base material thickness transition or counterbore. VT-2 examination will be performed in accordance with the applicable class 1 or 2 criteria.  
This item number was adopted from later versions of Code Case N-578 as it does not cover components without a degradation mechanism.

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3.0 Development of the Class 1 Examination Plan

Plant controlled isometric, Code boundary, component drawings, and plant walkdowns were used to develop the ISI drawings and the scope of examinations. During examinations, drawings will be used to locate and identify each component. Other plant controlled drawings or documents will be used when additional information is required.

Refer to the Units 1 and 2 Class 1, 2, and 3 ISI Program Schedules for a complete listing of components subject to examination and the proposed examination schedule.

3.1 Class 1 Code Exemptions

PBNP is required to use the exemption criteria found in IWB-1220 of the 1989 Edition of Section XI. This will be used instead of the exemptions allowed by the 1998 Edition with Addenda through 2000 (Reference 10 CFR 50.55a[b][2][xi]).

IWB-1220, Components Exempt from Examination (1989 Edition of Section XI)

The following components (or parts of components) are exempted from the volumetric and surface examination requirements of IWB-2500:

- (a) Components (Notes 1 and 2) that are connected to the reactor coolant system and part of the reactor coolant pressure boundary (Note 3), and that are of such a size and shape so that upon postulated rupture the resulting flow of coolant from the reactor coolant system under normal plant operating conditions is within the capacity of makeup systems which are operable from on-site emergency power;
- (b) (1) Piping of NPS 1 and smaller, except for steam generator tubing;  
(2) components and their connections in piping (Note 4) of NPS 1 and smaller;
- (c) Reactor vessel head connections and associated piping, NPS 2 and smaller, made inaccessible by control rod drive penetrations.

Note 1: Refer to 10 CFR 50, Section 50.55a(e)(2), revised March 15, 1984.

Note 2: The exemptions from examination in IWC-1220 may be applied to those components permitted to be Class 2 in lieu of Class 1 by the regulatory authority having jurisdiction at the plant site.

Note 3: Reactor Coolant pressure boundary is defined in 10 CFR 50, Section 50.2(v), revised January 1, 1975.

Note 4: *In piping* is defined as having a cumulative inlet and a cumulative outlet pipe cross-sectional area, neither of which exceeds the nominal Outside Diameter (OD) cross-sectional area of the designated size.

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The following Class 1 components are considered exempt from surface and volumetric examination at PBNP:

Component	Exemption Criteria
Piping NPS 1 and smaller, except steam generator tubing	IWB-1220(b)(1)
Excess letdown heat exchanger	IWB-1220(b)(2)
Reactor vessel head connections and associated piping, NPS 2 and smaller made inaccessible by CRD penetrations	IWB-1220(c)

**Table 4 - Class 1 Exempt Components**

3.2 Component Examination Basis

This section describes each Examination Category. The required percentage of examinations and any limitations for each Examination Category is described. All other requirements are found in the 98A00 Edition of Section XI.

Category B-A, Pressure Retaining Welds in the Reactor Pressure Vessel

Augmented examinations have been performed on the Reactor Pressure Vessel. These are discussed in more detail in Section 10.0.

B1.10 - Shell Welds

B1.11 - Circumferential

B1.12 - Longitudinal

The examination of these welds will be deferred to the end of the interval. 100% of the weld length will be examined.

B1.20 - Head Welds

B1.21 - Circumferential

B1.22 - Meridional

There are no circumferential or meridional head welds at PBNP.

B1.30 - Shell-to-Flange Weld

Examine the shell to flange weld from the flange surface during the first period and from the vessel side at or near the end of the interval.

B1.40 - Head-to-Flange Weld

Examine 100% of the weld during the first period.

B1.50 - Repair Welds

B1.51 - Beltline Region

There are no repair welds in the Beltline region of either PBNP Reactor Pressure Vessel.

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Category B-B, Pressure Retaining Welds in Vessels Other than the Reactor Vessels

Pressurizer

B2.10 - Shell-to-Head Welds

B2.11 - Circumferential

B2.12 - Longitudinal

Both circumferential welds and one foot of the longitudinal welds will be examined.

B2.20 - Head Welds

B2.21 - Circumferential

B2.22 - Meridional

There are no circumferential or meridional head welds on either Pressurizer at PBNP.

Steam Generators (Primary Side)

B2.30 - Head Welds

B2.31 - Circumferential

B2.32 - Meridional

There are no circumferential or meridional head welds on the steam generators at PBNP.

B2.40 - Tubesheet-to-Head Weld

Examine the entire weld of one steam generator.

Heat Exchangers (Primary Side) - Head

The Regenerative Heat Exchanger is the only component in the next three Item Numbers.

B2.50 - Head Welds

B2.51 - Circumferential Welds

B2.52 - Meridional

There are no circumferential or meridional head welds on the Regenerative Heat Exchanger.

Heat Exchangers (Primary Side) - Shell

The Regenerative Heat Exchanger is the only component in the next three Item Numbers.

B2.60 - Tubesheet-To-Head Welds

Examine one of three vessels of the Regenerative Heat Exchanger per Unit.

B2.70 - Longitudinal Welds

There are no longitudinal welds on the Regenerative Heat Exchanger at PBNP.

B2.80 - Tubesheet-To-Shell Welds

Examine one of three vessels of the Regenerative Heat Exchanger per Unit.

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Category B-D, Full Penetration Welded Nozzles in Vessels – Program B

Examination category B-D, Item Numbers B3.120 and B3.140, will be examined in accordance with the requirements of the 1998 Edition of Section XI (Reference 10 CFR 50.55a(b)(2)(xxi)(A)).

Reactor Vessel

B3.90 - Nozzle-to-Vessel Welds

B3.100 - Nozzle Inside Radius Section

Examine all nozzles during the interval.

Pressurizer

B3.110 - Nozzle-to-Vessel Welds

B3.120 - Nozzle Inside Radius Section

Examine all nozzles during the interval. The inside radius sections are required to be examined by 10 CFR 50.55a(b)(2)(xxi)(A).

Steam Generators (Primary Side)

B3.130 - Nozzle-To-Vessel Welds

B3.140 - Nozzle Inside Radius Section

Examine all nozzles during the interval. The inside radius sections are required to be examined by 10 CFR 50.55a(b)(2)(xxi)(A).

Heat Exchangers (Primary Side)

The Regenerative Heat Exchanger is the only component in the next two Item Numbers.

B3.150 - Nozzle-to-Vessel Welds

B3.160 - Nozzle Inside Radius Section

Examine all nozzles of one vessel per Unit during the interval in accordance with Relief Request 10.

Category B-F, Pressure Retaining Dissimilar Metal Welds in Vessel Nozzles

These components will be examined under the RI ISI Program (Reference Section 2.0).

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Category B-G-1, Pressure Retaining Bolting, Greater Than 2 in. in Diameter

For heat exchangers, piping, pumps, and valves, examinations are limited to components selected for examination under B-B (vessels other than RPV), B-J (piping), B-L-2 (pump casings), and B-M-2 (valve bodies exceeding NPS 4).

Reactor Pressure Vessel

- B6.10 - Closure Head Nuts
- B6.20 - Closure Studs, in place
- B6.30 - Closure Studs, when removed
- B6.40 - Threads in Flange
- B6.50 - Closure Washers, Bushings

Examine 100% of the bolting each interval.

Pressurizer

- B6.60 - Bolts and Studs
- B6.70 - Flange Surface, when connection disassembled
- B6.80 - Nuts, Bushings, and Washers

There are no Examination Category B-G-1 Pressurizer bolting at PBNP on either Unit.

Steam Generators

- B6.90 - Bolts and Studs
- B6.100 - Flange Surface, when connection disassembled
- B6.110 - Nuts, Bushings, and Washers

There are no Examination Category B-G-1 Steam Generator bolting at PBNP on either Unit.

Heat Exchangers

- B6.120 - Bolts and Studs
- B6.130 - Flange Surface, when connection disassembled
- B6.140 - Nuts, Bushings, and Washers

There are no Examination Category B-G-1 Heat Exchanger bolting at PBNP on either Unit.

Piping

- B6.150 - Bolts and Studs
- B6.160 - Flange Surface, when connection disassembled
- B6.170 - Nuts, Bushings, and Washers

There are no Examination Category B-G-1 piping bolting at PBNP on either Unit.

Pumps

- B6.180 - Bolts and Studs
- B6.190 - Flange Surface, when connection disassembled
- B6.200 - Nuts, Bushings, and Washers

Examine 100% of the above items in one Reactor Coolant Pump.

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Valves

B6.210 - Bolts and Studs

B6.220 - Flange Surface, when connection disassembled

B6.230 - Nuts, Bushings, and Washers

There are no Examination Category B-G-1 valve bolting greater than 2 inches in diameter at PBNP.

Category B-G-2, Pressure Retaining Bolting, 2 in. and Less in Diameter

For heat exchangers, piping, pumps, and valves, examinations are limited to components selected for examination under B-B (vessels other than RPV), B-J (piping), B-L-2 (pump casings), and B-M-2 (valve bodies exceeding NPS 4).

B7.10 - Reactor Vessel Bolts, Studs, and Nuts

There are no Reactor Vessel bolting 2 in. and less in diameter at PBNP.

B7.20 - Pressurizer Bolts, Studs, and Nuts

Examine 100% of the bolting each interval.

B7.30 - Steam Generators Bolts, Studs, and Nuts

Examine 100% of the bolting each interval on the steam generator selected for examination under Examination Category B-B.

B7.40 - Heat Exchangers Bolts, Studs, and Nuts

There are no Heat Exchanger bolting 2 inches in diameter or less at PBNP.

B7.50 - Piping Bolts, Studs, and Nuts

Examine 100% of the bolting each interval.

B7.60 - Pumps Bolts, Studs, and Nuts

There are no pump bolting 2 inches in diameter or less at PBNP.

B7.70 - Valves Bolts, Studs, and Nuts

Examine 100% of the bolting on one of each group of valves each interval.

B7.80 - CRD Housings Bolts, Studs, and Nuts

There are no CRD Housing bolting 2 inches in diameter or less at PBNP.  
Examinations would be required under 10 CFR 50.55a(b)(2)(xxi)(B).

Category B-J, Pressure Retaining Welds in Piping

The Risk Informed Program has replaced the requirements of Examination Category B-J.  
See Code Category R-A in Section 2.0.

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Category B-K, Welded Attachments for Vessels, Piping, Pumps, and Valves

PBNP will examine 10% of Class 1 welded attachments. Each will receive a surface examination of 100% of required areas of each welded attachment. In the case of multiple components within a system of similar design, function, and service, only one of the welded attachments of only one of the multiple components will be examined. The welded attachments selected for examination shall correspond to those component supports selected for examination under IWF-2510. Examination is also required whenever component support member deformation (e.g., broken, bent, or pulled out parts) is identified during operation, refueling, maintenance, examination, Inservice Inspection, or testing. Examinations performed as a result of support deformation cannot be credited under the requirements of Inspection Program B.

B10.10 - Pressure Vessel Welded Attachments

Examine only one welded attachment of only one of multiple vessels. Under 10 CFR 50.55a(b)(xxi)(C), PBNP will utilize the 95A95 Section XI for the examination of Item No. B10.10 components. The difference is found in Note 7 of Examination Category B-K, which stipulates a different examination area.

B10.20 - Piping Welded Attachments

Examine 10% of the welded attachments to piping.

B10.30 - Pump Welded Attachments

Examine 10% of the welded attachments to pumps.

B10.40 - Valve Welded Attachments

There are no welded attachments to valves at PBNP.

Category B-L-1, Pressure Retaining Welds in Pump Casings

B12.10 - Pump Casing Welds

The outside surface of the welds of one reactor coolant pump will be VT-1 examined during the inspection interval per Unit.

Category B-L-2, Pump Casings

B12.20 - Pump Casing

Examine the interior surfaces of one of the Reactor Coolant Pumps if disassembled for maintenance, repair, or volumetric examination.

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Category B-M-1, Pressure Retaining Welds in Valve Bodies

Valves that are of the same size, constructional design, and manufacturing method, and that perform similar functions in the system are grouped together.

B12.30 - Valve Body Welds, < NPS 4

There are no B12.30 valve body welds at Point Beach.

B12.40 - Valve Body Welds, ≥ NPS 4

Examine one valve within each group. Valve Group Number 1 is the only group having body welds.

Category B-M-2, Valve Bodies

Valves that are of the same size, constructional design, and manufacturing method, and that perform similar functions in the system are grouped together.

B12.50 - Valve Body, > NPS 4

Examinations are limited to at least one valve of each group of valves once per interval when disassembled for maintenance, repair, or volumetric examination.

Unit 1 Class 1 Valve Groupings					
Group Number	System	Valve Number	Isometric Number	Type	Size
1	Residual Heat Removal	1RH-701	ISI-PRI-1122	Velan Gate Valve 88904-1	10"
		1RH-720	ISI-PRI-1124		
2	Residual Heat Removal	1RH-700	ISI-PRI-1122	Velan Gate Valve 11633	10"
3	Safety Injection	1SI-842A	ISI-PRI-1125	Darling Check Valve 11542	10"
		1SI-842B	ISI-PRI-1123		
		1SI-867A	ISI-PRI-1125		
		1SI-867B	ISI-PRI-1123		
4	Safety Injection	1SI-853A	ISI-PRI-1129	Velan Check Valve 78074	6"
		1SI-853B	ISI-PRI-1127		
		1SI-853C	ISI-PRI-1129		
		1SI-853D	ISI-PRI-1127		

Table 5 – Unit 1 Class 1 Valve Groupings

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Unit 2 Class 1 Valve Groupings					
Group Number	System	Valve Number	Isometric Number	Type	Size
1	Residual Heat Removal	2RH-701	ISI-PRI-2123	Velan Gate Valve 88904-1	10"
		2RH-720	ISI-PRI-2125		
2	Residual Heat Removal	2RH-700	ISI-PRI-2122	Velan Gate Valve 11633	10"
3	Safety Injection	2SI-842A	ISI-PRI-2126	Darling Check Valve 11542	10"
		2SI-842B	ISI-PRI-2124		
		2SI-867A	ISI-PRI-2126		
		2SI-867B	ISI-PRI-2124		
4	Safety Injection	2SI-853A	ISI-PRI-2131	Velan Check Valve 78074	6"
		2SI-853B	ISI-PRI-2128		
		2SI-853C	ISI-PRI-2131		
		2SI-853D	ISI-PRI-2130		

Table 6 – Unit 2 Class 1 Valve Groupings

Category B-N-1, Interior of Reactor Vessel

B13.10 - Reactor Vessel Interior

Examine accessible areas once each inspection period above and below the reactor core made accessible for examination by removal of components during normal refueling.

Category B-N-2, Integrally Welded Core Support Structures and Interior Attachments to Reactor Vessels

B13.50 - Interior Attachments within Beltline Region

Examine interior attachments within the beltline region once per interval. These examinations may be deferred until the end of the interval.

B13.60 Interior Attachments Beyond Beltline Region

Examine interior attachments beyond the beltline region once per interval. These examinations may be deferred until the end of the interval.

Category B-N-3, Removable Core Support Structures

B13.70 - Core Support Structure

Examine accessible surfaces of core support structures once per interval. The structure shall be removed from the vessel. These examinations may be deferred until the end of the interval.

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Category B-O, Pressure Retaining Welds in Control Rod Housings

B14.10 - Reactor Vessel Welds in CRD Housing

Examine 10% of peripheral CRD housings. These examinations may be deferred until the end of the interval.

Category B-P, All Pressure Retaining Components

The system leakage test shall be conducted prior to plant startup following each reactor refueling outage.

On systems borated for the purpose of controlling reactivity, insulation shall be removed from pressure retaining bolted connections for visual examination VT-2 (IWA-5242). The examinations may be performed in accordance with the alternative requirements of Code Case N-533-1 and/or N-616.

More information on pressure testing is included in Section 9.0, Pressure Testing.

B15.10 - Reactor Vessel Pressure Retaining Boundary

B15.20 - Pressurizer Pressure Retaining Boundary

B15.30 - Steam Generator Pressure Retaining Boundary

B15.40 - Heat Exchanger Pressure Retaining Boundary

B15.50 - Piping Pressure Retaining Boundary

B15.60 - Pumps Pressure Retaining Boundary

B15.70 - Valves Pressure Retaining Boundary

Category B-Q, Steam Generator Tubing

B16.10 - Steam Generator Tubing in Straight Tube Design

PBNP has U-Tube design steam generators and this Item Number does not apply.

B16.20 - Steam Generator Tubing in U-Tube Design

The extent and frequency of steam generator tubing surveillance requirements are contained in plant Technical Specification 5.5.8. Procedure NP 7.7.16, Steam Generator Program, contains the requirements for the steam generator examinations (Reference 10 CFR 50.55a[b][2][iii]).

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4.0 Development of the Class 2 Examination Plan

Plant controlled isometric, Code boundary, component drawings, and plant walkdowns were used to develop the ISI drawings and the scope of examinations. During examinations, drawings will be used to locate and identify each component. Other plant controlled drawings or documents will be used when additional information is required.

Refer to the Units 1 and 2 Class 1, 2, and 3 ISI Program Schedules for a complete listing of components subject to examination and the proposed examination schedule.

4.1 Class 2 Code Exemptions

The following Class 2 exemption criteria are followed. Subarticle IWC-1220 of the 98A00 Section XI lists those piping and components exempt from examination.

IWC-1220 Components Exempt from Examination

The following components or parts of components are exempt from the volumetric and surface examination requirements of IWC-2500.

IWC-1221 Components Within RHR, ECC, and CHR Systems or Portions of Systems (Note 1).

- (a) For systems, except high pressure safety injection systems in pressurized water reactor plants:
  - (1) Piping NPS 4 (DN100) and smaller.
  - (2) Vessels, pumps, and valves and their connections in piping (Note 2) NPS 4 (DN100) and smaller.
- (b) For high pressure safety injection systems in pressurized water reactor plants:
  - (1) Piping NPS 1-1/2 (DN40) and smaller.
  - (2) Vessels, pumps, and valves and their connections in piping (Note 2) NPS 1-1/2 (DN40) and smaller.
- (c) Vessels, piping, pumps, valves, other components, and component connections of any size in statically pressurized, passive (i.e., no pumps) safety injection systems (Note 3) of pressurized water reactor plants.
- (d) Piping and other components of any size beyond the last shutoff valve in open-ended portions of systems that do not contain water during normal plant operating conditions.

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IWC-1222 Components Within Systems or Portions of Systems Other Than RHR, ECC, and CHR Systems (Note 1).

- (a) For systems, except auxiliary feedwater systems in pressurized water reactor plants:
  - (1) Piping NPS 4 (DN100) and smaller.
  - (2) Vessels, pumps, and valves and their connections in piping (Note 2) NPS 4 and smaller.
- (b) For auxiliary feedwater systems in pressurized water reactor plants:
  - (1) Piping NPS 1-1/2 (DN40) and smaller.
  - (2) Vessels, pumps, and valves and their connections in piping (Note 2) NPS 1-1/2 and smaller.
- (c) Vessels, piping, pumps, valves, other components, and component connections of any size in systems or portions of systems that operate (when the system function is required) at a pressure equal to or less than 275 psig (1900 kPa) and at a temperature equal to or less than 200° F (93° C).
- (d) Piping and other components of any size beyond the last shutoff valve in open-ended portions of systems that do not contain water during normal plant operating conditions.

IWC-1223 Inaccessible Welds

Welds or portions of welds that are inaccessible due to being encased in concrete, buried underground, located inside a penetration, or encapsulated by guard pipe.

Welds in high energy fluid system containment penetration piping located inside a containment Penetration assembly or encapsulated by a guard pipe are not exempt from the examination provisions of IWC-1223 (Reference 10 CFR 50.55a[b][2][xii][A]).

Note 1: RHR, ECC, and CHR systems are the Residual Heat Removal, Emergency Core Cooling, and Containment Heat Removal Systems, respectively.

Note 2: *In piping* is defined as having a cumulative inlet and a cumulative outlet pipe cross-sectional area neither of which exceeds the nominal OD cross-sectional area of the designated size.

Note 3: Statically pressurized, passive safety injection systems of pressurized water reactor plants are typically called:

- (a) Accumulator tank and associated system.
- (b) Safety injection tank and associated system.
- (c) Core flooding tank and associated system.

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The following Class 2 systems are exempt from surface and volumetric requirements for reasons other than size:

Component/Piping System	Exemption Criteria
Accumulator Tank 2T-34A to valve 2-842A	IWC-1221(c)
Accumulator Tank 2T-34B to valve 2-842B	IWC-1221(c)
Containment Spray from valve 2-868A to the containment penetration and the spray headers inside containment.	IWC-1221(d)
Containment Spray from valve 2-868B to the containment penetration and the spray headers inside containment.	IWC-1221(d)

Table 7 – Class 2 Exempt Components

4.2 Component Examination Basis

This section describes each Examination Category. The number of examinations and any limitations for each Examination Category is described. All other requirements are found in the 98A00 Edition of Section XI.

Category C-A, Pressure Retaining Welds in Pressure Vessels

C1.10 - Shell Circumferential Welds

Examine 100% of welds at gross structural discontinuities. The examination may be limited to one vessel or distributed among the vessels.

C1.20 - Head Circumferential Welds

Examine 100% of head-to-shell welds. The examination may be limited to one vessel or distributed among the vessels.

C1.30 - Tubesheet to Shell Welds

Examine 100% of tubesheet to shell welds. The examination may be limited to one vessel or distributed among the vessels.

Category C-B, Pressure Retaining Nozzle Welds in Vessels

C2.10 - Nozzles in Vessels Less Than or Equal to 1/2 Inch Nominal Thickness

C2.11 - Nozzle to Shell (or Head) Weld

Applies to nozzles greater than 4 NPS. There are no nozzles in vessels less than or equal to 1/2 inch nominal thickness at PBNP.

C2.20 - Nozzles without Reinforcing Plate in Vessels Greater Than 1/2 inch Nominal Thickness.

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C2.21 - Nozzle to Shell or Head Welds

C2.22 - Nozzle Inside Radius Section

Applies to nozzles greater than 12 NPS. Examine nozzles at terminal ends of piping runs. The examination may be limited to one vessel or distributed among the vessels.

C2.30 - Nozzles with Reinforcing Plate in Vessels > 1/2 Inch Nominal Thickness.

C2.31 - Reinforcing Plate Welds to Nozzle and Vessel

Examine nozzles at terminal ends of piping runs. The examination may be limited to one vessel or distributed among the vessels.

C2.32 - Nozzle to Shell (or Head) Welds When Inside of Vessel is Accessible  
There are no Item No. C2.32 components at PBNP.

C2.33 - Nozzle to Shell (or Head) Welds When Inside of Vessel is Inaccessible  
Examine telltale hole in reinforcing plates once each period. The examination may be limited to one vessel or distributed among the vessels. This examination will be performed during the pressure test of the system.

Category C-C, Welded Attachments for Vessels, Piping, Pumps, and Valves

C3.10 - Pressure Vessel Welded Attachments

There are no welded attachments on the pressure vessels at PBNP.

C3.20 - Piping Welded Attachments

Examine 100% of the required areas of welded attachments, limited to attachments of components included within examination Categories C-F-1 and C-F-2. A minimum of 10% of the Welded Attachments will receive a surface examination. In the case of multiple components within a system of similar design, function, and service, only one of the integral attachments of only one of the multiple components will be examined. Examination is also required whenever component support member deformation (e.g., broken, bent, or pulled out parts) is identified during operation, refueling, maintenance, examination, Inservice Inspection, or testing. Examinations performed as a result of support deformation will not be credited under the requirements of Inspection Program B.

C3.30 - Pumps, Welded Attachments

There are no Class 2 Pumps with welded attachments at PBNP.

C3.40 - Valves, Welded Attachments

There are no Class 2 Valves with welded attachments at PBNP.

Category C-D, Pressure Retaining Bolting > 2 Inch in Diameter

C4.10 - Pressure Vessels, Bolts and Studs

C4.20 - Piping, Bolts and Studs

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C4.30 - Pumps, Bolts and Studs

C4.40 - Valve, Bolts and Studs

There is no Class 2 pressure retaining bolting greater than 2 inches at PBNP.

Category C-F-1, Pressure Retaining Welds in Austenitic Stainless Steel or High Alloy Piping

The Risk Informed Program has replaced the requirements of Examination Category C-F-1. See Code Category R-A in Section 2.0.

Category C-F-2,, Pressure Retaining Welds in Carbon or Low Alloy Steel Piping

The Risk Informed Program has replaced the requirements of Examination Category C-F-2. See Code Category R-A in Section 2.0.

Category C-G, Pressure Retaining Welds in Pumps and Valves

C6.10 - Pump Casing Welds

There are no pressure retaining welds in Class 2 pumps at PBNP.

C6.20 - Valve Body Welds

Examine the valve body welds of one valve in each group of multiple valves of similar design, size, function, and service in a system. See Technical Justifications for a listing of valve groups. The examination may be performed from either the outside or inside surface of the component. The extent of examination is limited to those valves in piping runs examined under Examination Category C-F.

Unit 1 Class 2 Valve Groupings					
Group Number	System	Valve Number	Isometric Number	Type	Size
5	Residual Heat Removal	1SI-850A 1SI-850B	ISI-PRI-1222 ISI-PRI-1223	Poppet S70-W-DD	10"

Table 8 – Unit 1 Class 2 Valve Groupings

Unit 2 Class 2 Valve Groupings					
Group Number	System	Valve Number	Isometric Number	Type	Size
5	Residual Heat Removal	2SI-850A 2SI-850B	ISI-PRI-2252 ISI-PRI-2253	Poppet S70-W-DD	10"

Table 9 – Unit 2 Class 2 Valve Groupings

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Category C-H, All Pressure Retaining Components

C7.10 - Pressure Retaining Components

The pressure retaining components within the Class 2 system boundaries are subjected to System Leakage Tests in accordance with IWC-5220 and visually examined per IWA-5240.

System Leakage tests will be performed in accordance with the rules of Section XI as modified by the following:

Code Case N-522, Pressure Testing of Containment Penetration Piping.

Code Case N-533-1, Alternative Requirements for VT-2 Visual Examination of Class 1 Insulated Pressure-Retaining Bolted Connection, Section XI, Division 1.

On systems borated for the purpose of controlling reactivity, insulation shall be removed from pressure retaining bolted connections for visual examination VT-2 (Reference IWA-5240). The examinations may be performed in accordance with the alternative requirements of Code Case N-533-1 at any time during an outage without the system being at nominal operating pressure.

Additional information on pressure testing is included in Section 9.0, Pressure Testing.

5.0 Development of the Class 3 Examination Plan

Plant controlled isometric, Code boundary, component drawings, and plant walkdowns were used to develop the ISI drawings and the scope of examinations. During examinations, drawings will be used to locate and identify each component. Other plant controlled drawings or documents will be used when additional information is required.

Refer to the Units 1 and 2 Class 1, 2, and 3 ISI Program Schedules for a complete listing of components subject to examination and the proposed examination schedule.

The Class 3 system boundary examination is based upon the requirements of Regulatory Guide 1.26, and ASME Section XI, Table IWD-2500-1. See the ISI Program Basis document for additional details.

5.1 Class 3 Code Exemptions

The following Class 3 exemption criteria are followed. Subarticle IWD-1220 of the 98A00 Edition of Section XI lists those piping and components exempt from examination.

IWD-1220 Components Exempt from Examination

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The following components or parts of components are exempted from the VT-1 visual examination requirements of IWD-2500:

- (a) Piping NPS 4 (DN100) and smaller.
- (b) Vessels, pumps, and valves and their connections in piping (Note 1) NPS 4 (DN100) and smaller.
- (c) Components that operate at a pressure of 275 psig (1900kPa) or less and at a temperature of 200° F (93° C) or less in systems (or portions of systems) whose function is not required in support of reactor residual heat removal, containment heat removal, and emergency core cooling.
- (d) Welds or portions of welds that are inaccessible due to being encased in concrete, buried underground, located inside a penetration, or encapsulated by guard pipe.

IWD-5222(g) System Hydrostatic Test Exemption

Open ended vent and drain lines from components extending beyond the last shutoff valve and open ended safety or relief valve discharge lines, including safety or relief valve piping which discharges into the containment pressure suppression pool, shall be exempt from hydrostatic test.

Note 1: *In piping* is defined as having a cumulative inlet and cumulative outlet pipe cross-sectional area neither of which exceeds the nominal OD cross-sectional area of the designated size.

The following Class 3 components have been identified as exempt from examination requirements:

Component/Piping System	Exemption Criteria
Welded attachments of Class 3 supports and restraints to components on 4 inches and smaller piping.	IWD-1220(a)

Table 10 –Class 3 Exempt Components

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5.2 Component Examination Basis

This section describes each Examination Category. The number of examinations and any limitations for each Examination Category is described. All other requirements are found in the 98A00 Edition of Section XI.

Category D-A, Welded Attachments for Vessels, Piping, Pumps, and Valves

Examine 10% of the integral attachments, 100% of required areas of each welded attachment. In the case of multiple components within a system of similar design, function, and service, one of the integral attachments of one of the multiple components shall be examined. The welded attachments selected for examination shall correspond to those component supports selected by IWF-2510. Examination is also required whenever component support member deformation (e.g., broken, bent, or pulled out parts) is identified during operation, refueling, maintenance, examination, Inservice Inspection, or testing. Examinations performed, as a result of support deformation, cannot be credited under the requirements of Inspection Program B.

D1.10 - Pressure Vessels, Welded Attachments

Selected samples of welded attachments shall be examined. The welded attachments of only one of multiple vessels need be examined. A 10% sample shall be examined.

D1.20 - Piping, Welded Attachments

The percentage sample shall be proportional to the total number of nonexempt welded attachments connected to the piping. A 10% sample shall be examined.

D1.30 - Pumps, Welded Attachments

There are no Item No. D1.30 pump welded attachments at PBNP.

D1.40 - Valves, Welded Attachments

There are no Item No. D1.40 valve welded attachments at PBNP.

Category D-B, All Pressure Retaining Components

D2.10 Pressure retaining components

A system leakage test (IWD-5221) shall be performed during each inspection period.

D2.20 Pressure retaining components

System Functional Test shall be performed in accordance with alternate examination techniques of Code Case N-498-1, Alternative Rules for 10 Year Hydrostatic Pressure Testing for Class 1 and 2 Systems Section XI, Division 1.

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6.0 IWE Metal Containment Requirements

The requirements for Code Class MC (Metal Containment) are found in the Point Beach Nuclear Plant Inservice Inspection - IWE Program. This document establishes the administrative, managerial, and implementation control for the IWE Containment Inspection Program Plan for the first 10-year Inservice Inspection interval.

The IWE Program follows the 1992 Edition of Section XI with Addenda through 1992 within the limitations and modifications required by 10 CFR 50.55a.

This program is administered separately from the ISI Program and the requirements are contained in a separate document.

7.0 Component Supports

Plant controlled isometric, Code boundary, component drawings, and plant walkdowns were used to develop the ISI drawings and the scope of examinations. During examinations, drawings will be used to locate and identify each component. Other plant controlled drawings or documents will be used when additional information is required.

Refer to the Units 1 and 2 Class 1, 2, and 3 ISI Program Schedules for a complete listing of components subject to examination and the proposed examination schedule.

The Class 1, 2, and 3 system boundaries are based upon the requirements 10 CFR 50, Regulatory Guide 1.26, and ASME Section XI, Table IWD-2500-1. See the ISI Program Basis document for additional details.

7.1 Code Exemptions for Supports

Under IWF-1230, component supports exempt from the examination requirements of IWF-2000 are those connected to piping and other items exempted from volumetric, surface, or VT-1 or VT-3 visual examination by IWB-1220, IWC-1220, IWD-1220, and IWE-1220. In addition, portions of supports that are inaccessible by being encased in concrete, buried underground, or encapsulated by guard pipe are also exempt from the examination requirements of IWF-2000.

Supports not exempted by IWF-1230 shall be examined in accordance with Table IWF-2500-1.

7.2 Support Examination Basis

This section describes each Examination Category. The number of examinations and any limitations for each Examination Category is described. All other requirements are found in the 98A00 Edition of Section XI.

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To the extent practical, the same supports selected for examination during the first inspection interval were selected for examination during the fourth inspection interval.

Category F-A

F1.10 - Class 1 Piping Supports

Examine 25% of Class 1 piping supports.

F1.20 - Class 2 Piping Supports

Examine 15% of Class 2 piping supports.

F1.30 - Class 3 Piping Supports

Examine 10% of Class 3 piping supports.

The total percentage sample shall be comprised of supports from each system (e.g., Main Steam, Feedwater, or RHR), where the individual sample sizes are proportional to the total number of nonexempt supports of each type and function within each system.

F1.40 - Supports Other Than Piping Supports

Examine the supports of only one of the multiple components within a system of similar design, function, and service.

Item Numbers

Item numbers will be categorized to identify support types by component support function.

A - Single Acting Restraints

B - Double Acting Restraints

C - Spring Hangers and Supports

S - Snubbers

Other Codes may be used as required.

Several supports hold more than one classified line. These supports are counted only once and if scheduled for examination, will cover all of the applicable lines. The support will be counted once for credit.

7.3 Snubbers

Snubbers are functionally tested under the Snubber Program. Visual VT-3 examinations are performed on snubbers as required by Examination Category F-A. If welded attachments are present, then these will be examined under the Class 1, 2, or 3 Examination Category as appropriate. These examinations are tracked within the ISI Program database.

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The requirements for the snubber program are included in NP 7.4.11. The snubber program specifies the ASME OM Code, 1995 Edition up to and including the 1996 Addenda may be used in lieu of the requirements for snubbers in Section XI, IWF-5200(a) and (b), and IWF-5300(a) and (b) (Reference 10 CFR 50.55a[b][3][v]).

#### 8.0 IWL Concrete Containment Requirements

The requirements for Code Class CC (Concrete Containment) are found in the Point Beach Nuclear Plant Inservice Inspection - IWL Program. This document establishes the administrative, managerial, and implementation control for the IWL Containment Inspection Program Plan for the first 10-year Inservice Inspection interval.

The IWL Program follows the 1992 Edition of Section XI with Addenda through 1992 within the limitations and modifications required by 10 CFR 50.55a.

This program is administered separately from the ISI Program and the requirements are contained in a separate document.

#### 9.0 Pressure Tests

Pressure test are performed on a periodic basis to determine the structural integrity of the pressure retaining boundary. Each Class of pipe has specific requirements that shall be followed. The boundaries subject to system leakage and hydrostatic tests are shown on the Classification Boundary Drawings (CBD).

Piping that penetrates the containment that is connected to piping that is outside the scope of Section XI is not exempt from the pressure testing provisions of Subsection IWA as permitted by IWA-5110(c) (Reference 10 CFR 50.55a(b)(2)(xii)(B)).

**System Leakage Tests** - conducted while the system is in operation, during a system operability test, or while the system is at test conditions using an external pressurization source. The hold time requirements for system leakage tests have been changed per 10 CFR 50.55a(b)(2)(xx). This requires PBNP to use the provisions of IWA-5213(a) of the 1995 Edition instead of the those found in the 98A00 Code.

Hold times shall be in accordance with the following:

1995 Edition, IWA-5213(a): No holding time is required after attaining test pressure and temperature conditions when the system has been in operation for at least 4 hours, otherwise, a 10-minute holding time for non-insulated systems or components, or 4 hours for insulated systems or components, is required after attaining system operating pressure.

Class 1 Systems: System leakage tests shall be conducted each refueling outage prior to plant startup following the reactor refueling outage.

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Class 2 Systems: System leakage tests shall be conducted each inspection period.

Class 3 Systems: System leakage tests shall be conducted each inspection period.

**System Hydrostatic Tests** - conducted during a plant shutdown at an elevated test pressure as specified in IWB-5230, IWC-5230, IWD-5222. A holding time of 10 minutes is required for those systems without insulation, and 4 hours for insulated components after attaining the test pressure and temperature.

**Note:** The 98A00 Code requires a system hydrostatic test at or near the end of the inspection interval for Class 3 systems. The alternative requirements of ASME Code Case N-498-1 will be used to perform a system pressure test in lieu of the hydrostatic test.

**System Pneumatic Tests** - A holding time of ten minutes is required after attaining the test pressure and temperature. These tests may be conducted in lieu of the above system pressure tests for Class 2 or Class 3 components as permitted by IWC-5000 or IWD-5000.

The following ASME Code Cases will be used during the conduct of pressure testing.

Code Case N-533-1

On systems borated for the purpose of controlling reactivity, insulation shall be removed from pressure retaining bolted connections for visual examination VT-2 (IWA-5240), unless the requirements of Code Case N-616 (Relief Request 7) can be met. Otherwise, the examinations may be performed in accordance with the alternative requirements of Code Case N-533-1 (see Relief Request 5).

Code Case N-616

Code Case N-616 (Relief Request 7) covers the leaving of insulation on bolted connections during the conduct of pressure test on systems borated for the purpose of controlling reactivity. Relief Request 8 was written to request the use of this Code Case. Additional restrictions beyond those in the Code Case on the materials are required. These are:

1. Insulation will be removed for VT-2 visual examination during the system pressure test for any 17-4 PH stainless steel of 410 stainless steel stud or bolt aged at a temperature below 1100 degrees F or with hardness above  $R_C$  30.
2. For A-286 stainless steel studs or bolts, the preload must be verified to below 100Ksi or the thermal insulation must be removed and the joint visually examined.
3. For nuts conforming to SA-194, removal of the insulation for visual examination will not be necessary.
4. A four-hour hold time at operating temperature and pressure will be performed prior to conducting the VT-2 examination.

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10.0 Augmented and Other Programs

This section identifies augmented inspection programs maintained within the ISI Program. Augmented examinations are not required by ASME Section XI. However, due to the nature of the augmented requirements, these programs have been included within the ISI Program. These augmented programs satisfy NRC requirements, operating experience, engineering judgment, etc. Augmented program revisions or deviations shall be governed by the referenced documents.

10.1 Reactor Coolant Pump Flywheels

Technical Specification 5.5.6 requires PBNP to implement a Reactor Coolant Pump (RCP) Flywheel Inspection Program to ensure its integrity.

The following examination requirements are specified in TS 5.5.6:

1. A qualified in-place ultrasonic examination over the volume from the inner bore of the flywheel to the circle one-half of the outer radius, or,
2. A surface examination (MT or PT) of exposed surfaces of the removed flywheel.

The examinations will be performed at approximately 10-year intervals coinciding with periodic RCP maintenance. The examinations will be performed as part of the ISI Program.

Fourth 10-year Interval Examinations

Reactor Coolant Pump	Unit 1	Unit 2
A	2011	2005
B	2009	2010

Table 11 – RCP Motor Flywheel Examination Schedule

Reference documents:

Technical Specification 5.5.6, Reactor Coolant Pump Inspection Program

Technical Requirement Manual 4.6, RCP Flywheel Inspection Program

WCAP-14535A, Topical Report on Reactor Coolant Pump Flywheel Inspection Elimination

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10.2 Reactor Pressure Vessel Augmented Examination

Reactor Pressure Vessel Examinations were performed during the 1998 outage for Unit 1 and 1999 for Unit 2. These examinations met the requirements stated in 10 CFR 50.55a(g)(6)(ii)(A), Augmented Examination of Reactor Vessel. Relief requests were submitted to the USNRC to cover those areas where examination requirements were not satisfied.

The examinations covered the reactor vessel shell welds specified in Item No. B1.10 of Examination Category B-A, Pressure Retaining Welds in Reactor Vessels, as defined in the 1989 Edition of Section XI, Division 1, of the ASME Boiler and Pressure Vessel Code, subject to the conditions specified in 10 CFR 50.55a(g)(6)(ii)(A)(3) and (4).

10.3 High Energy Fluid Systems

Welds in those portions of systems addressed in USNRC Branch Technical Position APCSB 3.1 paragraph B.2.c(4) will be examined during the interval.

10.4 Boric Acid Corrosion Program

Boric acid leaks can result in accelerated corrosion rates of carbon and low alloy steel components. This corrosion can lead to failures of components. There are numerous documented industry events of failures as a result of boric acid corrosion. The NRC has issued information notices and generic letters documenting these failures. The PBNP response to GL 88-05 indicates that those locations, which represent the principal locations where boric acid leakage will occur, will be inspected.

PBNP has incorporated these inspections into the RCS leak test that is performed each refueling outage. The applicable procedures have been modified to add components (outside normal leak test boundaries) to inspect for boric acid leakage. When components are identified to exhibit leakage or evidence of leakage, they are evaluated and corrective action is taken as necessary.

10.5 Heavy Load Lifting Devices

Heavy load lifting devices shall be examined in accordance with Plant Regulatory Commitments 7326 and 7327. These commitments are outlined in a Wisconsin Electric Company letter to Harold R. Denton of NRR, Special Lifting Devices, Enclosure Item 4, dated September 28, 1983. This letter defines how the plant will administer the requirements of ANSI N14.6 and NUREG 0612. The devices subject to the requirement are:

Reactor Pressure Vessel Head Lift Rig  
Reactor Pressure Vessel Internals Lift Rig  
Reactor Coolant Pump Lift Rig

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A Safety Evaluation Report (SER) was received from the NRC, March 27, 1984, detailing what Wisconsin Electric committed to for heavy loads. The SER only required the examinations be performed over a 10-year period consistent with ASME guidelines.

10.6 Safety Injection Weld Examination

Information Notice 97-019, "Safety Injection System Weld Flaw at Sequoyah Nuclear Power Plant Unit 2", was evaluated for applicability to PBNP. This Information Notice discussed the cracking of a pipe crack in a low pressure Safety Injection line. The records for similar welds at PBNP did not indicate any unusual repairs, but as a conservative measure, it was decided to look at the applicable welds during the course of normal ISI examinations. PBNP decided to examine the welds to ensure no problems were occurring as an internal commitment. These welds were also selected in accordance with the RI-ISI Program as they are all considered High Risk. The following welds were found to have the configuration discussed in the Information Notice:

Unit 1			
Valve Number	Component ID	Isometric Number	When Examined
1SI-853C	SIS-06-SI-1002-09	ISI-PRI-1129	4/11/2001
1SI-853D	AC-06-SI-1001-21	ISI-PRI-1127	4/11/2001
1SI-867A	SIS-10-SI-1003-19	ISI-PRI-1125	4/19/2001
1SI-867B	AC-10-SI-1001-19	ISI-PRI-1123	4/21/2001

Unit 2			
Valve Number	Component ID	Isometric Number	When Examined
2SI-853C	SIS-06-SI-2002-21	ISI-PRI-2131	Scheduled U2R28
2SI-853D	AC-06-SI-2001-18	ISI-PRI-2129	Scheduled U2R26
2SI-867A	SIS-10-SI-2003-21	ISI-PRI-2126	Scheduled U2R30
2SI-867B	AC-06-SI-2001-17	ISI-PRI-2124	Scheduled U2R26

Table 12 – IN 97-019 Examination Schedule

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11.0 Appendix VIII Personnel Qualification

- 11.1 On November 22, 1999, the NRC took action to implement a rulemaking in the Code of Federal Regulations (CFR) for the expedited implementation of Appendix VIII supplements. This rule defined additional Ultrasonic (UT) Nondestructive Examination (NDE) personnel and procedure qualification requirements.

On August 3, 2001, the NRC issued a proposed rule for implementing the 1998 Edition of Section XI with Addenda through 2000. This proposed rule modifies the requirements of the previous rule on Appendix VIII implementation. The ISI Program at PBNP will implement the proposed rule, as agreed upon with the USNRC in an SER dated November 6, 2001 (Reference 14.37). This rule is to implement Appendix VIII and the supplements of Appendix VIII of the 1998 Edition of ASME Section XI with Addenda through 2000. The following schedule has been implemented:

May 22, 2000 - Appendix VIII and Supplements 1, 2, 3, and 8  
November 22, 2000 - Supplements 4 and 6  
November 22, 2001 - Supplement 11  
November 22, 2002 - Supplements 5, 7, 10, 12, and 13

Appendix VIII and the supplements will apply when implementing paragraph IWA-2232 (Reference 10 CFR 50.55a[g][6][ii][C][2]). All other NDE will continue to be performed in accordance with the requirements of the 98A00 Code.

In the discussion of the proposed rule, the NRC states they will permit the licensees to implement either Appendix VIII, Performance Demonstration for Ultrasonic Examination Systems, to Section XI, Division 1, 98A00, or Appendix VIII as modified by the proposed rule (Reference 10 CFR 50.55a[b][2][xv]). PBNP will implement Appendix VIII as modified by the proposed rule.

In the discussion of the proposed rule, the NRC states they have reviewed the latest version of the PDI Program for vessels, piping, and bolting. They have agreed the PDI Program would provide reasonable assurance of detecting flaws of concern in Section XI components. Imposition of performance demonstration will enhance the overall level of assurance of the reliability of UT examination techniques in detecting and sizing flaws.

Appendix VIII provides requirements for performance demonstration for ultrasonic examination (UT) procedures, equipment, and personnel used to detect and size flaws. PBNP and non-PBNP personnel who perform ultrasonic examinations at PBNP shall be qualified by performance through the Performance Demonstration Initiative (PDI) Program Description, Revision 2, dated October 1, 2000, at the Performance Demonstration Administrators facility or designated alternate. Successful completion of the PDI Performance Demonstration is considered to fulfill the practical examination requirements for UT Level II or Level III certification. These requirements are found in the PBNP NDE Procedures manual.

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PDI procedures will be reviewed and qualification demonstrations witnessed by the Authorized Inspection Agency (AIA), which is Hartford Steam Boiler Inspection and Insurance Company of Connecticut (HSBCT) at PBNP. HSBCT has documented their review and acceptance of the generic PDI procedures in letters by Robert N. McGill, Inspection Specialist.

Prior to a specific Supplement of Appendix VIII coming into effect, UT shall be performed to the requirements of the 98A00 Code. Components not addressed by Appendix VIII (e.g., Steam Generator, Pressurizer, etc.) will be examined in accordance with the requirements of the 98A00 Code unless otherwise noted.

- 11.2 Level I and II nondestructive examination personnel, and personnel qualified under the American Society for Nondestructive Testing Central Certification Program and ANSI/ASNT CP-189 shall be recertified on a 3-year interval in lieu of the 5-year interval specified in IWA-2314(a) and IWA-2314(b) (Reference 10 CFR 50.55a[b][2][xviii][A]).

Paragraph IWA-2316 may be used to qualify visual examination personnel only for the performance of VT-2 visual examinations when the proficiency of the training required under IWA-2316 is demonstrated by administering an initial qualification examination and administering recertification examinations on a three-year interval (Reference 10 CFR 50.55a[b][2][xviii][B]).

Paragraph IWA-2317 may be used to qualify visual examination personnel only for the performance of VT-3 visual examinations when the proficiency of the training required under IWA-2317 is demonstrated by administering an initial qualification examination and administering re-certification examinations on a three-year interval (Reference 10 CFR 50.55a[b][2][xviii][A]).

The proposed rule also requires additional training for UT personnel. This consists of at least eight hours of hands on training on samples containing cracks no earlier than six months prior to performing examinations. PBNP shall comply with these additional training requirements for personnel performing Section XI UT examinations (Reference 10 CFR 50.55a[b][2][xiv]).

12.0 Repair/Replacement Activities

The requirements of the 1998 Edition with Addenda through 2000 of Section XI and the Repair and Replacement Program for Point Beach Nuclear Plant shall be met for Class 1, 2, and 3 piping and components and their supports. Procedure NP 7.2.5, Repair/Replacement Program, details specific requirements for the repair, replacement, or modification of ISI components.

Repair/replacement activities include welding, brazing, defect removal, and removing, adding, and modifying items or systems. The requirement areas applicable to procurement, design, installation, examination, and pressure testing of items within the scope of the ISI Program.

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The Repair and Replacement Program for IWE/IWL components is contained within the applicable ISI Program.

### 13.0 Relief Requests

Relief requests are required when there are situations where the Code requirements cannot be met or where an alternative is desired. Typical situations where relief requests are written:

On Class 1 and 2 weld examinations, relief is required if 90% or less of the Code required coverage was achieved (unable to meet Code examination requirements).

Radiation exposure in the examination area is very high (hardship).

The use of different forms for the Summary Report (desire an alternative).

There are three situations where it is recognized a need for relief from requirements may be granted or authorized:

#### 10 CFR 50.55a(b)

(3) Proposed alternatives to the requirements of paragraphs (c), (d), (e), (f), (g), and (h) of this section or portions thereof may be used when authorized by the Director of the Office of Nuclear Reactor Regulation. The applicant shall demonstrate that:

- (i) The proposed alternatives would provide an acceptable level of quality and safety; or
- (ii) compliance with the specified requirements of this section would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.

#### 10 CFR 50.55a(g)

(6)(i) The Commission will evaluate determinations under paragraph (g)(5) of this section that Code requirements are impractical. The Commission may grant such relief and may impose such alternative requirements as it determines is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest giving due consideration to the burden upon the licensee that could result if the requirements were imposed on the facility.

Relief requests will be reviewed for completeness, technical adequacy, and implementation. Reviewers may be the site ISI Coordinators, the ISI Specialist, Nondestructive Examination (NDE) personnel, and any other group the relief request may affect.

Relief requests will follow the following format:

Provide an itemized list of the specific weld(s) and/or component(s) for which relief is requested. Include the ASME Code Class, Examination Category, and Item Number(s). Relief cannot be granted for generic requests for relief.

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NOTE: Each relief request should contain only one Examination Category. Exceptions would be when the same relief will apply over several categories.

Report the Code examination requirement(s) for the specific weld(s) and/or component(s) for which relief is being requested.

Identify the specific Section XI examination or test requirements for the weld(s) and/or component(s) for which relief is being requested.

Provide a basis and technical justification to support the determination that the Code requirement is impractical (i.e., state and explain the basis for requesting relief). If the Code-required examination cannot be performed because of a limitation or obstruction, describe or provide drawings showing the specific limitation or obstruction. Address the following regarding why the Licensee feels relief should be granted (justification):

- How the proposed alternatives or partial examination provide a reasonable assurance of the continued structural integrity.
- The burden upon the Licensee should the Request for Relief be denied.
- Why public health and safety will not be jeopardized by the granting of relief.

If a partial Code-required examination can be performed, provide an estimate of the percentage of the Code-required examination that can be completed for each of the individual weld(s) and/or component(s) covered by the request for relief.

If justification for the request for relief is based on radiation considerations (ALARA), address the following:

- The total estimated man-rem exposure involved in the examination;
- the radiation levels at the test area;
- flushing or shielding capabilities that might reduce radiation levels;
- proposed alternative inspection techniques;
- the considerations involved in remote inspections;
- similar components in redundant systems or similar welds in the same systems that can be inspected;
- the results of previous Inservice Inspections that may help provide technical justification for the granting of relief; and
- the failure consequences of the component(s) that would not receive the Code required examination(s).

Provide the alternative examinations that will be applied. This should clearly spell out everything that will be performed.

Discuss the period of time for which relief is required.

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Note: Requests for relief are only applicable for the 10-year inspection interval during which relief was requested and approval does not apply for subsequent inspection intervals except when the NRC specifically states they are for longer periods of time. Relief Requests 1 and 2 will be applicable for the remainder of the life of the plant.

Relief requests are found in Appendix D.

#### 14.0 References

The Inservice Inspection Program for Class 1, 2, and 3 (or Quality Groups A, B, and C respectively) systems and components and supports, was developed after reviewing the following documents and procedures. Limitations of design, geometry, and materials of construction may have an impact on the implementation of some of these documents.

14.1 - 10 CFR 50.55a Code of Federal Regulations.

14.2 - ASME Section XI Code 1989 Edition.

14.3 - ASME Section XI Code 1995 Edition with no Addenda.

14.4 - ASME Section XI Code 1998 Edition with no Addenda.

14.5 - ASME Section XI Code 1998 Edition with Addenda through 2000.

14.6 - USNRC Regulatory Guide 1.26 - Quality Group Classifications and standards for Water-, Steam-, and Radioactive-Waste-Containing Components of Nuclear Power Plants, Revision 3, dated February 1976.

14.7 - USNRC Regulatory Guide 1.65 - Materials and Inspections for Reactor Vessel Closure Studs.

14.8 - USNRC Regulatory Guide 1.82 - Sump Augmented Exams for ECCS and CSS.

14.9 - USNRC Regulatory Guide 1.83 - Inservice Examination of Pressurized Water Reactor Steam Generator Tubes, Revision 1, dated July 1975.

14.10 - USNRC Regulatory Guide 1.84 - Design and Fabrication Code Case Acceptability, ASME Section III, Division 1.

14.11 - USNRC Regulatory Guide 1.85 - Materials Code Case Acceptability, ASME Section III, Division 1.

14.12 - USNRC Regulatory Guide 1.147 - Inservice Inspection Code Case Acceptability ASME Section XI.

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- 14.13 - USNRC Regulatory Guide 1.150 - Ultrasonic Testing of Reactor Vessel Welds during Preservice and Inservice Examinations, dated February 1983.
- 14.14 - USNRC Regulatory Guide 1.178 - An Approach for Plant-Specific Risk Informed Decision Making Inservice Inspection of Piping.
- 14.15 - USNRC Information Notice 89-77 - Debris in Containment Emergency Sumps and Incorrect Screen Configuration.
- 14.16 - USNRC Information Notice 89-77, Supplement 1.
- 14.17 - USNRC Information Notice 93-20, Thermal Fatigue Cracking of Feedwater Piping to Steam Generators, dated March 24, 1993.
- 14.18 - USNRC Information Notice 96-32, Implementation of 10 CFR 50.55a(g)(6)(ii)(A), Augmented Examination of Reactor Vessel.
- 14.19 - USNRC Generic Letter 83-15, Implementation of Regulatory Guide 1.150, Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations, Revision 1.
- 14.20 - USNRC Bulletin 88-11, Pressurizer Surge Line Thermal Stratification.
- 14.21 - USNRC Bulletin 79-13, Cracking in Feedwater System Piping.
- 14.22 - EPRI Document, Guideline For The Implementation Of Appendix VIII and 10 CFR 50.55a, Volume One Programmatic Implementation, dated 10/14/2000.
- 14.23 - The Performance Demonstration Initiative (PDI), a utility developed guideline for the qualification of ultrasonic examination personnel.
- 14.24 - Point Beach Nuclear Plant Technical Specifications (both custom and improved).
- 14.25 - Point Beach Nuclear Plant Final Safety Analysis Report.
- 14.26 - USNRC Standard Review Plan 6.6, Paragraph I.8, (for Class 2 Augmented Inspections).
- 14.27 - Branch Technical Position MEB 3-1, High Energy Fluid Systems, Protection Against Postulated Piping Failures in Fluid Systems Outside Containment.
- 14.28 - NUREG-0800, Section 9.5.4/9.5.8, Essential Class 3 Diesel Systems.
- 14.29 - First, Second, and Third Interval Inservice Inspection Long Term Plans for Point Beach Nuclear Plant.
- 14.30 - Third Interval Pressure Testing Program.

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14.31 - ISI Program Basis Document.

14.32 - Wisconsin Electric Power Company letter NPL-99-0473, dated August 27, 1999.

14.33 - Demonstration Program for Wisconsin Electric - Point Beach Nuclear Plant, titled, Volumetric Examination in Lieu of Surface Examination on Reactor Coolant Inlet, Outlet, and Safety Injection Nozzle to Safe-End Welds, by Southwest Research Institute, 17-8506-010, dated July 1997.

14.34 - Letter from Southwest Research Institute to Mr. Craig Prothero, Regenerative Heat Exchanger Nozzle-to-Shell and Inner Radius Examination Parameters and Recommended Calibration Block Design, dated December 12, 1997.

14.35 - Generic Aging Lessons Learned (GALL) Report.

14.36 - Safety Evaluation by the Office of Nuclear Reactor Regulation related to the Third and Fourth 10-Year Inservice Inspection Intervals, Relief Requests RR-1-23 and RR-2-29, Nuclear Management Company, LLC, Point Beach Nuclear Plant, Units 1 and 2, Docket Nos. 50-266 and 50-301, Change of ISI Program Interval Dates.

14.37 - Safety Evaluation by the Office of Nuclear Reactor Regulation related to the Third and Fourth 10-Year Inservice Inspection Intervals, Relief Requests R-1-24 and RR-2-30, Nuclear Management Company, LLC, Point Beach Nuclear Plant, Units 1 and 2, Docket Nos. 50-266 and 50-301, Use of ASME Code, Section XI, 1998 Edition with Addenda through 2000.

14.39 - SEM 7.11.1, Inservice Inspection and Testing Plans and Reports.

14.40 - SEM 7.11.2, ISI Datasheet Review and Indication Evaluation Guideline.

14.41 - NP 7.2.5, Repair/Replacement Program.

14.42 - NP 7.4.13, Inservice Inspection Program Procedure.

## Appendix A

### Previous Interval, Period, and Outage Dates

Commercial Service Date: Point Beach 1 - September 21, 1971

Point Beach Unit 1			
Interval/Period	Start	End	Length
1/1	12/21/1970	4/20/1974	1216
1/2	4/21/1974	8/20/1977	1217
1/3	8/21/1977	12/20/1980	1217
2/1	12/21/1980	4/20/1984	1216
2/2	4/21/1984	8/20/1987	1216
2/3	8/21/1987	12/20/1990	1217
3/1	12/21/1990	4/20/1994	1216
3/2	10/27/1994	12/14/1998	1509
3/3	12/15/1998	6/30/2002	1293
4/1	07/01/2002	06/30/2005	1095
4/2	07/01/2005	08/30/2009	1460
4/3	07/01/2009	08/30/2012	1095

Commercial Service Date: Point Beach Unit 2 - October 1, 1972

Point Beach Unit 2			
Interval/Period	Start	End	Length
1/1	10/01/1972	01/31/1976	1217
1/2	02/01/1976	06/01/1979	1216
1/3	05/30/1979	10/01/1982	1220
2/1	09/30/1982	01/30/1986	1218
2/2	02/01/1986	06/01/1989	1216
2/3	05/30/1989	11/16/1992	1266
3/1	11/17/1992	03/16/1996	1215
3/2	03/17/1996	07/16/1999	1216
3/3	07/17/1999	06/30/2002	944
4/1	07/01/2002	06/30/2005	1095
4/2	07/01/2005	08/30/2009	1460
4/3	07/01/2009	08/30/2012	1095

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Point Beach Unit 1						
Outage No.	Interval	Period	Outage	Start Date	End Date	Document ID
Preservice	0	0	0	1970	N/A	microfilm 11.10.2.1
U1R1	1	1	1	09/30/1972	03/03/1973	microfilm 11.10.2.1
U1R2	1	1	2	04/06/1974	06/07/1974	microfilm 11.10.2.1
U1R3	1	2	1	11/16/1975	01/08/1976	microfilm 11.10.2.1
U1R4	1	2	2	10/02/1976	11/24/1976	microfilm 11.10.2.1
U1R5	1	2	3	10/04/1977	11/03/1977	microfilm 11.10.2.1
U1R6	1	3	1	09/20/1978	10/15/1978	microfilm 11.10.2.1
U1R7	1	3	2	10/04/1979	12/01/1979	microfilm 11.10.2.1
U1R8	1	3	3	11/28/1980	12/23/1980	microfilm 11.10.2.1
U1R9	2	1	1	10/09/1981	12/09/1981	microfilm 11.10.2.1
U1R10	2	1	2	10/22/1982	12/07/1982	microfilm 11.10.2.1
U1R11*	2	1	3	10/01/1983	04/07/1984	microfilm 11.10.2.1
U1R12	2	2	1	04/05/1985	06/19/1985	microfilm 11.10.2.1
U1R13	2	2	2	04/11/1986	05/20/1986	microfilm 11.10.2.1
U1R14	2	2	3	04/04/1987	06/01/1987	microfilm 11.10.2.1
U1R15	2	2	4	04/08/1988	05/20/1988	microfilm 11.10.2.1
U1R16	2	3	1	04/02/1989	05/16/1989	microfilm T7.3
U1R17	2	3	2	03/31/1990	05/16/1990	microfilm T7.3
U1R18	3	1	1	04/06/1991	05/16/1991	microfilm T7.3
U1R19	3	1	2	04/11/1992	06/05/1992	microfilm T7.3
U1R20	3	1	3	03/27/1993	05/06/1993	microfilm T7.3
U1R21	3	2	1	04/02/1994	04/30/1994	microfilm T7.3
U1R22	3	2	2	03/11/1995	04/17/1995	microfilm T7.3
U1R23	3	2	3	03/30/1996	04/24/1996	EDMS Correspondence B045669-01
<b>Turbine Upgrade and Maintenance**</b>				02/18/1997	12/07/1997	
U1R24	3	3	1	02/14/1998	06/30/1998	Hard copy file Section T7.3
U1R25	3	3	2	10/15/1999	12/10/1999	Hard copy file Section T7.3
U1R26	3	3	3	04/07/2001	5/13/2001	Hard copy file Section T7.3
U1R27	4	1	1	09/14/2002	scheduled	
U1R28	4	1	2	03/27/2004	scheduled	
U1R29	4	2	1	09/24/2005	scheduled	
U1R30	4	2	2	03/27/2007	approximate	
U1R31	4	3	1	09/24/2008	approximate	
U1R32	4	3	2	03/27/2010	approximate	

\* Steam Generator Replacement outage lasted 189 days.

\*\* Turbine Upgrade and Maintenance outage lasted 292 days.

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Point Beach Unit 2						
Outage No.	Interval	Period	Outage	Start Date	End Date	Document ID
Preservice	0	0	0	1972	N/A	
U2R1	1	1	1	10/16/1974	12/21/1974	
U2R2	1	2	1	02/26/1976	03/26/1976	
U2R3	1	2	2	03/04/1977	04/23/1977	
U2R4	1	2	3	03/22/1978	04/18/1978	
U2R5	1	2	4	03/22/1979	04/13/1979	
U2R6	1	3	1	04/11/1980	05/14/1980	
U2R7	1	3	2	04/17/1981	05/21/1981	
U2R8	1	3	3	04/15/1982	05/26/1982	
U2R9	2	1	1	03/25/1983	07/01/1983	
U2R10	2	1	2	09/28/1984	11/20/1984	
U2R11	2	1	3	10/05/1985	11/24/1985	
U2R12	2	2	1	09/26/1986	11/29/1986	
U2R13	2	2	2	10/02/1987	11/18/1987	
U2R14	2	2	3	10/08/1988	11/22/1988	
U2R15	2	3	1	09/23/1989	11/25/1989	
U2R16	2	3	2	10/06/1990	11/18/1990	
U2R17	2	3	3	09/28/1991	11/14/1991	
U2R18	2	3	4	09/26/1992	11/18/1992	
U2R19	3	1	1	09/25/1993	10/30/1993	
U2R20	3	1	2	09/24/1994	11/01/1994	
U2R21	3	1	3	10/07/1995	12/03/1995	
U2R22 *	3	2	1	10/05/1996	08/16/1997	NPC 1999-05796
U2R23	3	2	2	12/04/1998	03/04/1999	NPC 1999-06002
U2R24	3	3	1	10/14/2000	12/27/2000	
U2R25	3	3	2	04/13/2002	scheduled	
U2R26	4	1	1	09/13/2003	scheduled	
U2R27	4	1	2	03/19/2005	scheduled	
U2R28	4	2	1	09/13/2006	approximate	
U2R29	4	2	2	03/19/2008	approximate	
U2R30	4	3	1	09/13/2009	approximate	
U2R31	4	3	2	03/19/2011	approximate	

\* Steam Generator Replacement outage lasted 315 days.

## Appendix B

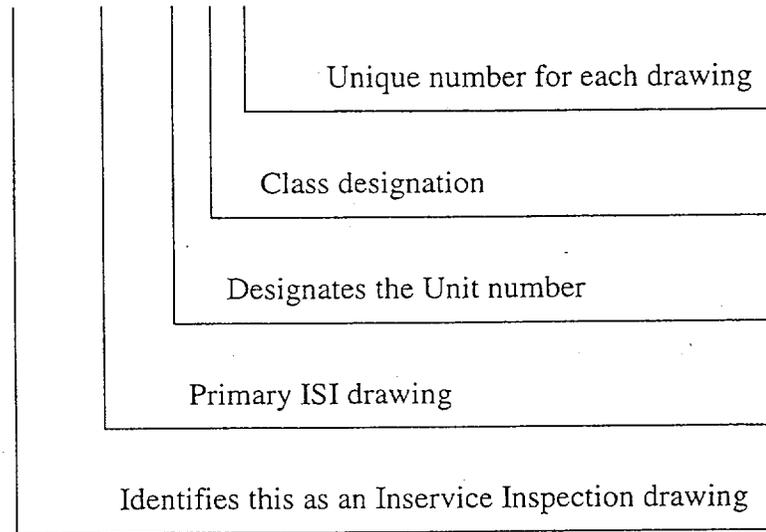
# Inservice Inspection Isometric Drawings

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ISI isometric drawings have been drawn to identify the locations of components. The ISI drawing numbers consist of three parts:

ISI - PRI - 1 2 34



For information on drawing symbols, refer to ISI-PRI-0000.

Each drawing will be checked against a controlled copy to ensure the latest revision is used.

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<b>Unit 1 Class 1 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-0000	ISI Isometric Legend
ISI-PRI-1101	Reactor Pressure Vessel
ISI-PRI-1102	Reactor Pressure Vessel Head to Flange Weld
ISI-PRI-1103	Reactor Pressure Vessel Bolting and Control Rod Drive Housings
ISI-PRI-1104	Pressurizer Upper Head
ISI-PRI-1105	Pressurizer T-1
ISI-PRI-1106	Steam Generator Lower Head
ISI-PRI-1107	Regenerative Heat Exchanger HX-2
ISI-PRI-1108	Reactor Coolant Pump Supports Bottom View
ISI-PRI-1109	Reactor Coolant Pump Casing Welds
ISI-PRI-1110	Reactor Coolant Pump Flange and Bolting Outline
ISI-PRI-1111	Control Rod Drive Housing Detail
ISI-PRI-1112	Steam Generators HX-1A and HX-1B (Bottom View)
ISI-PRI-1113 Through ISI-PRI-1119	Not Used
ISI-PRI-1120	Main Reactor Coolant Loop "A"
ISI-PRI-1121	Main Reactor Coolant Loop "B"
ISI-PRI-1122	RHR Suction Off Loop "A"
ISI-PRI-1123	Accumulator "B" to Loop "B"
ISI-PRI-1124	Accumulator "B" Line
ISI-PRI-1125	Accumulator "A" to Loop "A"
ISI-PRI-1126	Pressurizer Surge Line
ISI-PRI-1127	Reactor Vessel Injection and Core Deluge to RPV
ISI-PRI-1128	Reactor Vessel Injection and Core Deluge to RPV
ISI-PRI-1129	Core Deluge to RPV
ISI-PRI-1130	RHR and Core Deluge to RPV
ISI-PRI-1131	Pressurizer Safety
ISI-PRI-1132	Pressurizer Spray
ISI-PRI-1133	Pressurizer Spray from Loop "A"
ISI-PRI-1134	Pressurizer Spray from Loop "B"
ISI-PRI-1135	Pressurizer Power Operated Relief Valves
ISI-PRI-1136	Loop "A" RTD Bypass
ISI-PRI-1137	Loop "B" RTD Bypass
ISI-PRI-1138	Charging Line to/from Regenerative Heat Exchanger
ISI-PRI-1139	Auxiliary Charging
ISI-PRI-1140	Auxiliary Spray
ISI-PRI-1141	Letdown
ISI-PRI-1142	Reactor Coolant Pump "A" Seal Water Supply
ISI-PRI-1143	Reactor Coolant Pump "A" Seal Water Supply
ISI-PRI-1144	Reactor Coolant Pump "B" Seal Water Supply
ISI-PRI-1145	Reactor Coolant Pump "B" Seal Water Supply

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<b>Unit 1 Class 1 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-1146	Letdown from Loop "B" Cold Leg
ISI-PRI-1147	Letdown from Loop "B" Cold Leg
ISI-PRI-1148	Loop "A" Drain
ISI-PRI-1149	Safety Injection to Loop "B" Cold Leg
ISI-PRI-1150	High Head Safety Injection to Loop "A"
ISI-PRI-1151	Auxiliary Charging

<b>Unit 1 Class 2 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-1201	Steam Generators HX-1A and HX-1B
ISI-PRI-1203	Regenerative Heat Exchanger HX-2
ISI-PRI-1204	Residual Heat Removal Heat Exchangers HX-11A and HX-11B
ISI-PRI-1205	Seal Water Heat Exchanger HX-5
ISI-PRI-1206 through ISI-PRI-1219	Not Used
ISI-PRI-1220	RHR to Accumulator "B" Line in Primary Auxiliary Building
ISI-PRI-1221	RHR to Accumulator "B" Line in Containment
ISI-PRI-1222	RHR Pump "A" Suction from Containment Sump and Loop "A" Hot Leg
ISI-PRI-1223	RHR Pump "B" Suction from Containment Sump and Loop "A" Hot Leg
ISI-PRI-1224	RHR Pump Suction from Loop "A" Hot Leg in Containment
ISI-PRI-1225	RHR Heat Exchanger HX-11A Outlet
ISI-PRI-1226	RHR Heat Exchanger HX-11B Outlet
ISI-PRI-1227	RHR Pumps Discharge to HX-11A and Cross-tie
ISI-PRI-1228	RHR Heat Exchanger HX-11B Inlet
ISI-PRI-1229	Residual Heat Removal to Refueling Water Storage Tank
ISI-PRI-1230	RHR Heat Exchanger HX-11A Bypass
ISI-PRI-1231	RHR HX-11A Outlet to SI and CS Pumps
ISI-PRI-1232	RHR HX-11B Outlet to SI and CS Pumps
ISI-PRI-1233 and ISI-PRI-1234	Not Used
ISI-PRI-1235	Safety Injection Pump "A" Inlet and Outlet
ISI-PRI-1236	Safety Injection Pump "B" Inlet and Outlet
ISI-PRI-1237	RHR Heat Exchangers to Reactor Vessel
ISI-PRI-1238	RHR to RPV Injection Line
ISI-PRI-1239	Safety Injection Line
ISI-PRI-1240	Loop "A" Main Steam Inside Containment
ISI-PRI-1241	Loop "B" Main Steam Inside Containment
ISI-PRI-1242	Loop "A" Main Steam Outside Containment

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<b>Unit 1 Class 2 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-1243	Loop "B" Main Steam Outside Containment
ISI-PRI-1244	Feedwater Loop "A"
ISI-PRI-1245	Loop "A" Feedwater Inside Containment
ISI-PRI-1246	Loop "B" Feedwater Inside Containment
ISI-PRI-1247	RHR HX-11A Outlet to SI and CS Pumps
ISI-PRI-1248	RHR HX-11B Outlet to SI and CS Pumps
ISI-PRI-1249	Safety Injection Pump "B" Discharge to Loop "B" and RPV
ISI-PRI-1250	Safety Injection Pump "A" Discharge to Loop "A" and RPV
ISI-PRI-1251	Safety Injection Pump "A" Discharge to Loop "A" and "B" Cold Leg
ISI-PRI-1252	Safety Injection Pump P-15A Discharge
ISI-PRI-1253	Safety Injection Pump P-15B Discharge
ISI-PRI-1254	Safety Injection Pumps P-15A and P-15B Discharge
ISI-PRI-1255	Safety Injection Pumps P-15A and P-15B Discharge
ISI-PRI-1256	Safety Injection Pump P-15A Test Line
ISI-PRI-1257	Safety Injection Pump P-15B Test Line
ISI-PRI-1258	Safety Injection Pumps P-15A and P-15B Test Line
ISI-PRI-1259	Containment Spray Pumps P-14A and P-14B Test Line
ISI-PRI-1260	Containment Spray Pumps P-14A and P-14B Discharge
ISI-PRI-1261	Loop "A" Main Steam Inside Containment
ISI-PRI-1262	Loop "B" Main Steam Inside Containment
ISI-PRI-1263	RWST Discharge to SI, RHR, and CS Pumps
ISI-PRI-1264	Safety Injection to RHR Pumps
ISI-PRI-1265	Safety Injection from RWST
ISI-PRI-1266	Safety Injection from RWST
ISI-PRI-1267	Safety Injection to Safety Injection Pumps
ISI-PRI-1268	Safety Injection to Containment Spray Pumps
ISI-PRI-1269	Auxiliary Feedwater to Steam Generator A
ISI-PRI-1270 sh. 1	Auxiliary Feedwater to Steam Generator A
ISI-PRI-1270 sh. 2	Auxiliary Feedwater to Steam Generator A
ISI-PRI-1271 sh. 1	Auxiliary Feedwater to Steam Generator A
ISI-PRI-1271 sh. 2	Auxiliary Feedwater to Steam Generator A
ISI-PRI-1272	Auxiliary Feedwater to Steam Generator B
ISI-PRI-1273 sh. 1	Auxiliary Feedwater to Steam Generator B
ISI-PRI-1273 sh. 2	Auxiliary Feedwater to Steam Generator B
ISI-PRI-1274 sh. 1	Auxiliary Feedwater to Steam Generator B

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<b>Unit 1 Class 2 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-1274 sh. 2	Auxiliary Feedwater to Steam Generator B

<b>Unit 1 Class 3 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-1301 through ISI-PRI-1319	Not Used
ISI-PRI-1320	Service Water Pump Discharge and Strainers
ISI-PRI-1321	Service Water Pump Discharge and Strainers
ISI-PRI-1322	Service Water Supply Header Air Compressor and Diesel Generator Rooms
ISI-PRI-1323	Service Water Supply Header Diesel Generator Room
ISI-PRI-1324	Service Water Supply Header Auxiliary Feed Pump Tunnel
ISI-PRI-1325	Service Water Supply Header to Area 3
ISI-PRI-1326	Service Water Supply Header Auxiliary Building
ISI-PRI-1327	Service Water Supply Header Auxiliary Building
ISI-PRI-1328	Service Water Supply Header Auxiliary Building
ISI-PRI-1329	Service Water Supply to HX-15B
ISI-PRI-1330	Service Water Supply to HX-15D
ISI-PRI-1331	Service Water Supply to HX-15A
ISI-PRI-1332	Service Water Supply to HX-15A
ISI-PRI-1333	Service Water Supply to Auxiliary Feedwater Pump P-29
ISI-PRI-1334	Service Water Supply to Auxiliary Feedwater Pump P-38A
ISI-PRI-1335	Service Water Supply to Auxiliary Feedwater Pump P-38B
ISI-PRI-1336	Spent Fuel Pool Discharge Heat Exchangers and Pool
ISI-PRI-1337	Spent Fuel Pool Pump Suction
ISI-PRI-1338	Service Water Supply Spent Fuel Pool Heat Exchangers
ISI-PRI-1339 Through ISI-PRI-1346	Not Used
ISI-PRI-1347	Service Water Supply Header to Area 3
ISI-PRI-1348	Service Water to Decon Area Auxiliary Building
ISI-PRI-1349	Service Water Supply to Component Cooling Water HX-12A, HX-12B, HX-12C, and HX-12D
ISI-PRI-1350	Service Water Return from Component Cooling Water HX-12A, HX-12B, HX-12C, and HX-12D
ISI-PRI-1351	Service Water Supply to Component Cooling Water HX-12A, HX-12B, HX-12C, and HX-12D
ISI-PRI-1352	Service Water Discharge from Auxiliary Feedwater Pump 1P-29
ISI-PRI-1353	Service Water Discharge from Auxiliary Feedwater Pump P-38A
ISI-PRI-1354	Service Water Discharge from Auxiliary Feedwater Pump P-38A

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<b>Unit 1 Class 3 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-1355 Through ISI-PRI-1360	Not Used
ISI-PRI-1361	Service Water Return from 1HX-15A
ISI-PRI-1362	Service Water Return from 1HX-15A
ISI-PRI-1363	Service Water Return from 1HX-15B
ISI-PRI-1364	Service Water Return from 1HX-15B
ISI-PRI-1365	Service Water Return from 1HX-15C
ISI-PRI-1366	Service Water Return from 1HX-15C
ISI-PRI-1367	Service Water Return from 1HX-15D
ISI-PRI-1368	Service Water Return from 1HX-15D
ISI-PRI-1369 or P-167 sh. 2	Service Water Supply to Spent Fuel Pool Heat Exchangers or Alternate Service Water Supply to Spent Fuel Pool Heat Exchangers HX-13A and HX-13B

<b>Unit 1 Augmented Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-1401	Reactor Pressure Vessel Head Lift Rig
ISI-PRI-1402	Reactor Pressure Vessel Internals Lift Rig
ISI-PRI-1403	Reactor Coolant Pump Motor Lift Rig
ISI-PRI-1404	Reactor Coolant Pump Motor Flywheels
ISI-PRI-1405	Main Steam Bypass Energy Absorbers
0870 005-001 Sh. 1	Pressurizer Safety and Relief Valve Discharge Piping System and Reactor Coolant System

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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<b>Unit 2 Class 1 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2101	Reactor Pressure Vessel
ISI-PRI-2102	Reactor Pressure Vessel Head to Flange Weld
ISI-PRI-2103	Reactor Pressure Vessel Bolting and Control Rod Drive Housings
ISI-PRI-2104	Pressurizer Upper Head
ISI-PRI-2105	Pressurizer T-1
ISI-PRI-2106	Steam Generator Lower Head
ISI-PRI-2107	Regenerative Heat Exchanger HX-2
ISI-PRI-2108	Reactor Coolant Pump Supports Bottom View
ISI-PRI-2109	Reactor Coolant Pump Casing Welds
ISI-PRI-2110	Reactor Coolant Pump Flange and Bolting Outline
ISI-PRI-2111	Control Rod Drive Housing Detail
ISI-PRI-2112	Steam Generators HX-1A and HX-1B (Bottom View)
ISI-PRI-2113 Through ISI-PRI-2119	Not Used
ISI-PRI-2120	Main Reactor Coolant Loop "A"
ISI-PRI-2121	Main Reactor Coolant Loop "B"
ISI-PRI-2122	RHR Suction Off Loop "A"
ISI-PRI-2123	RHR Suction Header
ISI-PRI-2124	Loop "B" Injection
ISI-PRI-2125	RHR to Loop "B"
ISI-PRI-2126	Loop "A" Injection
ISI-PRI-2127	Pressurizer Surge Line
ISI-PRI-2128	Reactor Vessel Injection
ISI-PRI-2129	Reactor Vessel Injection
ISI-PRI-2130	SIS Penetration to RPV
ISI-PRI-2131	RHR to Reactor Pressure Vessel
ISI-PRI-2132	Reactor Vessel Injection
ISI-PRI-2133	Pressurizer Safety
ISI-PRI-2134	Pressurizer Spray
ISI-PRI-2135	Spray Line from Loop "A" to Pressurizer
ISI-PRI-2136	Spray Line from Loop "B" to Pressurizer
ISI-PRI-2137	Pressurizer Power Operated Relief Valves
ISI-PRI-2138	Loop "A" RTD Bypass
ISI-PRI-2139	Loop "B" RTD Bypass
ISI-PRI-2140	Charging Line to Loop "A"
ISI-PRI-2141	Auxiliary Charging
ISI-PRI-2142	Pressurizer Spray
ISI-PRI-2143	Auxiliary Spray to Pressurizer
ISI-PRI-2144	Letdown
ISI-PRI-2145	Reactor Coolant Pump "A" Seal Water Supply
ISI-PRI-2146	Reactor Coolant Pump "A" Seal Water Supply

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<b>Unit 2 Class 1 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2147	Reactor Coolant Pump "B" Seal Water Supply
ISI-PRI-2148	Reactor Coolant Pump "B" Seal Water Supply
ISI-PRI-2149	Cold Leg Injection
ISI-PRI-2150	Cold Leg Injection to Loop "A"
ISI-PRI-2151	Letdown from Loop "B" Cold Leg
ISI-PRI-2152	Letdown from Loop "B" Cold Leg
ISI-PRI-2153	Excess Letdown to CVCS
ISI-PRI-2154	Core Deluge Injection Line
ISI-PRI-2155	Cold Leg Injection Loop "B"
ISI-PRI-2156	Core Deluge Header
ISI-PRI-2157	Pressurizer Spray
ISI-PRI-2158	Letdown from Loop "B" Cold Leg
ISI-PRI-2159	Cold Leg Injection Loop "B"

<b>Unit 2 Class 2 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2201	Steam Generators HX-1A and HX-1B
ISI-PRI-2202	Not Used
ISI-PRI-2203	Regenerative Heat Exchanger HX-2
ISI-PRI-2204	Residual Heat Removal Heat Exchangers HX-11A and HX-11B
ISI-PRI-2205	Seal Water Heat Exchanger HX-5
ISI-PRI-2206 through ISI-PRI-2219	Not Used
ISI-PRI-2220	RHR from HX to Penetration
ISI-PRI-2221	RHR from HX to Penetration
ISI-PRI-2222	RHR to Safety Injection Accumulator "B" in Containment
ISI-PRI-2223	RHR Pump "A" Suction from Containment Sump and Loop "A" Hot Leg
ISI-PRI-2224	RHR Pump "B" Suction from Containment Sump and Loop "B" Hot Leg
ISI-PRI-2225	RHR Pump Suction from Loop "A" Hot Leg in Containment
ISI-PRI-2226	RHR Heat Exchanger HX-11A Outlet
ISI-PRI-2227	RHR Heat Exchanger HX-11B Outlet
ISI-PRI-2228	Residual Heat Removal Pump Discharge
ISI-PRI-2229	RHR Heat Exchanger HX-11B Inlet
ISI-PRI-2230	RHR to "B" Safety Injection Pump Suction
ISI-PRI-2231	Residual Heat Removal Heat Exchanger Bypass
ISI-PRI-2232	RHR to "A" Safety Injection Pump Suction
ISI-PRI-2233	RHR to "A" Safety Injection Pump Suction
ISI-PRI-2234	RHR to "B" Safety Injection Pump Suction

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<b>Unit 2 Class 2 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2235	RHR to Safety Injection Pump Suction
ISI-PRI-2236 and ISI-PRI-2237	Not Used
ISI-PRI-2238	Safety Injection Pump "A" Inlet and Outlet
ISI-PRI-2239	Safety Injection Pump "B" Inlet and Outlet
ISI-PRI-2240	RHR to Reactor Pressure Vessel
ISI-PRI-2241	RHR to Reactor Pressure Vessel Injection
ISI-PRI-2242	RHR to Reactor Pressure Vessel Injection
ISI-PRI-2243	Loop "A" Main Steam Inside Containment
ISI-PRI-2244	Loop "A" Main Steam Inside Containment
ISI-PRI-2245	Loop "B" Main Steam Inside Containment
ISI-PRI-2246	Loop "A" Main Steam Outside Containment
ISI-PRI-2247	RHR to "B" Safety Injection Pump Suction
ISI-PRI-2248	Loop "B" Main Steam Outside Containment
ISI-PRI-2249	Feedwater Loop "A"
ISI-PRI-2250	Feedwater Loop "A"
ISI-PRI-2251	Feedwater Loop "B"
ISI-PRI-2252	RHR Pump "A" Suction from Containment Sump and Loop "A" Hot Leg
ISI-PRI-2253	RHR Pump "B" Suction from Containment Sump and Loop "B" Hot Leg
ISI-PRI-2254	Safety Injection Pump P-15A and P-15B Test Line
ISI-PRI-2255	Safety Injection Pump P-15B Test Line
ISI-PRI-2256	Safety Injection Pump P-15A Test Line
ISI-PRI-2257	Containment Spray Pumps P-14A and P-14B Test Line
ISI-PRI-2258	Core Deluge and Cold Leg Injection Header
ISI-PRI-2259	Core Deluge and Cold Leg Injection Header
ISI-PRI-2260	Core Deluge and Cold Leg Injection Header
ISI-PRI-2261	Core Deluge and Cold Leg Injection Header
ISI-PRI-2262	Core Deluge and Cold Leg Injection Header
ISI-PRI-2263	Core Deluge and Cold Leg Injection Header
ISI-PRI-2264	Core Deluge Header
ISI-PRI-2265	Core Deluge Header
ISI-PRI-2266	Core Deluge Header
ISI-PRI-2267	Containment Spray Pumps P-14A and P-14B Discharge
ISI-PRI-2268	Cold Leg Injection Header
ISI-PRI-2269	RHR to Safety Injection Pump Suction
ISI-PRI-2270	Containment Spray Pumps P-14A and P-14B Suction
ISI-PRI-2271	Loop "A" Main Steam Inside Containment
ISI-PRI-2272	Loop "B" Main Steam Inside Containment
ISI-PRI-2273	RWST Discharge to SI, RHR, and CS Pumps

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<b>Unit 2 Class 2 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2274	Safety Injection to RHR Pumps
ISI-PRI-2275	Safety Injection from RWST
ISI-PRI-2276	Safety Injection from RWST
ISI-PRI-2277	Safety Injection to Safety Injection Pumps
ISI-PRI-2278	Safety Injection to Containment Spray Pumps
ISI-PRI-2279	Auxiliary Feedwater to Steam Generator A
ISI-PRI-2280 sh. 1	Auxiliary Feedwater to Steam Generator A
ISI-PRI-2280 sh. 2	Auxiliary Feedwater to Steam Generator A
ISI-PRI-2281 sh. 1	Auxiliary Feedwater to Steam Generator A
ISI-PRI-2281 sh. 2	Auxiliary Feedwater to Steam Generator A
ISI-PRI-2282	Auxiliary Feedwater to Steam Generator B
ISI-PRI-2283 sh. 1	Auxiliary Feedwater to Steam Generator B
ISI-PRI-2283 sh. 2	Auxiliary Feedwater to Steam Generator B
ISI-PRI-2284	Auxiliary Feedwater to Steam Generator B

<b>Unit 2 Class 3 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2301 through ISI-PRI-2324	Not Used
ISI-PRI-2325	Service Water Supply to Auxiliary Feedwater Pump P-29
ISI-PRI-2326 through ISI-PRI-2328	Not Used
ISI-PRI-2329	Service Water Supply to HX-15D
ISI-PRI-2330	Service Water Supply to HX-15D
ISI-PRI-2331	Service Water Supply to HX-15A
ISI-PRI-2332	Service Water Supply to HX-15A
ISI-PRI-2333	Service Water Supply to HX-15C
ISI-PRI-2334	Service Water Supply to HX-15C
ISI-PRI-2335	Service Water Supply to HX-15B
ISI-PRI-2336	Service Water Supply to HX-15B
ISI-PRI-2337	Service Water Supply to HX-15B
ISI-PRI-2338	Service Water Discharge from Auxiliary Feedwater Pump 2P-29

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<b>Unit 2 Class 3 Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2339 through ISI-PRI-2343	Not Used
ISI-PRI-2344	Service Water Return from 2HX-15A
ISI-PRI-2345	Service Water Return from 2HX-15A
ISI-PRI-2346	Service Water Return from 2HX-15B
ISI-PRI-2347	Service Water Return from 2HX-15B
ISI-PRI-2348	Service Water Return from 2HX-15C
ISI-PRI-2349	Service Water Return from 2HX-15C
ISI-PRI-2350	Service Water Return from 2HX-15D
ISI-PRI-2351	Service Water Return from 2HX-15D

<b>Unit 2 Augmented Systems</b>	
<b>Drawing No.</b>	<b>Title</b>
ISI-PRI-2401	Reactor Pressure Vessel Head Lift Rig
ISI-PRI-2402	Reactor Pressure Vessel Internals Lift Rig
ISI-PRI-2403	Reactor Coolant Pump Motor Lift Rig
ISI-PRI-2404	Reactor Coolant Pump Motor Flywheel
ISI-PRI-2405	Main Steam Bypass Energy Absorbers
0870 005-001 Sh. 2	Pressurizer Safety and Relief Valve Discharge Piping System and Reactor Coolant System

## Appendix C

### Calibration Blocks

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

Ultrasonic calibration blocks controlled by PBNP are maintained at the plant. On occasion, blocks are borrowed from other plants or vendors. These blocks have been qualified for use at PBNP, and the certifications are on file in document control or may be filed in the ISI outage report when the blocks were used.

Calibration Block Number	Brief ID Number	Application	Remarks
10-SS-160-1.125-1-PTB	1-PTB	10" Stainless Steel Pipe	
3-SS-160-.438-2-PTB	2-PTB	3" Stainless Steel Pipe	
4-SS-160-.531-3-PTB	3-PTB	4" Stainless Steel Pipe	
6-SS-120-.562-4-PTB	4-PTB	6" Stainless Steel Pipe	
FS/NS-CSCL-5-PTB	5-PTB	Flange to Shell and Nozzle to Shell	
4-SS-120-.438-6-PTB	6-PTB	4" Stainless Steel Pipe	
PL-3-CSCL-7-PTB	7-PTB	3" Plate	Vessel
IR-CSCL-8-PTB	8-PTB	RPV, SG, and PZR Inner Radius	
3.5-.750-8-CS-9-PTB	9-PTB	Reactor Coolant Pump Stud	Superseded by 53-PTB
PL-3.0-SS-10-PTB	10-PTB	3" Vessel and Piping	Reactor Coolant
5-CSCL-11-PTB	11-PTB	5" Vessel	
7-CSCL-12-PTB	12-PTB	7" Vessel	
9-CSCL-13-PTB	13-PTB	9" Vessel	
IR-CSCL-14-PTB	14-PTB	Inlet and Outlet Nozzle Inner Radius	
10-SS-140-1.0-15-PTB	15-PTB	10" Stainless Steel Pipe	Superseded by 51-PTB
5.375-3.5-8-CS-16-PTB	16-PTB	Reactor Coolant Pump Nut	
8-SS-140-.820-17-PTB	17-PTB	8" Stainless Steel Pipe	
5.5-SS-XX-1.563-18-PTB	18-PTB	5-1/2" Stainless Steel Nozzles	
PL-1.5-CS-19-PTB	19-PTB	1.5" Plate	
8.5-6-8-CS-20-PTB	20-PTB	Reactor Pressure Vessel Nut	
6-1-8-CS-21-PTB	21-PTB	Reactor Pressure Vessel Stud	
SL-CSCL-22-PTB	22-PTB	Reactor Pressure Vessel Support Lug	
SI-CSCL-23-PTB	23-PTB	Safety Injection Nozzle	
29ID-CSS-X-3.0-24-PTB	24-PTB	Reactor Coolant Elbows	
18-CS-X-.688-25-PTB	25-PTB	Feedwater Nozzle to Reducer	
3-CS-80-.3-26-PTB	26-PTB	8" Stainless Steel Pipe	
4-SS-80-.337-27-PTB	27-PTB	8" Stainless Steel Pipe	
6-SS-10-.134-28-PTB	28-PTB	8" Stainless Steel Pipe	

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Calibration Block Number	Brief ID Number	Application	Remarks
6-SS-40-.280-29-PTB	29-PTB	8" Stainless Steel Pipe	
8-SS-10-.148-30-PTB	30-PTB	8" Stainless Steel Pipe	
8-SS-40-.322-31-PTB	31-PTB	8" Stainless Steel Pipe	
10-SS-10-.165-32-PTB	32-PTB	10" Stainless Steel Pipe	
10-SS-40-.365-33-PTB	33-PTB	10" Stainless Steel Pipe	
16-CS-80-.844-34-PTB	34-PTB	Feedwater Carbon Steel Pipe	Steam Generator
15-SS-X-1.56-35-PTB	35-PTB	Surge Nozzle Safe End	Pressurizer
6-SS-X-.750-36-PTB	36-PTB	Spray Nozzle	Pressurizer
4.9-SS-X-.7-37-PTB	37-PTB	Safety and Relief Nozzle to Safe-End	Pressurizer
PL-CSCL-5.0-38-PTB	38-PTB	5" Clad Plate	Pressurizer
IR-CCSCL-39-PTB	39-PTB	Manual Inner Radius,	Steam Generator/ Pressurizer
PL-CCSCL-5.0-40-PTB	40-PTB	5" Clad Plate	Steam Generator
PL-CS-3.0-41-PTB	41-PTB	3" Plate	Steam Generator
IR-CS-42-PTB	42-PTB	Manual Inner Radius	Steam Generator
PL-SS-.750-43-PTB	43-PTB	Stainless Steel Plate	RHR Heat Exchanger
31-CS-X-1.5-44-PTB	44-PTB	31" Carbon Steel Pipe	
30-CS-X-1.125-45-PTB	45-PTB	30" Carbon Steel for Main Steam Pipe to Elbow	
30-CS-X-.908-46-PTB	46-PTB	30" Main Steam Pipe	
30-CS-X-1.5-47-PTB	47-PTB	30" Main Steam Pipe to Cap	
FW-NOZ.EX-CS-48-PTB	48-PTB	Feedwater Nozzle	
PL-SS-.375-49-PTB	49-PTB	Stainless Steel Plate. Block Waste Holdup Tank	
6-SS-X-1.1-50-PTB	50-PTB	Safe End	
10-SS-140-1.00-51-PTB	51-PTB	10" Stainless Steel Pipe	Replaced 15-PTB
6-SS-80-.432-52-PTB	52-PTB	6" Stainless Steel Pipe	
3.5-.75-8-CS-53-PTB	53-PTB	Reactor Coolant Pump Stud	Replaced 9-PTB
3.5-.75-8-CS-54-PTB	54-PTB	Reactor Coolant Pump Stud	Qualification Block
6-1-8-CS-55-PTB	55-PTB	Reactor Pressure Vessel Stud	Qualification Block
3.5-.75-8-CS-56-PTB	56-PTB	Reactor Coolant Pump Stud	Full Length stud
SG-FW-NOZZ-57-PTB	57-PTB	Feedwater Nozzle	
IR-CS-58-PTB	58-PTB	Feedwater Nozzle Inner Radius	
SI-NOZ-MOCKUP-59-PTB	59-PTB	Safety Injection Nozzle	Nozzle Mockup block

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Calibration Block Number	Brief ID Number	Application	Remarks
IN/OUT-NOZ-MOCKUP-60-PTB	60-PTB	Inlet and Outlet Nozzle	Nozzle Mockup block
RHE-SS-61-PTB	61-PTB	Regenerative Heat Exchanger nozzles	Nozzle Mockup block
RHE-SS-62-PTB	62-PTB	Regenerative Heat Exchanger nozzles	Nozzle Mockup block
6148E57 H01	RW46	Reactor Coolant Nozzle Inner Radius	Replacement Steam Generators Unit 2
9739D02 H09	SD41	Steam Generator Secondary Side	Replacement Steam Generators Unit 2
6148E09 H01	SD77	Feedwater Nozzle Inner Radius	Replacement Steam Generators Unit 2
9739D03 H04	SD78	Steam Generator Channel Head to Tubesheet	Replacement Steam Generators Unit 2
9741D03 H02	SW53	Reactor Coolant Nozzle to Safe-End	Replacement Steam Generators Unit 2
9739D02 H08	SD40	N/A	

# Appendix D

## Relief Requests

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
 PROGRAM

Relief Request	Description	Status
1	Altering the Date of the Start of the Fourth Inservice Inspection Interval	Authorized on June 18, 2001
2	Use of Later Code Editions	Granted on November 6, 2001
3	Risk Informed Examination of Class 1 and Class 2 Piping Butt Welds (Code Case N-578 and EPRI TR-112657)	Submitted under separate cover
4	Use of Code Case N-532-1, Alternative Requirements to Repair and Replacement Documentation Requirements and Inservice Inspection Summary Report Preparation and Submission as Required by IWA-4000 and IWA-6000	Submitted
5	Use of Code Case N-533-1, Alternative Requirements for VT-2 Visual Examination of Class 1, 2, and 3 Insulated Pressure-Retaining Bolted Connections	Submitted
6	Use of Code Case N-566-1, Corrective Action for Leakage Identified at Bolted Connections	Submitted
7	Use of Code Case N-616, Alternative Requirements for VT-2 Visual Examination of Classes 1, 2, and 3 Insulated Pressure Retaining Bolted Connections, Section XI, Division 1	Submitted
8	Use of Code Case N-624, Successive Inspections	Submitted
9	Alternative to Welding and Brazing Performance Qualification Requirements	Submitted
10	Relief from Regenerative Heat Exchanger Examinations	Submitted
11	Emergency Diesel System VT-2 Examinations	Submitted

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**Relief Request No. 1, Altering the Date of the Start of the Fourth Inservice Inspection Interval**

This relief request was sent in prior to the beginning of the fourth interval and was approved as Relief Requests RR-1-23 and RR-2-29. It was authorized on June 18, 2001, and is included here for completeness.

Pursuant to 10 CFR 50.55a(a)(3)(i), Point Beach Nuclear Plant requests an alternative to the Section XI Code required IWA-2430(b) requirement for the determination of the fourth and successive inspection intervals as specified in the ASME Section XI, 1995 Edition with 1996 Addenda, and the similar requirements of 10 CFR 50.55a(f)(4)(ii) and 10 CFR 50.55a(g)(4)(ii). Adopting the alternative requirements will enhance the quality and safety of the plants.

Background

Point Beach Nuclear Plant (PBNP) Unit 1 and Unit 2 commercial service dates are December 21, 1970, and October 1, 1972, respectively. These dates established the inspection intervals for Inservice Inspection (ISI) and Inservice Testing (IST), and are 650 days apart. PBNP desires to align these dates so both Units are on the same schedule and working to the same rules.

PBNP has recently written two (2) letters to inform the NRC of the need to alter the existing Interval dates. These letters are:

- NPL 99-0473, dated 8/27/1999, ASME Section XI ISI Long Term Plan Extension.
- NPL 2000-0141, dated 3/21/2000, Inservice Testing (IST) Program Interval Extension.

This relief will supercede the two previous letters and modify the fourth and subsequent interval dates. This is to align the Interval start dates so applicable Section XI ISI and IST rules are the same for both Units.

Plants for Which Relief is Requested

Point Beach Nuclear Plant Unit 1  
Point Beach Nuclear Plant Unit 2

Code of Federal Regulation and ASME Section XI Requirements

10 CFR 50.55a(b)(2) - As used in this section, references to Section XI of the ASME Boiler and Pressure Vessel Code refer to Section XI, Division 1, and include editions through the 1995 Edition and Addenda through the 1996 Addenda.

10 CFR 50.55a(f)(4)(ii) - Inservice tests to verify operational readiness of pumps and valves, whose function is required for safety, conducted during successive 120-month intervals must comply with the requirements of the latest edition and addenda of the Code incorporated by reference in paragraph (b) of this section 12 months prior to the start of the 120-month interval, subject to the limitations and modifications listed in paragraph (b) of this section.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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10 CFR 50.55a(g)(4)(ii) - Inservice examination of components and system pressure tests conducted during successive 120-month inspection intervals must comply with the requirements of the latest edition and addenda of the Code incorporated by reference in paragraph (b) of this section 12 months prior to the start of the 120-month inspection interval, subject to the limitations and modifications listed in paragraph (b) of this section.

ASME Section XI, 1995 Edition 1996 Addenda - IWA-2430(b) Inspection Intervals - The inspection interval shall be determined by calendar years following placement of the plant into commercial service.

ASME Section XI, 1995 Edition 1996 Addenda - IWA-2432 - Successive Inspection Intervals - 10 years following the previous inspection interval.

ASME Section XI, 1995 Edition 1996 Addenda

Table IWB-2412-1, Inspection Program B

Table IWC-2412-1, Inspection Program B

Table IWD-2412-1, Inspection Program B

Table IWE-2412-1, Inspection Program B

Table IWF-2410-1, Inspection Program B

Relief Requested

Relief is requested from updating the ISI and IST Programs on the timetable required by ASME Section XI and the Code of Federal Regulations. PBNP requests the start date for the fourth interval ISI and IST Programs be the same for both Units.

Basis for Relief

PBNP Units 1 and 2 began commercial operation on December 21, 1970, and October 1, 1972, respectively. These dates are almost two years apart (650 days), and since the ISI and IST Programs are linked to the commercial operating dates, the Editions of ASME Section XI effective during the 650 days between the Unit updates is different. When the next update becomes effective, PBNP Unit 1 will be working to the ASME Section XI 1995 Edition with the 1996 Addenda (95A96), while PBNP Unit 2 will still be working to the 1986 Edition. Updating to a new Code edition is a learning process requiring a tremendous amount of work on the part of many personnel to ensure compliance, and requires the updating of numerous documents and procedures.

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Having both Units on the same edition of ASME Section XI and schedule has distinct advantages. There are fewer procedures to maintain, and all procedures will be meeting the requirements of one Code, instead of trying to comply with two Codes, or having separate procedures. The ISI and IST Programs can be written as documents covering both Units, reducing administrative burden. This also reduces the chance of someone referring to the wrong Unit and performing examinations to the wrong requirements.

A review of the outages was performed for PBNP Units 1 and 2. Unit 1 had two outages lasting six continuous months or more. The first extended outage for PBNP Unit 1 occurred during the steam generator replacement project in 1983/84 and was in the second inspection interval. The outage lasted 6 months and 8 days (189 days total). At that time, PBNP did not take advantage of the Section XI Code (1977 Edition, Summer 1979 Addenda, IWA-2430[e]) that allowed plants out of service continuously for 6 months or more to extend the interval.

The second extended outage for PBNP Unit 1 occurred in 1997 (third inspection interval), and lasted for 10 months and 18 days (292 days total). PBNP has elected to take advantage of the IWA-2430(e) allowance to extend the interval. With the extended interval, the PBNP Unit 1 outages have remained in their initially scheduled periods, and no differences in percentages completed have occurred.

On PBNP Unit 2, an outage of 315 days occurred during the 1996/97 Steam Generator replacement. PBNP elected to not use the IWA-2430(e) interval extension allowed by the 1986 Code. By using the Unit 1 extension, and not using the Unit 2 extension, this reduced the gap between the Unit's interval dates.

The last outage for the third interval for PBNP Unit 1 will end approximately July 15, 2001. In order to meet the Program B percentage requirements, PBNP will use the 292-day interval extension allowed by IWA-2430(e) from the 1997 outage, extending the interval end date from December 20, 2000 to October 21, 2001. Third interval examinations should be completed by the end of July 2001. PBNP proposes further extending the third interval for PBNP Unit 1 to December 31, 2001. The next PBNP Unit 1 outage, which will be the first for the fourth interval, is scheduled for December 2002 and would be one year after the fourth interval begins. Extending the interval until the end of 2001 will have no effect on the number of examinations completed for the third interval, and no effect on the fourth interval ISI and IST plans and schedules.

The third interval for Unit 2 is scheduled to end on September 30, 2002. The proposed alternative will reduce the fourth interval by 9 months, to December 31, 2001. This will mean the ISI/IST examinations for the last outage of the PBNP Unit 2 third interval, which is scheduled for April/May, 2002, will be performed after the third interval end date. In order to meet the Program B requirements, PBNP will use approximately a 6-month interval extension as allowed by IWA-2430(d) and IWB-2412(b).

Altering the start of the intervals for both Units will allow them to begin on the same date. All Non-Destructive Examination (NDE) procedures will be updated to the 95A96 Edition of Section XI at that time. The procedures used for the PBNP Unit 2 April/May 2002 outage will be to the new Edition of the Code (fourth interval requirements), but the examinations will be counted for the third interval. This proposed schedule for PBNP Unit 2 would cause the NDE to be performed to procedures incorporating the latest approved Code requirements, which will enhance safety.

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In no case will PBNP credit the same examination for both intervals. Examinations for PBNP Unit 1 should be completed by the middle of 2001. The examinations for the April/May 2002 PBNP Unit 2 outage will be credited for the third interval. If any examinations are counted for the fourth interval, there will be no credit for that examination in the third interval.

With the proposed alternative schedule, the required fourth interval examinations will be completed by the end of the interval. There will be no reduction in the number of examinations for either Unit due to the date change.

PBNP is currently evaluating the options for implementing a Risk Informed Inservice Inspection (RI ISI) Program. The RI ISI Program update will begin January 1, 2001, which coincides with the proposed regular ISI Program update. This program is scheduled to commence on the first outage of the fourth interval for both Units.

Attachment A shows a timeline of intervals, periods, and outages as they were performed and those currently scheduled for each Unit and the proposed alternative.

Alternate Requirement

PBNP Units 1 and 2 will update the ISI and IST Programs to the ASME Section XI Code referenced on January 1, 2001, in 10 CFR 50.55a. It is anticipated this will be the 1995 Edition with the 1996 Addenda. The updated ISI/IST Programs will become effective January 1, 2002. NDE procedures will be updated to the 1995 Edition of Section XI with the 1996 Addenda, effective July 1, 2002. Any examinations performed after this date will be to the new procedures.

Justification of Relief

By altering the fourth interval start dates so both PBNP Units begin at the same time, the administrative costs and burden of complying with two different editions of Section XI are reduced. The possibility of working to the wrong procedures or documents is also reduced as they would all be to the latest edition of Section XI approved for use by the NRC. The proposed start date will require that some examinations performed on PBNP Unit 2 under the rules of the 1986 Edition of Section XI will now be performed to a later approved edition. This would enhance the quality of the examination and the safety of the plant.

Implementation Schedule

The ISI and IST Programs will be updated and become effective on January 1, 2002. The new interval schedule will remain in effect for the fourth and subsequent Inservice Inspection and Inservice Testing intervals.

Attachment

None

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Status

Authorized on June 18, 2001. The fourth interval will begin on July 1, 2002, for both Units.

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**Relief Request No. 2, Use of Later Code Editions**

This relief request was sent in prior to the beginning of the fourth interval and was approved as Relief Requests RR-1-24 and RR-2-30. It was granted on November 6, 2001, and is included here for completeness.

Plants for Which Exemption is Requested

Pt. Beach Nuclear Power Plant Unit 1, Docket No. 50-266  
Commercial Service Date: December 21, 1970

Pt. Beach Nuclear Power Plant Unit 2, Docket No. 50-301  
Commercial Service Date: October 1, 1972

Code of Federal Regulation Requirements

10 CFR 50.55a(b)(2) - As used in this section, references to Section XI of the ASME Boiler and Pressure Vessel Code refer to Section XI, Division 1, and include editions through the 1995 Edition and Addenda through the 1996 Addenda.

10 CFR 50.55a(g)(4)(ii) - Inservice examination of components and system pressure tests conducted during successive 120-month inspection intervals must comply with the requirements of the latest edition and addenda of the Code incorporated by reference in paragraph (b) of this section 12 months prior to the start of the 120-month inspection interval, subject to the limitations and modifications listed in paragraph (b) of this section.

Exemption Requested

An exemption is requested from using the 1995 Edition of ASME Section XI with Addenda through 1996 for the fourth inspection interval.

This exemption will not be applicable to the IWE/IWL Containment ISI Program or the Inservice Testing Program.

Basis for Exemption

ASME Section XI is an industry consensus document written by industry representatives with NRC involvement. It is constantly evolving to incorporate the latest knowledge and experience of the nuclear industry. This knowledge is gathered by a large range of experts in various technical fields, many of who are able to test some of the ideas in a controlled environment and/or gather data from sources within the nuclear power and similar industries. The information is then thoroughly discussed to determine its validity to Inservice Inspection. Analytical analysis is performed on many of the ideas to test their validity.

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Peer review is performed during the ASME meetings (and often between committee members outside the conferences) to discuss what improvements are needed and if value can be added to all proposed changes. Existing requirements are revisited to determine if they are still valid and if they should be improved, strengthened, or eliminated.

Industry events are brought to the ASME meetings and discussed. This has often led to the incorporation of additional requirements to ensure the focus of Inservice Inspection is appropriate. This has also refocused Inservice Inspection requirements to the areas where they are most effective.

The latest editions and addenda of Section XI incorporate all the improvements as approved by the ASME. Many of the Code Cases the NRC has approved have been incorporated. These improvements add value to the ISI process in that they continually refocus the industry on the problem areas and evaluate new developments in technology.

Alternate Requirement

PBNP Units 1 and 2 will update the ISI Programs for both Units to the ASME Section XI Code, 1998 Edition of Section XI with the 2000 Addenda. The updated ISI Programs will be effective for the fourth interval.

NDE procedures will be updated to the requirements of the selected Edition of Section XI, effective for the fourth interval. Any examinations performed after the ISI Program update would be to the new procedures.

PBNP will implement the Appendix VIII supplements of the 1998 Edition of Section XI with Addenda through 2000 on the schedule as determined in 10 CFR 50.55a(g)(6)(ii)(C)(1).

If the 1998 Edition of Section XI with Addenda through 2000 is referenced in a future revision of 10 CFR 50.55a, PBNP will follow any restrictions placed upon this edition and addenda.

Update for Request for Additional Information.

**Question:** NMC is requested to indicate whether it will commit to comply with the applicable limitations and modifications set forth in the staff's published position, until 10 CFR 50.55a is revised.

**Response:** The NMC will commit to complying with the limitations and modifications set forth in the NRC staff's published position, which was made public on August 3, 2001, until 10 CFR 50.55a is revised. If there are specific exceptions PBNP wishes to take, a relief request will be submitted in accordance with 10 CFR 50.55a(3).

**Question:** NMC is requested to indicate how its application of the above Code, along with the staff's proposed limitations and modifications, would provide an acceptable level of quality and safety, as per the requirements of 10 CFR 50.55a(a)(3)(i).

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**Response:** The 1998 Edition of Section XI with Addenda through 2000 was the latest published version of Section XI at the time this relief request was initially submitted. This edition and addenda of Section XI incorporated the improvements that have been studied and tested since the 1995 Edition of Section XI with Addenda through 1996 which was approved for use by the NRC. There were areas in the 1995 Edition with Addenda through 1996 needing to be strengthened, so limitations and modifications were suggested in the proposed rule. These limitations and modifications require additional examinations to ensure components are appropriately addressed when inadequate analysis was provided for change in requirements.

Therefore, the use of the 1998 Edition of Section XI with Addenda through 2000, along with the modifications and limitations in the proposed rule, ensures the Inservice Inspection requirements at PBNP adequately address all areas of concern and will provide an acceptable level of quality and safety.

Justification of Exemption

Updating to the 1998 Edition with Addenda through 2000 or later Edition of ASME Section XI will allow PBNP to use the latest improvements made to the Code as of the writing of this relief. The 1998 Edition with Addenda through 2000 has been announced as the possible replacement for the currently referenced 1995 Edition with 1996 Addenda of Section XI in the Code of Federal Regulations. This is anticipated to become effective after the date PBNP selects the edition/addenda from the Code of Federal Regulations.

Implementation Schedule

The update to the 1998 Edition with Addenda through 2000 or later Edition of ASME Section XI will be effective for the PBNP Units 1 and 2 fourth 10-year inspection intervals, which are scheduled to begin on July 1, 2002.

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**Relief Request No. 3, Risk Informed Examination of Class 1 and Class 2 Piping Welds  
(Code Case N-578 and EPRI TR-112657)**

This relief request will be submitted under separate cover.

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**Relief Request No. 4, Use of Code Case N-532-1**

Pursuant to 10 CFR 50.55a(3)(i), PBNP requests an alternative to the Code required IWA-6000 reporting requirements as specified in the 1998 Edition of ASME Section XI with Addenda through 2000.

Identification

Point Beach Units 1 and 2

NIS-1 and NIS-2 reports submitted at the end of each refueling outage.

Code Requirement

Rules for Inservice Inspection of Nuclear Power Plant Components, Section XI, 1998 Edition with Addenda through 2000.

Article IWA-6000, Records and Reports

IWA-6210(c) The Owner shall prepare Preservice and Inservice Inspection summary reports for Class 1 and 2 pressure retaining components and their supports. A cover sheet containing the information of IWA-6230(d) shall be provided.

IWA-6210(d) The Owner shall prepare the Owner's Report for Inservice Inspections, Form NIS-1, for preservice and inservice examination of Class 1 and 2 pressure retaining components and their supports.

IWA-6210(e) The Owner shall prepare the Owner's Report for Repair/Replacement Activities, Form NIS-2.

IWA-6240 Summary reports shall be submitted to the regulatory and enforcement authorities having jurisdiction at the plant site.

Relief Requested

PBNP requests relief from preparing the NIS-1 and NIS-2 forms for reporting the results of Inservice Inspection activities after each outage as prescribed by Section XI, IWA-6000, Records and Reports. PBNP will utilize the alternative requirements in ASME Code Case N-532-1, Alternative Requirements to Repair and Replacement Documentation Requirements and Inservice Summary Report Preparation and Submission as Required by IWA-4000 and IWA-6000, Section XI, Division 1.

Basis for Relief

The documentation required by IWA-6000 includes the NIS-1 and NIS-2 forms. These forms give a summary of Class 1 and 2 examinations. They include what components and piping systems were examined. They do not require the inclusion of Class 3 components or the new IWE and IWL examinations.

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The information included on the current NIS-1 is verified by the ANII on an examination by examination basis, and reviewed by several other groups. These may include QA auditors and the USNRC. The NIS-1 is a narrative account of the ISI activities for an outage, and does not provide information showing Code requirements were met.

The NIS-2 reports on repairs and replacements list the components where work has been performed. Again, the ANII reviews and verifies each report. NRC regional inspectors review some of these reports during refuel outages. Sending these reports to the regulatory agencies does not provide any detail about the repair or replacement.

Code Case N-532-1 is an alternative to the documentation requirements of IWA-6000 of Section XI. Use of this alternative reduces the documentation gathered and submitted each outage. The forms in the Code Case enhance the type of information submitted, being more specific in the number of examinations performed and what types of problems were encountered. PBNP will use this Code Case to replace the NIS-1 and NIS-2 reports.

The current requirement is to submit the NIS-1 and NIS-2 summary reports to the enforcement and regulatory authorities having jurisdiction at the plant site within 90 days of the end of the outage. The NIS-1 report gives a summary of the types of examinations performed and the results. The NIS-2 reports give details on Repairs and Replacements. Code Case N-532-1 allows the use of the OAR-1 form and three tables of specific examination information and the NIS-2A forms for repairs and replacements. By using this Code Case, PBNP will provide more specific information about the status of examinations. This will reduce the amount of time and costs involved in collection of the data, and provide more specific information to enforcement and regulatory agencies that may audit the plant records.

Code Case N-532-1 requires specific Program B information to be submitted. The preparation of the Code required documentation involves a significant effort in man-hours and associated costs. The information provided with the NIS-1 and NIS-2 forms do not provide any supporting evidence of the status of examinations performed.

The NRC has published Draft Regulatory Guide DG-1091, which includes an acceptance of ASME Code Case N-532. The only restriction the NRC is planning on imposing is to submit the OAR-1 form within 90 days after a refueling outage. PBNP will follow this guidance with the use of Code Case N-532-1. There were two differences between N-532 and N-532-1. The first is the later Code Case references the 1998 Edition with Addenda through 2000, which PBNP will be using for the fourth interval. The second is the addition of Table 4, which shows how to reference previous editions and addenda of Section XI. Table 4 will not be applicable to PBNP.

ASME approved Code Case N-532-1 for use on March 28, 2001, as an alternative to the Code required documentation of IWA-6000. Use of this Code Case provides an acceptable level of quality and safety by the use of alternate documentation. This enhances the process of determining if PBNP has met the Program B requirements of Section XI and the elimination of verbiage not adding to the quality and safety of the plant.

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Alternative Documentation and Requirements

PBNP will prepare OAR-1 and NIS-2A forms in accordance with the requirements of Code Case N-532-1. The OAR-1 and NIS-2A forms will be filed in accordance with the requirements of IWA-6000 on site for review by enforcement and regulatory authorities having jurisdiction at the plant site.

For Table 1 of the Code Case, PBNP will provide the required information. On those Code Categories that are covered by requirements in addition to Section XI (e.g., Code Category B-Q for steam generator tubes), a note will be placed in the table explaining the details.

For Table 2, PBNP will list flaws or relevant conditions requiring evaluation for continued service in accordance with the following sections:

- IWB-3132.3, Acceptance by Analytical Evaluation
- IWB-3142.4, Acceptance by Analytical Evaluation
- IWC-3122.3, Acceptance by Evaluation
- IWC-3132.3, Acceptance by Evaluation
- IWD-3000, Acceptance Standards (the rules of IWB-3000 will be used)
- IWF-3122.3, Acceptance by Evaluation or Test

For component supports, if an evaluation or test accepts a relevant condition, and the condition will be left in place, this will be reported on the submittal to the regulatory and enforcement agencies having jurisdiction at the plant site.

For Table 3, Repairs, Replacements, or Corrective Measures that were required for continued service since the previous OAR-1 Form (or NIS-1 form) was filed, will be included.

This relief will not be applicable to IWE and IWL Inservice Inspection examinations.

Implementation Schedule

PBNP will implement ASME Code Case N-532-1 during the PBNP fourth 10-year Inservice Inspection interval. If this Code Case is published in a future revision of Regulatory Guide 1.147, and PBNP intends to continue implementation, limitations issued in the Regulatory Guide will be implemented.

Attachments

Code Case N-532-1

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Status

Submitted.

CASE N-532-1  
CASES OF ASME BOILER AND PRESSURE VESSEL CODE

Approval Date: March 28, 2001

See Numeric Index for expiration and any reaffirmation dates.

Alternative Requirements to Repair and Replacement Documentation Requirements and Inservice Summary Report Preparation and Submission

Section XI, Division 1

**Inquiry:** What alternatives may be used to the requirements of IWA-4180(d) and IWA-6210(e) for completion of Form NIS-2 following repair or replacement, and IWA-6210(c) and (d), IWA-6220, IWA-6230(b), (c), and (d), and IWA-6240(b) for preparation and submittal of the inservice summary report and Form NIS-1<sup>1</sup>?

**Reply:** It is the opinion of the Committee that as an alternative to the requirements of IWA-4180(d), IWA-6210(c), (d), and (e), IWA-6220, IWA-6230(b), (c), and (d), and IWA-6240(b), the following provisions may be used. This Case shall be utilized at least until the end of the inspection period in which it was invoked.

1.0 CERTIFICATION OF THE REPAIR OR REPLACEMENT

- (a) The Owner's Repair/Replacement Program shall identify use of this Case.
- (b) A Repair/Replacement Plan shall be prepared in accordance with IWA-4150, and shall be given a unique identification number.
- (c) Upon completion of all required activities as associated with the Repair/Replacement Plan, the Owner shall prepare a REPAIR/REPLACEMENT CERTIFICATION RECORD, FORM NIS-2A.
- (d) Form NIS-2A shall be presented to the Inspector for certification.

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<sup>1</sup>All references to IWA-4000 and IWA-6000 used in this Case refer to the 1998 Edition with the 2000 Addenda. For use with other Editions and Addenda of Section XI, refer to Table 4 for applicable Section XI references. The term Repair/Replacement contained in the Case replaces those terms previously known as "Repair", "Replacement", and "Modification" in earlier Editions and Addenda of Section XI.

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- (e) The completed Form NIS-2A shall be maintained by the Owner.
- (f) The Owner shall maintain an index of Repair/Replacement Plans in accordance with IWA-6340. The index shall identify the identification number required by (b) above and the inspection interval and period during which each repair or replacement was completed.

## 2.0 OWNER'S ACTIVITY REPORT PREPARATION AND SUBMITTAL

An OWNER'S ACTIVITY REPORT FORM OAR-1 shall be prepared and certified upon completion of each refueling outage. Each Form OAR-1 prepared during an inspection period shall be submitted following the end of the inspection period. Each Form OAR-1 shall contain the following:

- (a) Abstract of applicable examinations and tests with the information and format of Table 1.
- (b) A listing of item(s) with flaws or relevant conditions that required evaluation to determine acceptability for continued service, whether or not the flaw or relevant condition was discovered during a scheduled examination or test. The listing shall provide the information in the format of Table 2.
- (c) Abstract for repairs, replacements and corrective measures performed, which were required due to an item containing a flaw or relevant condition that exceeded IWB-3000, IWC-3000, IWD-3000, IWE-3000, IWF-3000, or IWL-3000 acceptance criteria; even though the discovery of the flaw or relevant condition that necessitated the repair, replacement or corrective measure, may not have resulted from an examination or test required by this Division. If acceptance criteria for a particular item is not specified in this Division, the provisions of IWA-3100(b) shall be used to determine which repairs, replacements, and corrective measures are required to be included in the abstract. The abstract shall provide the information in the format of Table 3.

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FORM NIS-2A REPAIR/REPLACEMENT CERTIFICATION RECORD  
OWNER'S CERTIFICATE OF CONFORMANCE

I certify the \_\_\_\_\_ represent by Repair/Replacement Plan  
repair or replacement

Number \_\_\_\_\_ conforms to the requirements of Section XI.

Type Code Symbol Stamp \_\_\_\_\_

Certificate of Authorization No. \_\_\_\_\_ Expiration Date \_\_\_\_\_

Signed \_\_\_\_\_ Date \_\_\_\_\_  
Owner of Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of \_\_\_\_\_ and employed by \_\_\_\_\_ of \_\_\_\_\_ have inspected the items described in Repair/Replacement Plan number \_\_\_\_\_ during the period \_\_\_\_\_ to \_\_\_\_\_, and state that to the best of my knowledge and belief, the Owner has performed all the activities described in the Repair/Replacement Plan in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the activities described in the Repair/Replacement Plan. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

\_\_\_\_\_  
Inspector's Signature                      Commissions \_\_\_\_\_  
National Board, State, Province, and Endorsements

Date \_\_\_\_\_

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FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number \_\_\_\_\_  
Owner \_\_\_\_\_  
(Name and Address of Owner)  
Plant \_\_\_\_\_  
(Name and Address of Plant)  
Unit No. \_\_\_\_\_ Commercial Service Date \_\_\_\_\_ Refueling Outage No. \_\_\_\_\_  
(If applicable)  
Current Inspection Interval \_\_\_\_\_  
(1st, 2nd, 3rd, 4th, other)  
Current Inspection Period \_\_\_\_\_  
(1st, 2nd, 3rd)  
Edition and Addenda of Section XI applicable to the inspection plan \_\_\_\_\_  
Date and Revision of inspection plan \_\_\_\_\_  
Edition and Addenda of Section XI applicable to repairs and replacements, if different than the inspection plan  
\_\_\_\_\_

CERTIFICATE OF CONFORMANCE

I certify that the statements made in the Owner's Activity Report are correct, and that the examinations, tests, repairs, replacements, evaluations, and corrective measures represented by this report conform to the requirements of Section XI.

Certificate of Authorization No. \_\_\_\_\_ Expiration Date \_\_\_\_\_  
(If applicable)

Signed \_\_\_\_\_ Date \_\_\_\_\_  
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and the State or Province of \_\_\_\_\_ and employed by \_\_\_\_\_ of \_\_\_\_\_ have inspected the items described in this Owner's Activity Report, during the period \_\_\_\_\_ to \_\_\_\_\_, and state that to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the examinations, tests, repairs, replacements, evaluations, and corrective measures described in this report. Furthermore, neither the inspector nor his employer shall be liable in any manner for any personal injury or property damage or loss of any kind arising from or connected with this inspection.

Inspector's Signature \_\_\_\_\_ Commissions \_\_\_\_\_  
National Board, State, Province, and Endorsements

Date \_\_\_\_\_

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TABLE 1  
ABSTRACT OF EXAMINATIONS AND TESTS

Examination Category	Total Examinations Required for the Interval	Total Examinations Credited for this Period	Total Examinations Credited (%) for the Period	Total Examinations Credited (%) to Date for the Interval	Remarks
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TABLE 2  
ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT  
REQUIRED EVALUATION FOR CONTINUED SERVICE

Examination Category	Item Number	Item Description	Flaw Characterization (IWA-3300)	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes or No)
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TABLE 3  
ABSTRACT OF REPAIRS, REPLACEMENTS, OR CORRECTIVE MEASURES  
REQUIRED FOR CONTINUED SERVICE

Code Class	Repair, Replacement, or Corrective Measure	Item Description	Description of Work	Flaw or Relevant Condition Found During Scheduled Section XI Examination or Test (Yes or No)	Date Complete	Repair/ Replacement Plan Number
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**TABLE 4**  
**PARAGRAPH NUMBER CROSS REFERENCE FOR USE WITH EARLIER EDITIONS AND**  
**ADDENDA**

1998 Edition with the 2000 Addenda (Code Case References)	1983 Winter Addenda through 1989 Addenda	1990 Addenda	1991 Addenda through 1995 Edition	1995 Addenda through 2000 Addenda
IWA-4150	IWA-4130, IWA-7130	IWA-4130, IWA-7130	IWA-4140	IWA-4150
IWA-4180(d)	IWA-7520(a)(8)	IWA-7520(a)(8)	IWA-4910(d)	IWA-4180(d)
IWA-6210(c)	IWA-6210(c)	IWA-6210(c)	IWA-6210(c)	IWA-6210(c)
IWA-6210(d)	IWA-6220(b)(10)	IWA-6210(d)	IWA-6210(d)	IWA-6210(d)
IWA-6210(e)	IWA-6220(b)(10)	IWA-6210(e)	IWA-6210(e)	IWA-6210(e)
IWA-6220	IWA-6220	IWA-6220	IWA-6220	IWA-6220
IWA-6230(b)	IWA-6220(c)	IWA-6230(b)	IWA-6230(b)	IWA-6230(b)
IWA-6230(c)	IWA-6220(d)	IWA-6230(c)	IWA-6230(c)	IWA-6230(c)
IWA-6230(d)	IWA-6220(e)	IWA-6230(d)	IWA-6230(d)	IWA-6230(d)
IWA-6240(b)	IWA-6230	IWA-6240(b)	IWA-6240(b)	IWA-6240(b)

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**Relief Request No. 5, Use of Code Case N-533-1**

Pursuant to 10 CFR 50.55a(a)(3)(i), PBNP requests an alternative to the Code required IWA-5242 VT-2 examination requirements for bolted connections as specified in the 1998 Edition of ASME Section XI with Addenda through 2000.

Component Identification

Bolted connections on Class 1, 2, and 3 systems borted for the purpose of controlling reactivity.

Code Requirement

Rules for Inservice Inspection of Nuclear Power Plant Components, Section XI, 1998 Edition with Addenda through 2000.

Article IWA-5242(a) - For systems borted for the purpose of controlling reactivity, insulation shall be removed from pressure retaining bolted connection for visual examination VT-2.

Relief Requested

PBNP requests relief from performing VT-2 examinations on bolted connections on systems borted for controlling reactivity during system pressure tests as prescribed by Section XI, Article IWA-5242(a).

Basis for Relief

The ambient conditions during the installation of insulation after VT-2 examinations at normal operating pressure and temperature (NOP/NOT) require heat stress work restrictions. Containment entries at NOP/NOT are physically demanding on personnel due to the adverse heat environment. Stay times for personnel in many areas are less than one hour and would require multiple containment entries to complete the examination activities. Ambient temperatures range from 95 to 110 degrees F. Personnel should not be exposed to such an adverse work environment unnecessarily without a compensating increase in the level of quality and safety. Performing the VT-2 visual examination using Code Case N-533 will accomplish the examinations and the insulation installation while maintaining personnel safety and examination quality at an appropriate level.

Historical data indicate that personnel contamination increase with increasing environmental temperatures due to the profuse sweating caused by the elevated temperatures. Reinstalling contaminated insulating material under adverse conditions (i.e., to piping that is at 2250 psia and greater than 500 degrees F) would negatively impact total personnel contamination and expose personnel to unnecessary safety risk. Additionally, increased dose would be accumulated due to reduced examination efficiency as a result of the necessity to wear special protective equipment (e.g., ice vest).

The removal of scaffolding from containment would be through the reactor containment building personnel hatch rather than the equipment hatch since the plant is in Mode 4 with the equipment hatch secured. This will place added physical and heat stress limitations on the personnel involved.

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Code Case N-533-1 was approved for use by ASME on February 26, 1999, as an alternative to the Code requirements of IWA-5242(a). The Code Case was written when it was recognized that personnel safety during the conduct of the VT-2 examinations would be compromised, and that examination of bolted connections during plant shutdown would accomplish the desired results. In addition, evaluation and repairs could be performed without the need to bring the plant to a cold shutdown condition since the VT-2 examinations are normally performed at normal operating pressure and temperature during startup.

Use of this Code Case provides an acceptable level of quality and safety by requiring Section XI examinations to be performed while a component is at safe operating pressures and temperatures. The examinations performed will find evidence of leakage by having the examiners looking for boric acid residue, which accumulates around leakage sites, or any other evidence of leakage. This Code Case meets the intent of Section XI IWA-5240 requirements by requiring PBNP to examine bolted connections for evidence of leakage and bolt degradation.

Alternate Examinations

PBNP will utilize the alternative requirements of ASME Code Case N-533-1, Alternative Requirements for VT-2 Visual Examination of Class 1 Insulated Pressure Retaining Bolted Connections, Section XI, Division 1.

Each refueling outage, PBNP will remove the insulation from the bolted connections and perform a VT-2 visual examination in accordance with paragraph (b) of the Code Case on Class 1 systems. For Class 2 and 3 systems, this same examination will be performed once each period. The connections will not be pressurized during the examination. Any evidence of leakage will be evaluated in accordance with IWA-5250.

In addition to the requirements of paragraph (a) of the Code Case, the system pressure test and VT-2 examination with the insulation installed on bolted joints at normal operating pressure and temperature will include a 4-hour hold time as required.

Implementation Schedule

PBNP will implement ASME Code Case N-533-1 during the fourth 10-year Inservice Inspection interval. If this Code Case is published in a future revision of Regulatory Guide 1.147, and PBNP intends to continue implementation, limitations issued in the Regulatory Guide will be implemented.

Attachments

Code Case N-533-1

IX Status

Submitted.

CASE N-533-1

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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CASES OF ASME BOILER AND PRESSURE VESSEL CODE

Approval Date: February 26, 1999

See Numeric Index for expiration and any reaffirmation dates.

Alternative Requirements for VT-2 Visual Examination of Class 1, 2, and 3 Insulated  
Pressure-Retaining Bolted Connections

Section XI, Division 1

**Inquiry:** What alternative requirements may be used in lieu of those of IWA-5242(a) to remove insulation from Class 1, 2, and 3 pressure-retaining bolted connections to perform a VT-2 visual examination?

**Reply:** It is the opinion of the Committee that, as an alternative to the requirements of IWA\_5242(a) to remove insulation from Class 1, 2, and 3 pressure-retaining bolted connections to perform a VT-2 visual examination, the following requirements shall be met.

- (a) A system pressure test and VT-2 visual examination shall be performed each refueling outage for Class 1 connections and each period for Class 2 and 3 connections without removal of insulation.
- (b) The insulation shall be removed from the bolted connection, each refueling outage for Class 1 connections and each period for Classes 2 and 3 connections, and a VT-2 visual examination shall be performed. The connection is not required to be pressurized. Any evidence of leakage shall be evaluated in accordance with IWA-5250.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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**Relief Request No. 6, Use of Code Case N-566-1, Corrective Action for Leakage at Bolted Connections**

Pursuant to 10 CFR 50.55a(a)(3)(i), PBNP requests an alternative to the Code required IWA-5250(a)(2) requirements for bolted connections when leakage is detected as specified in the 1998 Edition with Addenda through 2000 of ASME Section XI.

Component Identification

Point Beach Units 1 and 2

Class 1, 2, and 3 Pressure Retaining Bolted Connections

Corrective measures for leakage at bolted connections.

Examination Requirements

Rules for Inservice Inspection of Nuclear Power Plant Components, Section XI, 1998 Edition with Addenda through 2000

IWA-5250(a)(2) - If leakage occurs at a bolted connection, one of the bolts shall be removed, VT-3 visually examined for corrosion, and evaluated in accordance with IWA-3100. The bolt selected shall be the one closest to the source of leakage.

Relief Requested

Nuclear Management Company requests relief from removing and performing a VT-3 visual examination to the bolt closest to the source of leakage when leakage is observed at a bolted connection during a system pressure test.

Basis for Relief

The requirement to remove bolting from a bolted connection to check for degradation is a significant burden. This requirement does not take into account the corrosiveness of the fluid, the material of the leaking component, the type and location of the leakage, the accessibility of the area of concern, and the history of material degradation in a similar environment. Additional examinations are performed by system engineers and plant operations personnel during routine surveillance required by plant Technical Specifications and procedures. Previous corrective actions are not taken into account.

ASME Code interpretation XI-1-92-01 states that new bolting or bolting that has received a visual examination prior to installation and has not been inservice does not have to be evaluated in accordance with this section. This is recognition by the Code that leakage at this point would be considered a maintenance item, and one in which the requirements of IWA-5250(a)(2) do not apply.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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Removal of pressure retaining bolting at mechanical connections for VT-3 visual examination and subsequent evaluation in locations where leakage has been identified is not always the most prudent course of action to determine the acceptability of the bolting. This is a task that could cause a detrimental effect on the bolted connection. Many bolted connections are studs threaded into a component, such as valves and pumps. Removal of these studs can be difficult due to the length of time they have been installed. Large studs, such as those found on the Reactor Coolant Pumps, pose additional problems with removal. Damage to the pump casings can occur if too much torque is required to remove a stud. This could require disassembly of the pump in order to perform the examination, resulting in significant radiation exposure and the possibility of additional damage to the pump components.

The Code requirement to remove, examine, and evaluate bolting in this situation does not allow the Owner to consider other factors which may indicate the acceptability the bolting. PBNP considers this requirement to be unnecessarily prescriptive and restrictive.

There are other factors that should be considered when evaluating bolting acceptability. When leakage has been identified at a mechanical joint, an evaluation should include, but not limited to: joint bolting materials, service age of joint bolting materials, location of the leakage, history of leakage at the joint, evidence of corrosion with the joint assembled, and corrosiveness of process fluid.

ASME Section XI is written to primarily address examinations and testing during periods of plant or system shutdown. No guidance is given to address components that are examined or tested while the plant or system is in service. However, Class 2 and 3 systems are pressure tested while the plant is operating.

Performance of the test while the system is inservice may identify leakage at a bolted connection that, upon evaluation, may conclude that the joint's structural integrity and pressure retaining ability is not challenged. It would not be prudent to negatively impact a system's availability by removing the system from service to address a leak that does not challenge the system's ability to perform its safety function.

In addition, a situation frequently encountered at commercial nuclear plants such as Point Beach Nuclear Plant, is the complete replacement of bolting materials (studs, bolts, nuts, washers, etc.) at mechanical joints during plant outages. When the associated system process piping is pressurized during plant start-up, leakage is identified at these joints. The root cause of this leakage is most often due to thermal expansion of the piping and bolting materials at the joint and subsequent process fluid seepage at the joint gasket. Proper re-torquing of the joint bolting, in most cases, stops the leakage. Removal of any of the joint bolting to evaluate for corrosion would be unwarranted in this situation due to new condition of the bolting materials.

#### Alternative Examinations

When PBNP finds leakage at bolted connections by VT-2 visual examination during system pressure testing, the alternative rules of ASME Code Case N-566-1, Corrective Action for Leakage Identified at Bolted Connections, Section XI, Division 1, will be implemented.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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Implementation Schedule

PBNP will implement the alternative requirements of Code Case N-566-1 during the fourth 10-year Inservice Inspection interval.

Attachments

ASME Code Case N-566-1

Status

Submitted.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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CASE N-566-1

CASES OF ASME BOILER AND PRESSURE VESSEL CODE

Approval Date: February 15, 1999

See Numerical Index for expiration and any reaffirmation dates.

Case N-566-1

Corrective Action for Leakage at Bolted Connections, Section XI, Division 1

**Inquiry:** What alternative to the requirements of IWA-5250(a)(2) may be used when leakage is detected at bolted connections?

**Reply:** It is the opinion of the Committee that, as an alternative to the requirements of IWA-5250(a)(2) bolted connections, the requirements of (a) or (b) below shall be met:

- (a) The leakage shall be stopped, and the bolting and component material shall be evaluated for joint integrity as described in (c) below.
- (b) If the leakage is not stopped, the joint shall be evaluated in accordance with IWB-3142.4 for joint integrity. This evaluation shall include considerations listed in (c) below.
- (c) The evaluation of (a) and (b) above is to determine the susceptibility of the bolting to corrosion and failure. This evaluation shall include the following:
  - (1) The number and service age of the bolts;
  - (2) bolt and component material;
  - (3) corrosiveness of process fluid;
  - (4) leakage location and system function;
  - (5) leakage history at the connection or other system components;
  - (6) visual evidence of corrosion at the assembled connection.

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**Relief Request No. 7, Use of Code Case N-616, Alternative Requirements for VT-2 Visual Examination of Classes 1, 2, and 3 Insulated Pressure Retaining Bolted Connections**

Pursuant to 10 CFR 50.55a(a)(3)(i), PBNP requests an alternative to the requirements of IWA-5242(a) as specified in the 1998 Edition of ASME Section XI with Addenda through 2000.

Identification

Point Beach Nuclear Plant, Units 1 and 2

ASME Section XI, 1998 Edition with Addenda through 2000

Examination Categories B-G-1 and B-G-2

Examination Requirements

IWA-5242 Insulated Components

(a) For systems borated for the purpose of controlling reactivity, insulation shall be removed from pressure retaining bolted connections for the VT-2 visual examination.

Relief Requested

PBNP is requesting relief from removing the insulation at bolted connections in systems borated for the purpose of controlling reactivity as described in IWA-5242(a).

Basis for Relief

The ambient conditions during the installation of insulation after VT-2 examinations at normal operating pressure and temperature (NOP/NOT) require heat stress work restrictions. Containment entries at NOP/NOT are physically demanding on personnel due to the adverse heat environment. Stay times for personnel in many areas are less than one hour and would require multiple containment entries to complete the examination activities. Ambient temperatures range from 95 to 110 degrees F. Personnel should not be exposed to such an adverse work environment unnecessarily without a compensating increase in the level of quality and safety.

Historical data indicate that personnel contamination increase with increasing environmental temperatures due to the profuse sweating caused by the elevated temperatures. Reinstalling contaminated insulating material under adverse conditions (i.e., piping at 2250 psia and greater than 500 degrees F) would negatively impact total personnel contamination and exposes personnel to unnecessary safety risk. Increased dose is accumulated due to reduced examination efficiency as a result of the necessity to wear special protective equipment (e.g., ice vest).

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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PBNP has requested to use the alternative requirements of ASME Code Case N-533-1, which allows the removal of insulation when the system is not pressurized or at temperature. This increases the safety of the examiners, insulators, and any additional personnel who are needed for support during the examinations. By eliminating the requirement to remove insulation during the conduct of a pressure test, the test can be performed without the need of these additional personnel. This reduces the amount of radiation dose accumulated and the number of people performing the test or in support of it. This will reduce the chances of injury during the setup, performance of, and recovery from the test.

Code Case N-616 was approved for use by ASME on May 7, 1999, as an alternative to the Code requirements of IWA-5242(a). The Code Case was written when it was recognized that further reduction of personnel radiation dose and increased safety during the conduct of the VT-2 examinations could be enhanced when the bolting material is resistant to corrosion, thus eliminating the requirement to remove the insulation.

Use of this Code Case provides an acceptable level of quality and safety by requiring Section XI examinations to be performed in the same manner as the remainder of the pressure retaining components. The examinations performed will find evidence of leakage by looking at those areas where water accumulates around leakage sites. This Code Case meets the intent of Section XI IWA-5240 requirements by requiring PBNP to examine bolted connections by looking at the insulation and areas under the bolted connections for evidence of leakage.

Alternative Examinations

PBNP will use the alternative requirements of ASME Code Case N-616, Alternative Requirements for VT-2 Visual Examination of Classes 1, 2, and 3 Insulated Pressure Retaining Bolted Connections.

In addition to the requirements of Code Case N-616, PBNP will perform the following:

1. Insulation will be removed for VT-2 visual examination during the system pressure test for any 17-4 PH stainless steel or 410 stainless steel stud or bolt aged at a temperature below 1100 degrees F or with hardness above  $R_C$  30.
2. For A-286 stainless steel studs or bolts, the preload must be verified to below 100Ksi or the thermal insulation must be removed and the joint visually examined.
3. For nuts conforming to SA-194, removal of the insulation for visual examination will not be necessary.
4. A 4-hour hold time at operating temperature and pressure will be performed prior to conducting the VT-2 examination.

Implementation Schedule

PBNP will implement the alternative requirements of Code Case N-616 and the additional requirements as stated above during the fourth 10-year Inservice Inspection interval.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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Attachments

ASME Code Case N-616

Status

Submitted.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

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CASE N-616

CASES OF ASME BOILER AND PRESSURE VESSEL CODE

Approval Date: May 7, 1999

See Numerical Index for expiration and any reaffirmation dates.

Case N-616

Alternative Requirements for VT-2 Visual Examination of Classes 1, 2, and 3 Insulated Pressure Retaining Bolted Connections, Section XI, Division 1

**Inquiry:** What alternative to the requirements may be used in lieu of those of IWA-5250(a)(2) for removal of the insulation from Classes 1, 2, and 3 pressure retaining bolted connections to perform a VT-2 visual examination, when the bolting material is resistant to boric acid degradation?

**Reply:** It is the opinion of the Committee that when corrosive resistant bolting material that is used has a chromium content greater than or equal to 10%, such as SA-564 Grade 630 H1100, SA-452 Grade 660, SB-667 UNS N07718, or SB 637 UNS N07750, it is permissible to perform the VT-2 examination without insulation removal.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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**Relief Request No. 8, Use of Code Case N-624, Successive Inspections**

Pursuant to 10 CFR 50.55a(a)(3)(i), PBNP requests an alternative to the Code requirement for scheduling of components for examination as specified in the 1998 Edition of ASME Section XI with Addenda through 2000.

Identification

Class 1, 2, and 3 component examinations scheduled for the fourth interval.

Requirement

IWB-2420(a) - The sequence of component examinations which was established during the first inspection interval shall be repeated during each successive inspection interval, to the extent practical.

IWC-2420(a) - The sequence of component examinations which was established during the first inspection interval shall be repeated during each successive inspection interval, to the extent practical.

IWD-2420(a) - The sequence of component examinations which was established during the first inspection interval shall be repeated during each successive inspection interval, to the extent practical.

IWF-2420(a) - The sequence of component examinations which was established during the first inspection interval shall be repeated during each successive inspection interval, to the extent practical.

Relief Requested

Relief is requested from performing component examinations in the sequence that was established in the first inspection interval.

Basis for Relief

Examinations performed during the first two periods of the First Interval were in accordance with Technical Specifications, which were modeled on the 1970 Edition with Addenda through Summer 1971 of Section XI. The third period was performed to the 1974 Edition of Section XI with Addenda through Summer 1975 on all Code Classes (pro-rated for the Interval). With this schedule, the sequence of examinations was not established for about one-half of the areas until the second interval.

Modifying the sequence of examinations reduces the need for personnel to prepare and examine components in essentially the same area several times. Changing the sequence of examinations can significantly reduce the radiation exposure, time, and manpower required to perform these tasks. Point Beach has not had any unusual problems with piping and components, so modifying the ISI schedule would have no effect on the safe operation of the plant.

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Altering the sequence of examinations has proven to be an affective method of reducing radiation exposure and costs. By altering the sequence of examinations, St. Lucie and Turkey Point plants achieved reductions of over 50% in exposure, with significant cost savings and more efficient outage schedules. This type of exposure and cost reductions are possible at Point Beach with a schedule based on emphasizing performing examinations in close proximity to each other during one outage instead of during separate outages.

PBNP will be implementing a Risk Informed ISI Program for piping welds for the fourth interval. During the selection process, previously examined welds were scheduled to the extent practical. The examinations, however, could have been performed in any of the three previous intervals. There was a conscientious attempt to pick examination areas close together to eliminate as much scaffold requirements as possible. By scheduling in this manner, the costs and radiation dose were reduced even further.

Rescheduling ISI activities has been proven to lower radiation exposure, manpower, and costs associated with the preparation, examination, and recovery of the selected areas. This also reduces radiation exposure to other workers in the areas by eliminating barriers caused by scaffold and removed insulation, decreasing the amount of time required to perform other tasks.

PBNP realizes that the objective of the Code selection method is to examine components in all parts of the plant and to repeat those examinations on a regular basis to determine if changes are occurring. This philosophy was used when the selection and scheduling of Class 1 and 2 piping welds was performed.

Grouping of examinations was performed to eliminate duplicate scaffold and the removal of the same insulation multiple times to the extent practical. The groupings were scheduled around the piping butt welds selected for the Risk Informed Program, which has no guidance for examining the same welds, and to the extent practical, the same sequence as was previously scheduled. In fact, the Risk Informed ISI Program would specifically exclude the Code required selection criteria, as it is a living program designed to ensure the highest risk areas are receiving the greatest attention.

Since PBNP began performing ISI examinations, the rules for radiation exposure, safety, and the selection and scheduling of ISI examination areas have changed significantly. The changes were enacted to provide additional safety to personnel working at the plants and to enhance the safety to the general public. 10 CFR 20.1101(b) mandates PBNP to reduce radiation exposure to as low as reasonably achievable. Procedures and engineering controls based upon sound radiation protection principles are being used to the extent practicable. In order to meet this and other regulations, PBNP must reevaluate every aspect of every job at PBNP. Revising the ISI schedule will allow PBNP to minimize the amount of work being performed in radiation areas, meet safety and ALARA requirements, and still meet Section XI requirements.

By utilizing the selection criteria of the Code Case in conjunction with the Risk Informed ISI Program, this will ensure the Inservice Inspection requirements at PBNP adequately address all areas of concern and will provide an acceptable level of quality and safety.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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Alternate Requirements

PBNP proposes to use the alternative requirements of ASME Code Case N-624.

The sequence of examinations established during the previous inspection interval will be repeated to the extent practical, but will be modified in a manner that reduces scaffold, insulation, and radiation exposure. The examinations will be selected to coincide with the requirements of the Risk Informed ISI Program to the extent practical.

Point Beach Nuclear Plant will schedule the same areas for examination that were performed during the third interval to the extent practical. The sequence of examinations established during the third interval will be followed to the extent practical, but will be altered to reduce radiation exposure and expense, and allow the examination, preparation of areas, and the recovery process to be minimized. Substitute welds may be selected (the substitution of other components is defined in the ISI Program). When welds are substituted, they will be similar in configuration to those originally scheduled, and on the same or similar lines, if possible. The number of examinations performed will meet or exceed the minimum number required by each Examination Category. The number of components examined will meet the percentage requirements of Program B.

Implementation Schedule

PBNP will implement this alternative to the Code during the Point Beach Nuclear Plant fourth 10-year Inservice Inspection interval.

Attachments

ASME Code Case N-624

Status

Submitted.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

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CASE N-624

CASES OF ASME BOILER AND PRESSURE VESSEL CODE

Approval Date: May 7, 1999

See Numeric Index for expiration and any reaffirmation dates.

Case N-624 Successive Inspections, Section XI, Division 1

**Inquiry:** What alternative to the requirements of IWB-2420(a), IWC-2420(a), IWD-2420(a) (starting with the 1991 Addenda), IWE-2420(a) (starting with the Winter 1981 Addenda), and IWF-2420(a) (starting with the Winter 1978 Addenda) may be used to modify the sequence of examinations established during the first inspection interval, in a manner that optimizes scaffolding, radiological, insulation removal, or other considerations?

**Reply:** It is the opinion of the Committee that, as an alternative to the requirements of IWB-2420(a), IWC-2420(a), IWD-2420(a), IWE-2420(a), and IWF-2420(a), the sequence of examinations may be modified, provided the percentage requirements of Tables IWB-2411-1, IWC-2411-1, IWD-2411-1, and IWE-2411-1, Tables IWB-2412-1, IWC-2412-1, IWD-2412-1, and IWE-2412-1, Table IWF-2410-1, or IWF-2410-2 are satisfied.

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**Relief Request No. 9, Alternative to Welding and Brazing  
Performance Qualification Requirements**

Pursuant to 10 CFR 50.55a(a)(3)(ii), PBNP requests an alternative to the Code required welding and brazing performance qualification requirements as specified in the 1998 Edition with Addenda through 2000 of ASME Section XI.

Component Identification

Code Classes: 1, 2, and 3  
References: ASME, Section XI, IWA-4000, 1998 Edition, 2000 Addenda  
Examination Category: All  
Item Number: All  
Description: Alternative to Welding and Brazing Performance Qualification Requirements  
Component Numbers: All

Code Requirements

ASME, Section XI, IWA-4440(c) and (d), 1998 Edition, 2000 Addenda requires that the Owner / Repair Organization shall qualify all welders, welding operators, brazers, and brazing operators in accordance with the Codes specified in the Repair / Replacement Plan.

Basis for Relief

Relief is requested from the requirements for qualification of welders, welding operators, brazers, and brazing operators stated in ASME, Section XI, IWA-4000. This request also includes relief for all Welder Performance Qualification Records and Brazing Qualification Records (WPQ / BPQ) supporting alternative repair procedures in accordance with ASME, Section XI, IWA-4600 and balance of plant welding.

Point Beach Nuclear Plant (PBNP) requests to utilize an alternative to qualification of welders, welding operators, brazers, and brazing operators. This alternative extends the logic already found in Code Case N-573 (sharing Procedure Qualification Records between Owners), which has been incorporated into the 1998 Edition of Section XI with Addenda through 2000, which will be used at PBNP.

Maintaining the original requirement presents an undue hardship, as considerable utility specific resources would be duplicated when qualified welders, welding operators, brazers, and brazing operators from other NRC licensed facilities have already been demonstrated to provide an acceptable level of quality and safety. The requested alternative will not reduce safety or quality.

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Alternative Examinations

PBNP will use as an alternative to welding and brazing performance qualification requirements of IWA-4000, a welder, welding operator, brazer, or brazing operator qualified by other Owners. The following requirements shall be met:

1. The Owner that performed the qualification test shall certify that testing was performed in accordance with Section IX by signing the record of Welder/Welding Operator, Brazer, Brazing Operator Performance Qualification (WPQ/BPQ).
2. The Owner that performed the qualification test shall certify, in writing, that the qualification was conducted in accordance with a Quality Assurance Program that satisfies the requirements of ASME, Section XI, IWA-1400.
3. PBNP, when accepting the WPQ/BPQ, shall obtain any necessary supporting information to satisfy the requirements of ASME, Section IX, QW301.4 (e.g., welding Procedure Specification, type of tests) from the Owner supplying the WPQ/BPQ.
4. PBNP, when accepting the WPQ/BPQ, shall require each welder, welding operator, brazer, or brazing operator to demonstrate proficiency by completing a renewal qualification test in accordance with Section IX, QW 322.2(a) or QB-322(b).
  - When WPQ transfer involves prior groove tests, the renewal test shall use a groove configuration.
  - When WPQ transfer involves fillet tests, the renewal test may be either fillet or groove configuration.
5. PBNP, when accepting the WPQ/BPQ, shall accept responsibility for the Performance Qualification Test, and shall document acceptance on the WPQ/BPQ for the renewal test. This WPQ/BPQ shall reference the WPQ/BPQ supplied by the Owner that performed the qualification.
6. PBNP shall accept responsibility for compliance with Section IX, QW-322.
7. PBNP may accept and use a WPQ/BPQ only when it is received directly from the Owner that performed the qualification.
8. PBNP will comply with the PBNP Quality Assurance requirements of IWA-4142(a).

This alternative follows the requirements of ASME Code Case N-600, which was approved January 2001, but has not been published.

Implementation Schedule

The alternative is requested for the fourth 10-year interval of the Inservice Inspection Program for PBNP.

Attachments

None.

Status

Submitted.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
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**Relief Request No. 10, Relief from Regenerative Heat Exchanger Examinations**

Pursuant to 10 CFR 50.55a(a)(3)(ii), PBNP requests an alternative to the Code requirement for scheduling of components for examination as specified in the 1998 Edition of ASME Section XI with Addenda through 2000. To perform the examinations as required would result in excessive radiation dose accumulation and is a hardship.

Identification

Point Beach Nuclear Plant, Units 1 and 2

Regenerative Heat Exchangers 1HX-2 and 2HX-2

Examination Requirements

1998 Edition of Section XI with Addenda through 2000

Examination Category B-B, Item No. B2.51, circumferential head welds, volumetric examination

Unit 1	Unit 2
RHE-01	RHE-01
RHE-05	RHE-05
RHE-09	RHE-09

Examination Category B-B, Item No. B2.80, tubesheet to shell welds, volumetric examination

Unit 1	Unit 2
RHE-02	RHE-02
RHE-06	RHE-06
RHE-10	RHE-10

Examination Category B-D, Item No. B3.150, nozzle to vessel welds, volumetric examination

Unit 1	Unit 2
RHE-N1	RHE-N1
RHE-N4	RHE-N4
RHE-N5	RHE-N5
RHE-N8	RHE-N8
RHE-N9	RHE-N9
RHE-N12	RHE-N12

Examination Category B-D, Item No. B3.160, nozzle inner radius section, volumetric examination

Unit 1	Unit 2
RHE-N1-IRS	RHE-N1-IRS
RHE-N4-IRS	RHE-N4-IRS
RHE-N5-IRS	RHE-N5-IRS

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RHE-N8-IRS	RHE-N8-IRS
RHE-N9-IRS	RHE-N9-IRS
RHE-N12-IRS	RHE-N12-IRS

Examination Category C-A, Item No. C1.20, tubesheet to shell weld, volumetric examination

Unit 1	Unit 2
RHE-04	RHE-04
RHE-08	RHE-08
RHE-12	RHE-12

Examination Category C-A, Item No. C1.30, tubesheet to shell weld, volumetric examination

Unit 1	Unit 2
RHE-03	RHE-03
RHE-07	RHE-07
RHE-11	RHE-11

Examination Category C-B, Item No. C2.21, tubesheet to shell weld, volumetric examination

Unit 1	Unit 2
RHE-N2	RHE-N2
RHE-N3	RHE-N3
RHE-N6	RHE-N6
RHE-N7	RHE-N7
RHE-N10	RHE-N10
RHE-N11	RHE-N11

Relief Requested

Relief is requested from performing the examinations of the Regenerative Heat Exchanger welds as required by the 1998 Edition of Section XI with Addenda through 2000.

Basis for Relief

The Regenerative Heat Exchanger is a high radiation component, located inside of a lock high radiation area. It is the greatest single source of radiation exposure accumulated during a normal refueling outage for ISI and support personnel. Just as an outage begins, Radiation Protection personnel make a survey of the area to document dose rates. These rates are typically 700 mr to 1400 mr for the general area. Hot spots of 3000 mr are normally found on contact with the heat exchanger. The following dose accumulations are expected using 3.0 Rem-hour due to the close contact the workers and NDE examination personnel experience in the course of performing their duties for each weld:

0.2 Man-hours for insulation removal	= 0.6 Man-Rem
0.2 Man-hours for weld cleaning and preparation	= 0.6 Man-Rem
0.75 Man-hours for conducting examinations	= 1.5 Man-Rem

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0.75 Man hours second examiner (700 mr dose area)	= 0.525 Man-Rem
0.5 Man-hours for insulation replacement	= 1.5 Man-Rem
Total	= 4.725 Man-Rem

By eliminating 23 of the required vessel examinations, a total reduction in excess of 100 Man-Rem can be realized. While it is recognized this dose accumulation is probably a high estimate, it is obvious a significant reduction in dose accumulation will occur.

As part of the ALARA program, shielding is placed over non-examination areas. The general dose rates are reduced by approximately 50%. However, the highest dose rates are encountered during the examinations. The benefit the examiner receives from the shielding is minimal.

Early examinations of these welds show there are significant restrictions to meeting full Code compliance. In some cases, only 25% of the examination area was achieved.

The examination of the lowest vessel of the Regenerative Heat Exchanger will satisfy the IWB-1220(a) and IWC-1220(a) requirements to perform examinations on the same welds as was examined previously. The welds on this vessel were examined during the third interval in accordance with the previously approved relief requests, RR-1-12 and RR-2-12.

At the beginning of an outage, Operations personnel walk down the containment with procedure PC-24, Containment Inspection Checklist. This checklist requires entry into the regenerative heat exchanger cubicle to look for leakage from valves. Since the heat exchanger and valves are in close proximity to each other, and operations personnel are trained to look for leakage, any leakage would be noticed. System engineers also perform an entry into this area to look over their systems. There is also a walk down performed by NDE personnel to look for leakage anywhere in containment.

The consequences of a weld failure of one of the Regenerative Heat Exchanger welds has been addressed in the plant's Final Safety Analysis Report (FSAR). To evaluate chemical and volume control system (CVCS) safety, failures or malfunctions were assumed to be concurrent with a loss of coolant accident (LOCA) and the consequences analyzed. A LOCA and a concurrent Regenerative Heat Exchanger weld failure is included in the more general category of a rupture in the CVCS line inside containment. During such an occurrence, the remote-operated valve located near the main coolant loop, upstream of the Regenerative Heat Exchanger, is closed on low Pressurizer level to prevent supplementary loss of coolant through the letdown line. The Regenerative Heat Exchanger would eventually be isolated, with leakage being confined to the containment, in the case of a weld failure without a LOCA.

The bottom heat exchanger welds are the logical ones to be examined. The bottom heat exchanger operates at the highest temperature of the three and is the most highly stressed. Typical operating temperatures for letdown flow are 538 degrees into the bottom shell and 252 degrees out the top shell. The bottom heat exchange welds can generally be more extensively examined than the other heat exchanger welds due to ease of access. This was documented and was found during a review of previous examination data.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

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By implementing the proposed alternatives, the intent of the Code requirements are being met. The welds on the most severely stressed vessel are being volumetrically examined. With the combination of the Section XI volumetric examinations and leakage tests, the system engineer walkdowns, and the walkdown of the containment by operations and NDE personnel looking for areas where leakage occurred, the alternative examinations will provide an acceptable level of quality and safety.

Alternative Examinations

PBNP proposes to examine one of the three vessels comprising the Regenerative Heat Exchanger component. The will be the bottom vessel of the three. The accessible portions of the circumferential, head welds, tubesheet to shell welds, nozzle to shell welds, and nozzle inside radius sections on one of the identical vessels will be examined to the extent practical. The vessel selected for examination is the same as for the previous interval.

Implementation Schedule

PBNP will implement this alternative to the Code during the Point Beach Nuclear Plant fourth 10-year Inservice Inspection interval.

Attachments

ISI Drawing ISI-PRI-1107

ISI-Drawing ISI-PRI-2107

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

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**Relief Request No. 11, Emergency Diesel System VT-2 Examinations**

Pursuant to 10 CFR 50.55a(a)(3)(i), PBNP requests an alternative to the Code requirement for performing VT-2 examinations on selected diesel systems as specified in the 1998 Edition of ASME Section XI with Addenda through 2000.

Identification

Class 3 Standby Emergency Diesel Generator Subsystems

Applicable drawings:

- M-209 CBD Sh.12, Emergency Diesel Air Starting System (G01 & G02)
- M-209 CBD Sh.14, Starting and Service Air System Diesel Generator Building (G03)
- M-209 CBD Sh.15, Starting Air System Diesel Generator Building (G04)
- M-219 CBD Sh. 1, Fuel Oil System (G01 & G02)
- M-219 CBD Sh. 2, Fuel Oil System (G01 & G02)
- M-219 CBD Sh. 3, Fuel Oil System (G03 & G04)
- M-227 CBD Sh.1, Glycol Cooling System Diesel Generator Building (G03)
- M-227 CBD Sh.1, Glycol Cooling System Diesel Generator Building (G04)

Examination Requirements

Examination Category D-B, Item D 2.10 - System leakage test of Class 3 pressure retaining components once each period.

Examination Category D-B, Item D 2.20 - System hydrostatic test of Class 3 pressure retaining components once each interval.

Relief Requested

Relief is requested from performing the system leakage and hydrostatic testing on the Class 3 Standby Emergency Diesel Generator Subsystems.

Basis for Relief

The primary intent of Technical Specification surveillance testing is slightly different from Code required examinations. Technical Specifications are intended to demonstrate component operability, whereas the system leakage and hydrostatic tests are intended to demonstrate pressure boundary integrity. There are no VT-3 visual examinations imposed on the Emergency Diesel Generator (EDG) subsystems due to pressure/temperature or size exemptions as allowed IWD-1220. Therefore, verification of pressure boundary structural integrity on EDG subsystems is not included in the PBNP ISI Program. Successful EDG operability testing requires the associated subsystems to maintain pressure boundary integrity and therefore, provides an equivalent level of quality and safety to that of ASME Section XI inspections. Those auxiliary support subsystems addressed within the scope of this request for relief include the starting air system, fuel oil system, and glycol cooling system (G03 and G04 only for glycol cooling).

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

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The repeatability of auxiliary subsystem instrumentation (pressure, level, and temperature) recorded during surveillance testing provides supporting data for the 'indirect verification of component integrity. Operations personnel are specifically trained in the testing of the standby EDGs and are aware of the necessity to maintain pressure boundary of the auxiliary subsystems. They are also aware of the necessity to maintain unobstructed flow characteristics for components discharging to a tank vented to atmosphere as do the diesel fuel oil transfer pumps. Although not a specific step in the surveillance procedure, verification of component pressure boundary integrity is administratively required of personnel performing standby EDG operability testing. If evidence of leakage is identified during the test, a Condition Report and/or work order is initiated with corrective actions or repairs implemented and follow-up confirmatory testing is performed.

The following paragraphs provide specific procedural actions which support the use of alternative operability testing in lieu of ASME Section XI system pressure testing and VT-2 visual examination.  
Starting Air Auxiliary Subsystem

PBNP surveillance test procedures TS-81, 82, 83 and 84 are performed monthly to demonstrate EDG operability. As part of these procedures, pressures of both right and left bank air receivers are recorded prior to and subsequent to starting the engine with the drop in pressure verified to occur at the air start motor outlet ports. The satisfactory completion of this test demonstrates the skid-mounted air start components are properly performing their function and provides positive indication the pressure boundary integrity of the starting air subsystem is intact. In addition to the monthly testing, Inservice Test Procedure IT-100 performs quarterly reverse exercising of the right/left bank air start receivers' inlet check valves. During the performance of this procedure, each air compressor is isolated with a vent path provided upstream of the air receiver supply check valves. Receiver pressure is observed for 15 minutes with stringent leakage criteria applied. If a through wall or otherwise excessive leak were to occur in the pressure boundary, seat leakage acceptance criteria for the check valves would be exceeded, resulting in a requirement to determine the source of the leak and repair/replacement. This data also provides a positive indication that pressure boundary integrity is being maintained for the starting air subsystem. Based on the monthly and quarterly test frequencies and the data collected during these alternative tests, PBNP considers that testing performed to satisfy the Technical Specification surveillance requirements provide an acceptable level of quality and safety as an alternative to ASME Section XI system pressure testing.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

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### Fuel Oil Transfer Subsystem

For the fuel oil transfer subsystem, an acceptable ASME Section XI pressure test would consist of a VT-2 visual examination on the outlet piping from the day tank to the engine. This is done when the day tank is filled to design capacity and demonstrates the transfer pump's ability to provide adequate makeup flow to the day tank during system operation. This is due to the day tank being vented to atmosphere. During the monthly performance of TS-81, 82, 83 and 84, the inventory in the day tank is drained down to the low-level setpoint for pump actuation. The pump is verified to automatically start and allowed to replenish the day tank inventory to the high level setpoint with verification the pump automatically stops. During this process, procedure steps require recording of the percentage of tank level when the transfer pump automatically starts, as well as the percentage of tank level upon cessation of pump operation. The pump flow rate is recorded during replenishment of day tank inventory for G03 and G04 with acceptance criteria applied to recorded flow rate values. Discharge flow rate for G01 and G02 transfer pumps is not measured during the monthly performance of TS-81 and TS-82 as there is no flow instrumentation in the pumps discharge lines to G01 and G02 day tanks. The flow rate to G01 and G02 day tanks is measured each Unit 1 refueling outage, utilizing an ultrasonic flow meter during inservice testing of unloader valves FO-3982A and FO-3983A. This data provides a positive indication that pressure boundary integrity is being maintained. Based on the Technical Specification surveillance testing frequency and the data collected during these alternative tests, PBNP considers the testing performed to satisfy the Technical Specification surveillance requirements provide an acceptable level of quality and safety as an alternative to ASME Section XI system pressure testing.

### Glycol Cooling Subsystem (G03 and G04 Only)

Standby emergency diesel generators G03 and G04 are provided with a glycol cooling subsystem consisting of a coolant to air type heat exchanger. During the monthly performance of TS-83 and TS-84, coolant tank level as well as multiple point temperature indication is recorded prior to starting the engine, after 30 minutes of loaded run time, and prior to shut down, or hourly for extended runs. Normal values for all acquired data are provided in the procedure log-sheet as well as limits for the data recorded. This data provides a positive indication that pressure boundary integrity is being maintained. Based on the monthly frequency and data collected during these tests, PBNP considers the testing performed to satisfy the Technical Specification surveillance requirements provide an acceptable level of quality and safety as an alternative to ASME Section XI system pressure testing.

Technical Specification surveillances requires standby emergency diesel generators to be subject to an inspection in accordance with procedures prepared per the manufacturer's recommendation. These examinations provide added assurance the components within the starting air, fuel oil transfer, and glycol cooling subsystems demonstrate pressure boundary integrity and the ability to provide adequate flow for satisfactory Standby Emergency Diesel Generator operation.

Essentially this same relief was authorized for use in SER dated May 31, 2000. The Technical Specification surveillance requirements provide an acceptable level of quality and safety and is an acceptable alternative to ASME Section XI system leakage and hydrostatic testing.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM

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Alternative Examinations

As an alternate to performing the required pressure testing on subsystems supporting the standby Emergency Diesel Generators, PBNP proposes utilizing Plant Technical Specifications surveillance testing as an acceptable alternative to that required by the Code.

Implementation Schedule

PBNP will implement this alternative to the Code during the Point Beach Nuclear Plant fourth 10-year Inservice Inspection interval.

Attachments

None

Status

Submitted.

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM SCHEDULE  
UNIT 1

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Point Beach Nuclear Plant Unit 1  
Class 1, 2, and 3 Inservice  
Inspection Program Schedule

Written by

Nuclear Management Company LLC  
6610 Nuclear Road  
Two Rivers, Wisconsin 54241

Commercial Service Date: December 21, 1970

USNRC Docket Number: 50-266

Effective Date: July 1, 2002

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM SCHEDULE  
UNIT 1

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Record of Revision

Revision No.	Date	Reason for Revision
0	03/11/2002	Original Issue

PBNP CLASS 1, 2, AND 3 INSERVICE INSPECTION  
PROGRAM SCHEDULE  
UNIT 1

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This document describes the Point Beach Nuclear Plant Unit 1, Class 1, 2, and 3 Inservice Inspection Program Schedule. Wisconsin Electric Power Company owns Point Beach Nuclear Plant. The Nuclear Management Company LLC is the Licensee and administrator of the plant. For purposes of the ISI Program, the NMC is considered the Owner in accordance with the definition in IWA-9000.

This ISI Program schedule was developed and prepared to meet the requirements of the American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section XI, 1998 edition with Addenda through 2000, for Class 1, 2, and 3 components. It identifies those components subject to examination and testing. As outage results, plant modifications, and updated plant drawing information is entered, the schedule of examinations will be updated.

Alternatives to the Code requirements, which are the subject of relief requests, are incorporated. Code Cases are also incorporated, but may not be identified specifically within the text of this document. Refer to the Point Beach Nuclear Plant Class 1, 2, and 3 Inservice Inspection Program for details.

The ISI Program Schedule for Unit 2 and Containment Examinations are covered under separate documents.

PBNP may substitute unscheduled components for scheduled components. This substitution may be performed when conditions such as limited physical access, high radiation levels, or other situations exist that necessitate a substitute examination. Substituted components shall meet the scheduling requirements of Inspection Program B, the appropriate Examination Category, and applicable reliefs. Specific examinations required by the Code that cannot be completed will be the subject of a relief request. For those components subject to the Risk-Informed selection process, substitutions shall follow the guidelines of the Risk-Informed ISI Program.

Components that are substituted for scheduled components shall, to the extent practical, be on the same line and in the same area as the originally scheduled component, and of the same configuration. If a suitable area on the same line cannot be examined, then a similar component on another train shall be substituted. If this is not possible, then the closest similar component shall be selected.

The sequence of component examinations will be modified in accordance with the requirements of ASME Code Case N-624. This allows PBNP to alter the sequence of examinations to allow the examination of several components in an area during one outage instead of over several outages. This will reduce costs and radiation exposure. The percentage requirements of IWB-2412, IWC-2412, IWD-2412, and IWF-2410 (Program B) shall be satisfied.

Additional requirements for augmented examinations are addressed. The ISI Program does not require these examinations, but rather are administered at the request of the plant. The ISI Coordinator performs the planning and the tracking of each item.

Point Beach Nuclear Plant  
Unit 1  
Inservice Inspection  
Program Schedule

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
<u>Reactor Pressure Vessel / Dwg. No. ISI-PRI-1101</u>															
001000	RPV-14-683 Shell to Flange Cal Block: 5-PTB, 13-PTB Examine from the flange surface during the first period and from the shell side during the third period.	B-A B1.30	VOL	X	-	-	-	-	-	-	-	X	-	-	-
001100	RPV-15-683 Upper Shell to Middle Shell Cal Block: 12-PTB	B-A B1.11	VOL	-	-	-	-	-	-	-	-	X	-	-	-
001200	RPV-16-683 Middle Shell to Lower Shell Cal Block: 12-PTB	B-A B1.11	VOL	-	-	-	-	-	-	-	-	X	-	-	-
001300	RPV-17-683 Lower Shell to Lower Head Ring Cal Block: 12-PTB	B-A B1.21	VOL	-	-	-	-	-	-	-	-	X	-	-	-
001400	RPV-18-683 Lower Head Ring to Lower Head Cal Block: 11-PTB	B-A B1.21	VOL	-	-	-	-	-	-	-	-	X	-	-	-
001500	RPV-MK-2 Middle Shell Longitudinal Seam 15 Deg Cal Block: 12-PTB	B-A B1.12	VOL	-	-	-	-	-	-	-	-	X	-	-	-
001600	RPV-MK-3 Lower Shell Longitudinal Seam at 195 Deg Cal Block: 12-PTB	B-A B1.12	VOL	-	-	-	-	-	-	-	-	X	-	-	-
001700	RPV-2-686-A Outlet Nozzle at 28.5 Deg Cal Block: 5-PTB	B-D B3.90	VOL	-	-	-	-	-	-	-	-	X	-	-	-
001900	RPV-2-686-A-IRS Outlet Nozzle at 28.5 Deg Cal Block: 14-PTB Reference LER #84-002-00 dated 3/28/84.	B-D B3.100	VOL	-	-	-	-	-	-	-	-	X	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
002000	RPV-2-686-B Inlet Nozzle at 148.5 Deg Cal Block: 5-PTB Core barrel must be removed for access.	B-D B3.90	VOL	-	-	-	-	-	-	-	-	X	-	-	-
002100	RPV-2-686-B-IRS Inlet Nozzle at 148.5 Deg Cal Block: 14-PTB, 13-PTB Core barrel must be removed for access.	B-D B3.100	VOL	-	-	-	-	-	-	-	-	X	-	-	-
002200	RPV-2-686-C Outlet Nozzle at 208.5 Deg Cal Block: 5-PTB	B-D B3.90	VOL	-	-	-	-	-	-	-	-	X	-	-	-
002400	RPV-2-686-C-IRS Outlet Nozzle at 208.5 Deg Cal Block: 14-PTB	B-D B3.100	VOL	-	-	-	-	-	-	-	-	X	-	-	-
002500	RPV-2-686-D Inlet Nozzle at 328.5 Deg Cal Block: 5-PTB Core barrel must be removed for access.	B-D B3.90	VOL	-	-	-	-	-	-	-	-	X	-	-	-
002600	RPV-2-686-D-IRS Inlet Nozzle at 328.5 Deg Cal Block: 14-PTB, 13-PTB Core barrel must be removed for access.	B-D B3.100	VOL	-	-	-	-	-	-	-	-	X	-	-	-
002700	RPV-687-01-A SI Nozzle at 288.5 Deg Cal Block: 23-PTB	B-D B3.90	VOL	-	-	-	-	-	-	-	-	X	-	-	-
002900	RPV-687-01-A-IRS SI Nozzle at 288.5 Deg Cal Block: 23-PTB	B-D B3.100	VOL	-	-	-	-	-	-	-	-	X	-	-	-
003000	RPV-687-01-B SI Nozzle at 108.5 Deg Cal Block: 23-PTB	B-D B3.90	VOL	-	-	-	-	-	-	-	-	X	-	-	-
003200	RPV-687-01-B-IRS SI Nozzle at 108.5 Deg Cal Block: 23-PTB	B-D B3.100	VOL	-	-	-	-	-	-	-	-	X	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
003300	RPV-MK-16 Support at 88.5 Deg Cal Block: 5-PTB, 13-PTB	B-K B10.10	VOL	-	-	-	-	-	-	-	-	X	-	-	-
003400	RPV-MK-17 Support at 268.5 Deg Cal Block: 5-PTB, 13-PTB	B-K B10.10	VOL	-	-	-	-	-	-	-	-	X	-	-	-
003500	RPV-MK-7-1 Core Support Guide at 0 Deg.  Examine when core barrel is removed.	B-N-2 B13.60	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
003600	RPV-MK-7-2 Core Support Guide at 90 Deg.  Examine when core barrel is removed.	B-N-2 B13.60	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
003700	RPV-MK-7-3 Core Support Guide at 180 Deg.  Examine when core barrel is removed.	B-N-2 B13.60	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
003800	RPV-MK-7-4 Core Support Guide at 270 Deg.  Examine when core barrel is removed.	B-N-2 B13.60	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
003900	RPV Core Support Core Support Structure  Examine accessible surfaces. Core barrel shall be removed from the reactor vessel for examination.	B-N-3 B13.70	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
004100	RPV Interior Surface Vessel Interior Surfaces  Examine accessible surfaces each period.	B-N-1 B13.10	VT-3	X	-	-	-	X	-	-	-	X	-	-	-
004220	HS-19 Snubber  Exempt from examination.		VT-3	-	-	-	-	-	-	-	-	-	-	-	-
004240	HS-20 Snubber  Exempt from examination.		VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
<u>Reactor Pressure Vessel Head to Flange / Dwg. No. ISI-PRI-1102</u>															
004300	RPV-HD-CLAD RPV Head Interior Cladding		VT-3	-	-	-	-	X	-	-	-	-	-	-	-
Examination required for NRC Information Notice 90-29 and PBM 91-0730.															
004400	RPV-HFLANGE Head to Flange Cal Block: 11-PTB Surface examination may have limitation due to rough scale on the flange side of the weld.	B-A B1.40	SUR VOL	X	-	-	-	-	-	-	-	-	-	-	-
<u>Reactor Pressure Vessel Bolting and CRD Housings / Dwg. No. ISI-PRI-1103</u>															
004500	CRD Housings Housings 1 thru 49	B-O B14.10	SUR	-	-	-	-	-	-	-	-	-	-	X	-
EXAMINE 10% PERIPHERAL CRD HOUSINGS.															
004700	RPV Closure Head Nuts RPV Closure Head Nuts 1 thru 48	B-G-1 B6.10	VT-1	X	-	-	-	X	-	-	-	X	-	-	-
Examine nuts 1 through 16 during the first period, 17 through 32 in the second, and 33 through 48 in the third.															
004800	RPV Closure Head Studs RPV Closure Head Studs 1 thru 48 Cal Block: 21-PTB Examine studs 1 through 16 during the first period, 17 through 32 in the second, and 33 through 48 in the third.	B-G-1 B6.30	VOL	X	-	-	-	X	-	-	-	X	-	-	-
004900	RPV Closure Head Washers RPV Closure Head Washers 1 thru 48	B-G-1 B6.50	VT-1	X	-	-	-	X	-	-	-	X	-	-	-
Examine small and large washers 1 through 16 during the first period, 17 through 32 in the second, and 33 through 48 in the third.															
005000	RPV THREADS IN FLANGE RPV Flange Threaded Areas	B-G-1 B6.40	VOL	X	-	-	-	-	-	-	-	-	-	-	-
<u>Pressurizer T-1 / Dwg. No. ISI-PRI-1104</u>															
006000	PZR-Manway-BLT-01 thru 16 Manway Bolting	B-G-2 B7.20	VT-1	X	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
006100	PZR-RELNOZ-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.120	VOL	-	-	-	-	X	-	-	-	-	-	-	-
006200	PZR-RELNOZ-SE Nozzle to Safe-End Cal Block: 37-PTB Radiography to be performed in lieu of Ultrasonics due to examination area limitations. During the fourth interval, the category will become R-A. Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
006300	PZR-SAFNOZ-1-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.120	VOL	-	-	-	-	X	-	-	-	-	-	-	-
006400	PZR-SAFNOZ-1 Nozzle to Safe-End  Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
006500	PZR-SAFNOZ-2-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.120	VOL	-	-	-	-	X	-	-	-	-	-	-	-
006600	PZR-SAFNOZ-2 Nozzle to Safe-End  Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
006700	PZR-SPRAYNOZ-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.120	VOL	-	-	-	-	X	-	-	-	-	-	-	-
006800	PZR-SPRAYNOZ-SE Safe-End to Nozzle Cal Block: 36-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>Pressurizer T-1 / Dwg. No. ISI-PRI-1105</u>															
007200	PZR-Cweld-1 Upper Head to Upper Shell Cal Block: 38-PTB Has an intersectiong seam weld PZR-Vweld-1 at 255 degrees.	B-B B2.11	VOL	-	-	-	-	X	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
007300	PZR-Cweld-2 Upper Shell to Lower Shell Cal Block: 38-PTB No examination required. This weld is not addressed by Section XI.			-	-	-	-	-	-	-	-	-	-	-	-
007400	PZR-Cweld-3 Lower Shell to Lower Head Cal Block: 38-PTB Has an intersecting seam weld PZR-Vweld-2 at 285 degrees.	B-B B2.11	VOL	-	-	-	-	-	-	-	-	X	-	-	-
007500	PZR-Vweld-1 Upper Shell Vertical Weld (Az 255) Cal Block: 38-PTB Examine one foot of the weld intersecting the circumferential weld PZR-CWELD-1.	B-B B2.12	VOL	-	-	-	-	X	-	-	-	-	-	-	-
007600	PZR-Vweld-2 Lower Shell Vertical Weld (Az 285) Cal Block: 38-PTB Examine one foot of the weld intersecting the circumferential weld PZR-CWELD-3.	B-B B2.12	VOL	-	-	-	-	-	-	-	-	X	-	-	-
007700	PZR-SKIRT Component Support	F-A F1.40	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
007800	PZR-SKIRT-IWA Welded Attachment Cal Block: 19-PTB No examination required. Weld is located beyond IWB boundary. Will be examined as part of Category F-A.			-	-	-	-	-	-	-	-	-	-	-	-
007900	PZR-SURGENOZ-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.120	VOL	X	-	-	-	-	-	-	-	-	-	-	-
008000	PZR-SURGENOZ-SE Safe-End to Nozzle Cal Block: 35-PTB Dissimilar metal weld.	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>Steam Generator Lower Head / Dwg. No. ISI-PRI-1106</u>															
009000	SG-A-LH-1 Lower Head to Tubesheet Cal Block: 40-PTB	B-B B2.40	VOL	-	-	-	-	-	-	-	-	X	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
009100	SG-A-INLET NOZZLE-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.140	VOL	-	-	-	-	-	-	-	-	X	-	-	-
009200	SG-A-OUTLET NOZZLE-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.140	VOL	-	-	-	-	-	-	-	-	X	-	-	-
009500	SG-B-LH-1 Lower Head to Tubesheet Cal Block: 40-PTB	B-B B2.40	VOL	-	-	-	-	-	-	-	-	-	-	-	-
009600	SG-B-INLET NOZZLE-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.140	VOL	-	-	-	-	X	-	-	-	-	-	-	-
009700	SG-B-OUTLET NOZZLE-IRS Inside Radius Section Cal Block: 39-PTB	B-D B3.140	VOL	-	-	-	-	X	-	-	-	-	-	-	-
<u>Regenerative Heat Exchanger HX-2 / Dwg. No. ISI-PRI-1107</u>															
010000	RHE-01 Head to Shell Cal Block: 17-PTB	B-B B2.51	VOL	X	-	-	-	-	-	-	-	-	-	-	-
010100	RHE-02 Shell to Tube Sheet Cal Block: 17-PTB	B-B B2.80	VOL	X	-	-	-	-	-	-	-	-	-	-	-
010200	RHE-05 Head to Shell Cal Block: 17-PTB	B-B B2.51	VOL	-	-	-	-	-	-	-	-	-	-	-	-
010300	RHE-06 Shell to Tube Sheet Cal Block: 17-PTB	B-B B2.80	VOL	-	-	-	-	-	-	-	-	-	-	-	-
010400	RHE-09 Head to Shell Cal Block: 17-PTB	B-B B2.51	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
010500	RHE-10 Shell to Tube Sheet Cal Block: 17-PTB	B-B B2.80	VOL	-	-	-	-	-	-	-	-	-	-	-	-
010600	RHE-N1 Inlet Nozzle to Shell Cal Block: 61-PTB, 62-PTB	B-D B3.150	VOL	X	-	-	-	-	-	-	-	-	-	-	-
010700	RHE-N1-IRS RHE Nozzle Inner Radius Section Cal Block: 61-PTB	B-D B3.160	VOL	X	-	-	-	-	-	-	-	-	-	-	-
010800	RHE-N4 Shell to Outlet Nozzle Cal Block: 61-PTB, 62-PTB	B-D B3.150	VOL	X	-	-	-	-	-	-	-	-	-	-	-
010900	RHE-N4-IRS RHE Nozzle Inner Radius Section Cal Block: 61-PTB	B-D B3.160	VOL	X	-	-	-	-	-	-	-	-	-	-	-
011000	RHE-N5 Inlet Nozzle to Shell Cal Block: 61-PTB, 62-PTB	B-D B3.150	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011100	RHE-N5-IRS RHE Nozzle Inner Radius Section Cal Block: 61-PTB	B-D B3.160	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011200	RHE-N8 Shell to Outlet Nozzle Cal Block: 61-PTB, 62-PTB	B-D B3.150	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011300	RHE-N8-IRS RHE Nozzle Inner Radius Section Cal Block: 61-PTB	B-D B3.160	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011400	RHE-N9 Inlet Nozzle to Shell Cal Block: 61-PTB, 62-PTB	B-D B3.150	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
011500	RHE-N9-IRS RHE Nozzle Inner Radius Section Cal Block: 61-PTB	B-D B3.160	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011600	RHE-N12 Shell to Outlet Nozzle Cal Block: 61-PTB, 62-PTB	B-D B3.150	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011700	RHE-N12-IRS RHE Nozzle Inner Radius Section Cal Block: 61-PTB	B-D B3.160	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011800	CVC-03-LD-1001-01 Nozzle to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
011900	CVC-03-LD-1001-02 Pipe to Nozzle	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
012000	CVC-03-LD-1001-03 Nozzle to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
012100	CVC-03-LD-1001-04 Pipe to Nozzle	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
012200	RHE-SHELL SUPPORT-1 thru 6 Component Support	F-A F1.40	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
<u>Reactor Coolant Pump Supports / Dwg. No. ISI-PRI-1108</u>															
013000	RCP-A-LEG-1 thru 3 Component Support	F-A F1.40	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
013100	RCP-A-1-IWA Welded Attachment	B-K B10.30	SUR	-	-	-	-	-	-	-	-	X	-	-	-

Examine one of the three welded attachments to the pump.

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
013200	RCP-A-2-IWA Welded Attachment	B-K B10.30	SUR	-	-	-	-	-	-	-	-	-	-	-	-
Examine one of the three welded attachments to the pump.															
013300	RCP-A-3-IWA Welded Attachment	B-K B10.30	SUR	-	-	-	-	-	-	-	-	-	-	-	-
Examine one of the three welded attachments to the pump.															
014000	RCP-B-LEG-1 thru 3 Component Support	F-A F1.40	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
014100	RCP-B-1-IWA Welded Attachment	B-K B10.30	SUR	-	-	-	-	-	-	-	-	-	-	-	-
014200	RCP-B-2-IWA Welded Attachment	B-K B10.30	SUR	-	-	-	-	-	-	-	-	-	-	-	-
014300	RCP-B-3-IWA Welded Attachment	B-K B10.30	SUR	-	-	-	-	-	-	-	-	-	-	-	-
<u>Reactor Coolant Pump Casing Welds / Dwg. No. ISI-PRI-1109</u>															
015000	RCP-A-WELD-A Pump Casing Weld	B-L-1 B12.10	VT-1	-	-	-	-	-	-	-	-	X	-	-	-
015100	RCP-A-WELD-B Pump Casing Weld	B-L-1 B12.10	VT-1	-	-	-	-	-	-	-	-	X	-	-	-
015200	RCP-A-WELD-C Pump Casing Weld	B-L-1 B12.10	VT-1	-	-	-	-	-	-	-	-	X	-	-	-
015300	RCP-A-CASE-EXT Pump Casing Exterior	B-P B15.60	VT-2	X	X	-	-	X	X	-	-	X	X	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
015400	RCP-A-Case-Interior Pump Internal Surface	B-L-2 B12.20	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Examine interior surface of pump if disassembled.															
016000	RCP-B-WELD-A Pump Casing Weld	B-L-1 B12.10	VT-1	-	-	-	-	-	-	-	-	-	-	-	-
016100	RCP-B-WELD-B Pump Casing Weld	B-L-1 B12.10	VT-1	-	-	-	-	-	-	-	-	-	-	-	-
016200	RCP-B-WELD-C Pump Casing Weld	B-L-1 B12.10	VT-1	-	-	-	-	-	-	-	-	-	-	-	-
016300	RCP-B-CASE-EXT Pump Casing Exterior	B-P B15.60	VT-2	X	X	-	-	X	X	-	-	X	X	-	-
016400	RCP-B-Case-Interior Pump Internal Surface	B-L-2 B12.20	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Examine interior surface of pump if disassembled.															
<u>Reactor Coolant Pump Flange and Bolting / Dwg. No. ISI-PRI-1110</u>															
017000	RCP A FLANGE Flange Surface	B-G-1 B6.190	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
Examine when connection is disassembled or if the presence of borated water is detected.															
017100	RCP A STUDS 24 Studs Cal Block: 53-PTB, 56-PTB	B-G-1 B6.180	VOL	-	-	-	-	X	-	-	-	-	-	-	-
017200	RCP A NUTS 24 Nuts	B-G-1 B6.200	VT-1	-	-	-	-	-	-	-	-	X	-	-	-
017300	RCP A SEAL BOLTS 18 Seal Bolts	B-G-2 B7.60	VT-1	X	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
018000	RCP B FLANGE Flange Surface	B-G-1 B6.190	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
Examine when connection is disassembled or if the presence of borated water is detected.															
018100	RCP B STUDS 24 Studs Cal Block: 53-PTB, 56-PTB	B-G-1 B6.180	VOL	-	-	-	-	-	-	-	-	-	-	-	-
018200	RCP B NUTS 24 Nuts	B-G-1 B6.200	VT-1	-	-	-	-	-	-	-	-	-	-	-	-
018300	RCP B SEAL BOLTS 18 Seal Bolts	B-G-2 B7.60	VT-1	-	-	-	-	-	-	-	-	-	-	-	-
<u>Steam Generators HX-1A and HX-1B / Dwg. No. ISI-PRI-1112</u>															
019000	SG-A-MAIN SUPPORT-1 thru 4 Component Support	F-A F1.40	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
019100	SG-A-MWAY-INLET-BLT-01 THRU 16 Manway Bolting	B-G-2 B7.30	VT-1	-	-	-	-	-	-	-	-	X	-	-	-
019200	SG-A-MWAY-OUTLET-BLT-01 THRU 16 Manway Bolting	B-G-2 B7.30	VT-1	-	-	-	-	-	-	-	-	X	-	-	-
020000	SG-B-MAIN SUPPORT-1 thru 4 Component Support	F-A F1.40	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
020100	SG-B-MWAY-INLET-BLT-01 THRU 16 Manway Bolting	B-G-2 B7.30	VT-1	-	-	-	-	X	-	-	-	-	-	-	-
020200	SG-B-MWAY-OUTLET-BLT-01 THRU 16 Manway Bolting	B-G-2 B7.30	VT-1	-	-	-	-	X	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
<u>Main Reactor Coolant Loop A / Dwg. No. ISI-PRI-1120</u>															
021000	RC-34-MRCL-AI-01 Outlet Nozzle to Pipe at 28.5 Deg Cal Block: 7-PTB, 24-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021100	RC-34-MRCL-AI-SI-BC 6" Branch Connection Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021200	RC-34-MRCL-AI-01A Pipe to Pipe Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021300	RC-34-MRCL-AI-AC-BC 10" Branch Connection Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021400	RC-34-MRCL-AI-02 Pipe to Elbow Cal Block: 10-PTB, 24-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021500	RC-34-MRCL-AI-DR-BC 4" Branch Connection Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021600	RC-34-MRCL-AI-03 Elbow to A S/G Inlet Nozzle Cal Block: 24-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021700	RC-06-SI-1003-01 Branch Connection to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
021800	RC-06-SI-1003-02 Pipe to Cap Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
021900	RC-04-DR-1004-01 Branch Connection to Cap Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				O U T A G E											
				1	2	3	4	1	2	3	4	1	2	3	4
023000	RC-36-MRCL-AII-01 Outlet Nozzle to Elbow Cal Block: 24-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
023100	RC-36-MRCL-AII-02 Elbow to Pipe Cal Block: 10-PTB, 24-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
023200	RC-36-MRCL-AII-03 Pipe to Elbow Cal Block: 10-PTB, 24-PTB Two longitudinal seam welds on the downstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
023210	RC-36-MRCL-AII-03L Elbow Longitudinal Seam Weld			-	-	-	-	-	-	-	-	-	-	-	-
023280	RC-36-MRCL-AII-04LUI Inside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
023290	RC-36-MRCL-AII-04LUO Outside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
023300	RC-36-MRCL-AII-04 Elbow to Pipe Cal Block: 10-PTB, 24-PTB Two longitudinal seam welds on the upstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
023400	RC-36-MRCL-AII-BP-BC 3" Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
023500	RC-36-MRCL-AII-DR-BC 2" Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
023600	RC-36-MRCL-AII-05 Pipe to Elbow Cal Block: 10-PTB, 24-PTB Two longitudinal seam welds on the downstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				O U T A G E											
				1	2	3	4	1	2	3	4	1	2	3	4
023610	RC-36-MRCL-AII-05LDI Inside Downstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
023620	RC-36-MRCL-AII-05LDO Outside Downstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
023680	RC-36-MRCL-AII-06LUI Inside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
023690	RC-36-MRCL-AII-06LUO Outside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
023700	RC-36-MRCL-AII-06 Elbow to Pump Cal Block: 24-PTB Two longitudinal seam welds on the upstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
025000	RC-32-MRCL-AIII-01 Pump to Pipe Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
025100	RC-32-MRCL-AIII-BP-BC 2" Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
025200	RC-32-MRCL-AIII-CH-BC 3" Branch Connection Cal Block: 10-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
025300	RC-32-MRCL-AIII-PS-BC 3" Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
025400	RC-32-MRCL-AIII-01A Pipe to Pipe Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
025500	RC-32-MRCL-AIII-SI-BC 10" Branch Connection Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
025600	RC-32-MRCL-AIII-02 Pipe to Elbow Cal Block: 10-PTB, 24-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
025700	RC-32-MRCL-AIII-03 Elbow to Inlet Nozzle at 328.5 Deg Cal Block: 7-PTB, 24-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>Main Reactor Coolant Loop B / Dwg. No. ISI-PRI-1121</u>															
027000	RC-34-MRCL-BI-01 Outlet Nozzle to Pipe at 208.5 Deg Cal Block: 7-PTB, 10-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
027100	RC-34-MRCL-BI-01A Pipe to Pipe  Located 6.5 ft from elbow.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
027200	RC-34-MRCL-BI-SI-BC 6" Branch Connection Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
027300	RC-34-MRCL-BI-PZR-BC 10" Branch Connection Cal Block: 10-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
027400	RC-34-MRCL-BI-DR-BC 4" Branch Connection Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
027500	RC-34-MRCL-BI-02 Pipe to Elbow Cal Block: 10-PTB, 24-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
027600	RC-34-MRCL-BI-03 Elbow to B S/G Inlet Nozzle Cal Block: 24-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
027700	RC-06-SI-1004-01 Branch Connection to Cap Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
027800	RC-04-DR-1003-01 Branch Connection to Cap Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
029000	RC-36-MRCL-BII-01 Outlet Nozzle to Elbow Cal Block: 24-PTB Dissimilar metal weld.	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
029100	RC-36-MRCL-BII-02 Elbow to Pipe Cal Block: 10-PTB, 24-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
029200	RC-36-MRCL-BII-03 Pipe to Elbow Cal Block: 10-PTB, 24-PTB Two longitudinal seam welds on the downstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
029210	RC-36-MRCL-BII-03LDI Inside Downstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029220	RC-36-MRCL-BII-03LDO Outside Downstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029280	RC-36-MRCL-BII-04LUI Inside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029290	RC-36-MRCL-BII-04LUO Outside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029300	RC-36-MRCL-BII-04 Elbow to Pipe Cal Block: 10-PTB, 24-PTB Two longitudinal seam welds on the upstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
029400	RC-36-MRCL-BII-DR-BC 8" Branch Connection Cal Block: 10-PTB, PI-60 Calibration block was borrowed from Prairie Island Nuclear Plant.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
029500	RC-36-MRCL-BII-BP-BC 3" Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
029600	RC-36-MRCL-BII-05 Pipe to Elbow Cal Block: 10-PTB, 24-PTB Two longitudinal seam welds on the downstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
029610	RC-36-MRCL-BII-05LDI Inside Downstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029620	RC-36-MRCL-BII-05LDO Outside Downstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029680	RC-36-MRCL-BII-06LUI Inside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029690	RC-36-MRCL-BII-06LUO Outside Upstream Longitudinal Weld Cal Block: 24-PTB			-	-	-	-	-	-	-	-	-	-	-	-
029700	RC-36-MRCL-BII-06 Elbow to Pump Cal Block: 24-PTB Two longitudinal seam welds on the upstream side.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
031000	RC-32-MRCL-BIII-01 Pump to Pipe Cal Block: 10-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
031100	RC-32-MRCL-BIII-BP-BC 2" Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
031200	RC-32-MRCL-BIIII-PS-BC 3" Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
031300	RC-32-MRCL-BIIII-SI-BC 10" Branch Connection Cal Block: 10-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
031400	RC-32-MRCL-BIIII-ACH-BC 2" Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
031500	RC-32-MRCL-BIIII-01A Pipe to Pipe Cal Block: 10-PTB Located 3 ft from pump.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
031600	RC-32-MRCL-BIIII-02 Pipe to Elbow Cal Block: 10-PTB, 24-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
031700	RC-32-MRCL-BIIII-03 Elbow to Inlet Nozzle at 148.5 Deg Cal Block: 7-PTB, 24-PTB Dissimilar metal weld. Volumetric examination will be performed from the inside surface.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>RHR Suction off Loop A / Dwg. No. ISI-PRI-1122</u>															
032000	RC-10-AC-1001-01 Branch Connection to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
032100	RC-10-AC-1001-02 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
032300	RC-10-AC-1001-03 Pipe to Elbow Cal Block: 51-PTB	R-A R1.11	VOL	X	-	-	-	-	-	-	-	-	-	-	-
032400	RC-10-AC-1001-04 Elbow to Pipe Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
032600	RC-10-AC-1001-05 Pipe to Elbow Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
032700	RC-10-AC-1001-06 Elbow to Pipe Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
033000	RC-10-AC-1001-07 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
033200	RC-10-AC-1001-08 Elbow to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
033300	RC-10-AC-1001-09 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
033400	RC-2501R-16-A5 Anchor	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
033500	RC-2501R-16-A5-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
033600	RC-10-AC-1001-10 Pipe to Valve RH-700 Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
033700	Valve RH-700 Valve Internal Surface	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Group 2 valve. Examine when disassembled. 10" Velan Gate Valve (11663)															
033800	Valve 1RH-700-BLT Valve Bolting 10" Gate	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
Group 2 valve. Examine when the valve is disassembled or during the third period, whichever is first.															

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
033900	AC-10-AC-1001-11 Valve RH-700 to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
034000	AC-2501R-1-R47 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
034050	AC-2501R-1-R47-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
034100	AC-10-AC-1001-12 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
034200	AC-10-AC-1001-13 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
034400	AC-10-AC-1001-14 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
034500	AC-10-AC-1001-15 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
034700	AC-10-AC-1001-16 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
034800	AC-2501R-1-AC-3 Spring Hanger	F-A F1.10C	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
034900	AC-2501R-1-AC3-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
035000	AC-10-AC-1001-17 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
035500	AC-10-AC-1001-18 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
035600	AC-10-AC-1001-19 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
035800	AC-2501R-1-R144 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
035850	AC-2501R-1-R144-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
035900	AC-10-AC-1001-20 Pipe to Valve RH-701 Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
036000	Valve RH-701 Valve Internal Surface  Group 1 valve. Examine when disassembled. 10" Velan Gate Valve (88904-1).	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
036100	Valve RH-701-Welds Two Valve Body Welds Cal Block: 35-PTB Group 1 valve. Examine when disassembled. 10" Velan Gate Valve (88904-1).	B-M-1 B12.40	VOL	-	-	-	-	-	-	-	-	X	-	-	-
036200	Valve 1RH-701-BLT Valve Bolting 10" Gate  Group 1 valve. Examine when the valve is disassembled or during the third period, whichever is first. 10" Velan Gate Valve (88904-1).	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-



POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				O U T A G E											
1	2	3	4	1	2	3	4	1	2	3	4				
038100	AC-10-SI-1001-13 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
038400	AC-10-SI-1001-14 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
038600	AC-10-SI-1001-15 Elbow to Tee Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
038700	AC-10-SI-1001-16 Tee to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
039000	AC-10-SI-1001-16-BC 2" Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
039100	AC-10-SI-1001-17 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
039200	AC-10-SI-1001-18 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
039300	AC-10-SI-1001-18A Pipe to Pipe Cal Block: 51-PTB	R-A R1.16	VOL	-	-	-	-	-	-	-	-	-	-	-	-
039400	AC-10-SI-1001-19 Pipe to Valve SI-867B Cal Block: 51-PTB Item No. R1.16 also applies.	R-A R1.11	VOL	X	-	-	-	-	-	-	-	-	-	-	-
039500	Valve SI-867B 10" Check Valve Internal Surface	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-

Group 3 valve. Examine when disassembled. 10" Darling Check Valve (11542).

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS															
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD							
				1	2	3	4	1	2	3	4	1	2	3	4				
039600	Valve 1SI-867B-BLT 10" Check Valve Bolting	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
Group 3 valve. Examine when the valve is disassembled or during the second period, whichever is first.																			
039700	RC-10-SI-1001-20 Valve SI-867B to Elbow Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
039800	RC-10-SI-1001-21 Elbow to Pipe Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
040000	RC-10-SI-1001-22 Pipe to Branch Connection Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
<u>RHR to Accumulator B Line / Dwg. No. ISI-PRI-1124</u>																			
041000	Valve 1RH-720 Valve Internal Surface	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
Group 1 valve. Examine when disassembled. 10" Velan Gate Valve (88904-1).																			
041100	Valve 1RH-720-Welds Two Valve Body Welds Cal Block: 35-PTB	B-M-1 B12.40	VOL	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
Group 1 valve. Examine when disassembled. 10" Velan Gate Valve (88904-1).																			
041200	Valve 1RH-720-BLT Valve Bolting 10" Gate	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
Group 1 valve. Examine when the valve is disassembled or during the third period, whichever is first. 10" Velan Gate Valve (88904-1).																			
041300	SIS-10-SI-1002-02 Valve RH-720 to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-	-
041500	SIS-10-SI-1002-03 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-	X	-	-	-

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INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
041600	SI-2501R-2-A2 Anchor	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
041700	SI-2501R-2-A2-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	X	-	-	-
041800	SIS-10-SI-1002-04 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
041900	SIS-10-SI-1002-05 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
042000	SIS-10-SI-1002-06 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
042200	SIS-10-SI-1002-07 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
042300	SIS-10-SI-1002-08 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
042400	SIS-10-SI-1002-09 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
042600	SI-2501R-2-SI1 Variable Spring	F-A F1.10C	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
042700	SIS-10-SI-1002-10 Pipe to Tee Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
<u>Accumulator A to Loop A / Dwg. No. ISI-PRI-1125</u>															
043000	Valve SI-842A Valve Internal Surface	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Group 3 valve. Examine when disassembled. 10" Darling Check Valve (11542).															
043100	Valve ISI-842A-BLT Valve Bolting 10" Check	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
Group 3 valve. Examine when the valve is disassembled or during the second period, whichever is first.															
043200	SIS-10-SI-1003-10 Valve SI-842A to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
043300	SIS-10-SI-1003-11 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	X	-	-
043400	SI-2501R-1-A6 Anchor	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	X	-	-
043500	SI-2501R-1-A6-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
043600	SIS-10-SI-1003-12 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	X	-	-
043700	SIS-10-SI-1003-13 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	X	-	-
043800	SIS-10-SI-1003-14 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	X	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
043900	SIS-10-SI-1003-15 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
044100	SI-2501R-1-SI4 Variable Spring	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
044200	SIS-10-SI-1003-16 Pipe to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
044300	SIS-10-SI-1003-17 Elbow to Pipe Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
044400	SIS-10-SI-1003-17-BC 2" Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
044500	SIS-10-SI-1003-18 Pipe to Elbow Cal Block: 51-PTB	R-A R1.16	VOL	-	-	-	-	-	-	-	-	-	-	-	-
044600	SIS-10-SI-1003-19 Elbow to Valve SI-867A Cal Block: 51-PTB Examination of this weld also satisfies the requirements of IN 97-19.	R-A R1.16	VOL	-	-	-	-	-	-	-	-	X	-	-	-
044700	Valve SI-867A Valve Internal Surface  Group 3 valve. Examine when disassembled. 10" Darling Check Valve (11542).	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
044800	Valve 1SI-867A-BLT Valve Bolting 10" Check  Group 3 valve. Examine when the valve is disassembled or during the second period, whichever is first.	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
044900	RC-10-SI-1003-20 Valve SI-867A to Elbow Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
045000	RC-10-SI-1003-21 Elbow to Branch Connection Cal Block: 51-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
----- Pressurizer Surge Line / Dwg. No. ISI-PRI-1126															
047100	RC-10-PZR-1001-01 Safe-End to Reducer Cal Block: 35-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
047200	RC-10-PZR-1001-02 Reducer to Pipe Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
047300	RC-10-PZR-1001-03 Pipe to Elbow Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
047400	RC-10-PZR-1001-04 Elbow to Pipe Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
047500	RC-10-PZR-1001-05 Pipe to Elbow Cal Block: 51-PTB	R-A R1.11	VOL	-	X	-	-	-	-	-	-	-	-	-	-
047600	RC-10-PZR-1001-06 Elbow to Pipe Cal Block: 51-PTB	R-A R1.11	VOL	-	X	-	-	-	-	-	-	-	-	-	-
047700	RC-2501R-2-RC3 Variable Spring	F-A F1.10C	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
048000	RC-2501R-2-RC2 Variable Spring	F-A F1.10C	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
048300	RC-10-PZR-1001-07 Pipe to Pipe Cal Block: 51-PTB Weld is located 9 ft 2 in from pipe bend.	R-A R1.11	VOL	-	X	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
048600	RC-2501R-2-RC1 Variable Spring	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
048900	RC-10-PZR-1001-11 Pipe to Elbow Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
049000	RC-10-PZR-1001-12 Elbow to Pipe Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
049100	RC-10-PZR-1001-13 Pipe to Branch Connection Cal Block: 51-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>Reactor Vessel Injection and Core Deluge to RPV / Dwg. No. ISI-PRI-1127</u>															
050000	Valve SI-853B Valve Internal Surface	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Group 4 valve. Examine when disassembled. 6" Velan Check Valve (78074)															
050100	Valve 1SI-853B-BLT 6" Check Valve Bolting	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
Group 4 valve. Examine when the valve is disassembled or during the first period, whichever is first.															
050200	AC-06-SI-1001-04 Valve SI-853B to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
050300	SI-2501R-5-SI10 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
050350	SI-2501R-5-SI10-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
050400	AC-06-SI-1001-05 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
050500	AC-06-SI-1001-06 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
050600	AC-06-SI-1001-07 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
050700	SI-2501R-5-SI9 Variable Spring	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
050800	SI-2501R-5-SI9-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
050900	AC-06-SI-1001-08 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
051000	AC-06-SI-1001-09 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
051100	AC-06-SI-1001-10 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
051200	SI-2501R-5-SI8 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
051300	SI-2501R-5-S221 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

Ganged with HB-19-R106. Examination will be performed at the same time for all three supports.

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
051500	AC-06-SI-1001-11 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
051600	AC-06-SI-1001-12 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
051650	SI-2501R-5-H205 Component Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Ganged with SI-2501R-5-SI7A and HB-19-H4A. Examination will be performed at the same time for both supports.															
051700	SI-2501R-5-SI7A Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Ganged with SI-2501R-5-H205 and HB-19-H4A. Examination will be done at the same time for all supports.															
051750	SI-2501R-5-SI7A-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
051800	AC-06-SI-1001-12A Pipe to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
051900	SI-2501R-5-HS-12 Snubber	F-A F1.10S	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
052000	SI-2501R-5-SI7B Rigid Support	F-A F1.10A	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
052100	SI-2501R-5-SI7B-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	X	-	-	-	-	-	-	-
052150	SI-2501R-5-H206 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
052200	AC-06-SI-1001-13 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
052300	AC-06-SI-1001-14 Elbow to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
052400	AC-06-SI-1001-15 Elbow to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
052500	AC-06-SI-1001-16 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
052600	SI-2501R-5-SI6 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
052700	AC-06-SI-1001-16-BC 2" Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
052800	SI-2501R-5-SI5 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
052900	AC-06-SI-1001-17 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
053000	AC-06-SI-1001-18 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
053100	AC-06-SI-1001-19 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
053200	AC-06-SI-1001-20 Elbow to Pipe Cal Block: 4-PTB	R-A R1.16	VOL	-	-	-	-	-	-	-	-	-	-	-	-
053300	AC-06-SI-1001-21 Pipe to Valve SI-853D Cal Block: 4-PTB	R-A R1.16	VOL	-	-	-	-	-	-	-	-	X	-	-	-
053400	Valve SI-853D Valve Internal Surface  Group 4 valve. Examine when disassembled. 6" Velan Check Valve (78074).	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
053500	Valve 1SI-853D-BLT 6" Check Valve Bolting  Group 4 valve. Examine when the valve is disassembled or during the first period, whichever is first.	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
053600	RC-06-SI-1001-22 Valve SI-853D to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
053700	RC-2501R-5-RC9 Rigid Support  Ganged with RC-2501R-1-R74. Examination will be performed for both supports at the same time.	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
053800	RC-2501R-5-RC9-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
053840	RC-2501R-5-R102 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
053850	RC-2501R-5-R102-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
053900	RC-06-SI-1001-23 Pipe to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
054100	RC-06-SI-1001-24 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
054200	RC-06-SI-1001-25 Pipe to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
054300	RC-06-SI-1001-26 Pipe to Elbow Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
054400	RC-06-SI-1001-27 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
054500	RC-06-SI-1001-28 Pipe to Elbow Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
054600	RC-06-SI-1001-29 Elbow to Pipe Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
054700	RC-06-SI-1001-30 Pipe to Elbow Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
054800	RC-06-SI-1001-31 Elbow to Reducer Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
054900	RC-04-SI-1001-32 Reducer to Safe-End Cal Block: 6-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
055000	RC-04-SI-1001-33 SI Safe-End to Nozzle (Az 288.5) Cal Block: 23-PTB Dissimilar metal weld. Weld is inaccessible from the outside surface.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
<u>Reactor Vessel Injection and Core Deluge to RPV / Dwg. No. ISI-PRI-1128</u>															
056000	Valve SI-845D 2" Check Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required														
056100	SIS-02-SI-1001-06 Valve SI-845D to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
056200	SIS-02-SI-1001-07 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
056300	26-07-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
	Component ID 1SI-878A-2														
056400	SIS-02-SI-1001-08 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
056500	SIS-02-SI-1001-08A Pipe to Coupling	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
056800	SIS-02-SI-1001-08B Coupling to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
057000	SIS-02-SI-1001-09 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
057100	SIS-02-SI-1001-10 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
057200	SIS-02-SI-1001-11 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
057300	SIS-02-SI-1001-12 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
057400	SIS-02-SI-1001-13 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
057500	SIS-02-SI-1001-14 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
057700	SIS-02-SI-1001-15 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
057800	SIS-02-SI-1001-16 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
058000	SIS-02-SI-1001-17 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
058100	SIS-02-SI-1001-18 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
058300	SIS-02-SI-1001-19 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
058400	SIS-02-SI-1001-20 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
058500	SIS-02-SI-1001-21 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
058600	SIS-02-SI-1001-22 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
058700	SIS-02-SI-1001-23 Pipe to Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>Core Deluge to RPV / Dwg. No. ISI-PRI-1129</u>															
059000	Valve SI-853A Valve Internal Surface	B-M-2 B12.50	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Group 4 valve. Examine when disassembled. 6" Velan Check Valve (78074).															
059100	Valve ISI-853A-BLT 6" Check Valve Bolting	B-G-2 B7.70	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
Group 4 valve. Examine when the valve is disassembled or during the first period, whichever is first.															
059200	SIS-06-SI-1002-04 Valve SI-853A to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
059300	SI-2501R-4-SI3 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
059400	SIS-06-SI-1002-06 Elbow to Elbow Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
059600	SIS-06-SI-1002-07- Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
059700	SIS-06-SI-1002-07-BC 2" Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
NRC IN 97-19 examination.															

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
059800	SI-2501R-4-HS-13 Snubber	F-A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
	Examination to be performed in conjunction with the snubber program.														
060000	SIS-06-SI-1002-08 Pipe to Elbow Cal Block: 4-PTB	R-A	VOL	-	-	-	-	-	-	-	-	-	-	-	-
060100	SIS-06-SI-1002-09 Elbow to Valve SI-853C Cal Block: 4-PTB	R-A	VOL	-	-	-	-	-	-	-	-	-	-	-	-
	Examination of this weld also satisfies the requirements of IN 97-19.														
060200	Valve SI-853C Valve Internal Surface	B-M-2	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
	Group 4 valve. Examine when disassembled. 6" Velan Check Valve (78074).														
060300	Valve 1SI-853C-BLT 6" Check Valve Bolting	B-G-2	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
	Group 4 valve. Examine when the valve is disassembled or during the first period, whichever is first.														
060400	RC-06-SI-1002-10 Valve SI-853C to Pipe Cal Block: 4-PTB	R-A	VOL	-	-	-	-	-	-	-	-	-	-	-	-
060500	RC-2501R-4-RC4 Variable Spring	F-A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
060800	RC-06-SI-1002-11 Pipe to Elbow Cal Block: 4-PTB	R-A	VOL	-	-	-	-	-	-	-	-	-	-	-	-
060900	RC-06-SI-1002-12 Elbow to Pipe Cal Block: 4-PTB	R-A	VOL	-	-	-	-	-	-	-	-	-	-	-	-
061000	RC-06-SI-1002-13 Pipe to Elbow Cal Block: 4-PTB	R-A	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
061100	RC-06-SI-1002-14 Elbow to Pipe Cal Block: 4-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
061200	RC-06-SI-1002-15 Pipe to Elbow Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
061300	RC-06-SI-1002-16 Elbow to Pipe Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
061400	RC-06-SI-1002-17 Pipe to Reducer Cal Block: 4-PTB Weld is not accessible.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
061500	RC-04-SI-1002-18 Reducer to Safe-End Cal Block: 6-PTB Weld could not be reached with the mechanized equipment from the inside surface. Weld is not accessible from the outside surface.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
061600	RC-04-SI-1002-19 SI Safe-End to Nozzle (Az 108.5) Cal Block: 23-PTB Dissimilar metal weld. Weld is inaccessible from the outside surface.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>RHR and Core Deluge to RPV / Dwg. No. ISI-PRI-1130</u>															
062000	Valve SI-845C 2" Check Valve  No examination required due to size.			-	-	-	-	-	-	-	-	-	-	-	-
062100	SIS-02-SI-1003-10 Valve SI-845C to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
062200	SIS-02-SI-1003-11 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
062300	SIS-02-SI-1003-12 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
062400	29-12-PSS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
062500	SIS-02-SI-1003-13 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
062600	SIS-02-SI-1003-14 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
062700	SIS-02-SI-1003-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
062800	SIS-02-SI-1003-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
062900	29-16-PSS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
063000	SIS-02-SI-1003-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
063100	SIS-02-SI-1003-18 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
063200	SIS-02-SI-1003-19 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
063300	SIS-02-SI-1003-20 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
063500	SIS-02-SI-1003-21 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
063600	SIS-02-SI-1003-22 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
063900	SIS-02-SI-1003-23 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
064000	SIS-02-SI-1003-24 Coupling to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
064300	SIS-02-SI-1003-25 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
064400	SIS-02-SI-1003-26 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
064500	SIS-02-SI-1003-27 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
064600	SIS-02-SI-1003-28 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
064700	SIS-02-SI-1003-29 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
064800	SIS-02-SI-1003-30 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
064900	SIS-02-SI-1003-31 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
065000	SIS-02-SI-1003-32 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
065100	SIS-02-SI-1003-33 Pipe to Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
<u>Pressurizer Safety / Dwg. No. ISI-PRI-1131</u>															
066100	RC-04-PR-1001-02 Safe-End to Elbow Cal Block: 3-PTB, 37-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
066200	RC-04-PR-1001-03 Elbow to Pipe Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
066300	RC-04-PR-1001-04 Pipe to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
066400	RC-04-PR-1001-05 Elbow to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
066500	RC-04-PR-1001-06 Elbow to Pipe Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
066600	RC-04-PR-1001-07 Pipe to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
066700	RC-04-PR-1001-08 Elbow to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
066800	RC-04-PR-1001-09 Elbow to Flange Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
066900	RC-04-PR-1001-09-FB Flange Bolting	B-G-2 B7.50	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
067000	Valve RC-435 4" Relief Valve  No examination required due to size.			-	-	-	-	-	-	-	-	-	-	-	-
068100	RC-04-PR-1002-02 Safe-End to Elbow Cal Block: 3-PTB, 37-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
068200	RC-04-PR-1002-03 Elbow to Pipe Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
068300	RC-04-PR-1002-04 Pipe to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
068400	RC-04-PR-1002-05 Elbow to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
068500	RC-04-PR-1002-06 Elbow to Pipe Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
068600	RC-04-PR-1002-07 Pipe to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
068700	RC-04-PR-1002-08 Elbow to Elbow Cal Block: 6-PTB Weld configuration and wall thickness prevent UT examination.	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
068800	RC-04-PR-1002-09 Elbow to Flange Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
068900	RC-04-PR-1002-09-FB Flange Bolting	B-G-2 B7.50	VT-1	X	-	-	-	-	-	-	-	-	-	-	-
069000	Valve RC-434 4" Relief Valve  No examination required due to size.			-	-	-	-	-	-	-	-	-	-	-	-
<u>Pressurizer Spray / Dwg. No. ISI-PRI-1132</u>															
070000	RC-03-AS-1001-01 Reducer to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
070100	RC-03-AS-1001-02 Elbow to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
071000	RC-03-PS-1002-24 Tee to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
071100	RC-2501R-1-RC8 Spring Hanger	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
071300	RC-03-PS-1002-25 Pipe to Reducer	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
072000	RC-04-PS-1001-01 Reducer to Tee Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
072100	RC-04-PS-1001-02 Tee to Pipe Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
072200	RC-04-PS-1001-03 Pipe to Elbow Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
072300	RC-04-PS-1001-04 Elbow to Elbow Cal Block: 6-PTB	R-A R1.11	VOL	-	-	-	-	X	-	-	-	-	-	-	-
072400	RC-04-PS-1001-05 Elbow to Elbow Cal Block: 6-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
072500	RC-04-PS-1001-06 Elbow to Pipe Cal Block: 6-PTB	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
072600	RC-04-PS-1001-07 Pipe to Safe-End Cal Block: 6-PTB	R-A R1.11	VOL	-	-	-	-	X	-	-	-	-	-	-	-
<u>Pressurizer Spray from Loop A / Dwg. No. ISI-PRI-1133</u>															
073000	RC-03-PS-1001-01 Branch Connection to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
073300	RC-03-PS-1001-02 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
073400	RC-03-PS-1001-03 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
073700	RC-03-PS-1001-04 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
073800	RC-03-PS-1001-05 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
074100	RC-03-PS-1001-06 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
074200	RC-03-PS-1001-07 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
074300	RC-2501R-1-A7 Anchor	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
074400	RC-2501R-1-A7-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
074600	RC-03-PS-1001-08 Pipe to Elbow	R-A R1.20	VOL	-	X	-	-	-	-	-	-	-	-	-	-
074700	RC-03-PS-1001-09 Elbow to Pipe	R-A R1.20	VOL	-	X	-	-	-	-	-	-	-	-	-	-
074800	RC-2501R-1-R68 Rigid Support	F-A F1.10B	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
Ganged with CH-2501R-4-R68. Examination will be performed at the same time for both supports.															
074850	RC-2501R-1-R68-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
074900	RC-2501R-1-R69 Rigid Support	F-A F1.10A	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
Ganged with CH-2501R-4-R69. Examination will be performed at the same time for both supports.															

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
075000	RC-2501R-1-R69-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
075100	RC-2501R-1-R70 Rigid Support	F-A F1.10B	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
075200	RC-2501R-1-R70-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
075300	RC-2501R-1-R73 Rigid Support	F-A F1.10A	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
Ganged with CH-2501R-4-R73. Examination will be performed at the same time for both supports.															
075350	RC-2501R-1-R73-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
075400	RC-2501R-1-R74 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Ganged with RC-2501R-5-RC9. Examination will be performed at the same time for both supports.															
075450	RC-2501R-1-R74-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
075500	RC-2501R-1-R75 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
075550	RC-2501R-1-R75-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
075600	RC-03-PS-1001-10 Pipe to Elbow	R-A R1.20	VOL	-	X	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
075700	RC-03-PS-1001-11 Elbow to Pipe	R-A R1.20	VOL	-	X	-	-	-	-	-	-	-	-	-	-
075900	RC-2501R-1-R78 Rigid Support	F-A F1.10A	VT-3	-	X	-	-	-	-	-	-	-	-	-	-
Ganged with CH-2501R-4-R78. Examination will be performed at the same time for both supports.															
076000	RC-2501R-1-R78-IWA Welded Attachment	B-K B10.20	SUR	-	X	-	-	-	-	-	-	-	-	-	-
076100	RC-03-PS-1001-12 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
076200	RC-03-PS-1001-13 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
076600	RC-03-PS-1001-14 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
076700	RC-03-PS-1001-15 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
077000	RC-2501R-1-A9 Anchor	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
077100	RC-2501R-1-A9-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
077300	RC-03-PS-1001-16 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
077400	RC-03-PS-1001-17 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
077500	RC-2501R-1-R91 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Ganged with RC-2501R-1-R91 on ISI-PRI-1134. Examination will be performed at the same time for both supports.															
077550	RC-2501R-1-R91-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
077600	RC-2501R-1-R92 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
077700	RC-03-PS-1001-18 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
077800	RC-03-PS-1001-19 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
077900	RC-2501R-1-R94 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Ganged with RC-2501R-1-R94. Examination will be performed at the same time for both supports.															
<u>Pressurizer Spary from Loop A / Dwg. No. ISI-PRI-1133</u>															
078000	RC-2501R-1-R95 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
<u>Pressurizer Spray from Loop A / Dwg. No. ISI-PRI-1133</u>															
078500	RC-03-PS-1001-20 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
078600	RC-2501R-1-RC-6 Spring Hanger	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
078700	RC-2501R-1-RC-6-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
078800	RC-03-PS-1001-21 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
078900	RC-03-PS-1001-22 Pipe to Valve RC-431A	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
079000	Valve RC-431A 3" Globe Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
079100	Valve RC-431A-BLT Valve Bolting 3" Globe		VT-1	-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
079200	RC-03-PS-1001-23 Valve RC-431A to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
079300	RC-03-PS-1001-24 Pipe to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
<u>Pressurizer Spray from Loop B / Dwg. No. ISI-PRI-1134</u>															
080000	RC-03-PS-1002-01 Branch Connection to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
080200	RC-03-PS-1002-02 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
080300	RC-03-PS-1002-03 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
080400	RC-03-PS-1002-04 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
080500	RC-03-PS-1002-05 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
080700	RC-03-PS-1002-06 Pipe to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
080900	RC-2501R-1-R32 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
081000	RC-2501R-1-RC5 Spring Hanger	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
081500	RC-2501R-1-R35 Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
081600	RC-03-PS-1002-07 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
081700	RC-03-PS-1002-08 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
081900	RC-2501R-1-R38 Component Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
082100	RC-03-PS-1002-09 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
082200	RC-2501R-1-R40 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
082250	RC-2501R-1-R40-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
082300	RC-03-PS-1002-10 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
082500	RC-03-PS-1002-11 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
082600	RC-03-PS-1002-12 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
082700	RC-2501R-1-A4 Anchor	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
082800	RC-2501R-1-A4-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
082900	RC-03-PS-1002-13 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
083000	RC-03-PS-1002-14 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
083100	RC-03-PS-1002-15 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
083300	RC-03-PS-1002-16 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
083400	RC-2501R-1-R91 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
	Ganged with RC-2501R-1-R91 on ISI-PRI-1133. Examination will be performed at the same time for both supports.														
083500	RC-2501R-1-R91-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
083600	RC-03-PS-1002-17 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
083700	RC-03-PS-1002-18 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
083900	RC-2501R-1-R94 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
	Ganged with RC-2501R-1-R94. Examination will be done for both supports at the same time.														
084400	RC-03-PS-1002-19 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
084500	RC-2501R-1-RC7 Constant Support	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
084600	RC-2501R-1-RC7-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
084700	RC-03-PS-1002-20 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
084900	RC-03-PS-1002-21 Pipe to Valve RC-431B	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
085000	Valve RC-431B 3" Globe Valve  No examination required			-	-	-	-	-	-	-	-	-	-	-	-
085200	RC-03-PS-1002-22 Valve RC-431B to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
085300	RC-03-PS-1002-23 Pipe to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>Pressurizer Power Operated Relief Valve Discharge / Dwg. No. ISI-PRI-1135</u>															
086000	RC-03-PSF-1002-01 Reducer to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
086100	RC-03-PSF-1002-02 Tee to Reducer	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
086200	RC-03-PSF-1002-03 Tee to Pipe	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
086300	RC-03-PSF-1002-04 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
086400	RC-03-PSF-1002-05 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
086600	RC-03-PSF-1002-06 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
086800	RC-03-PSF-1002-07R1 Elbow to Valve 1RC-515	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
086900	Valve RC-515 3" Gate Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
087200	RC-03-PSF-1002-08R1 Valve 1RC-515 to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
087300	RC-2501R-1-RC16 Variable Spring	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
087400	RC-2501R-2-HS22A Snubber	F-A F1.10S	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
087500	RC-03-PSF-1002-09 Pipe to Reducer	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
087600	RC-03-PSF-1002-09A Reducer to Valve RC-431C	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
087700	Valve RC-431C 3" Globe Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
088000	RC-03-PSF-1003-01 Tee to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
088100	RC-03-PSF-1003-02 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
088200	RC-03-PSF-1003-03 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
088300	RC-2501R-2-HS43 Snubber	F-A F1.10S	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
088400	RC-03-PSF-1003-04 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
088500	RC-2501R-2-RC14 Variable Spring	F-A F1.10C	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
088700	RC-03-PSF-1003-05 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
088900	RC-03-PSF-1003-06R1 Pipe to Valve 1RC-516	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
089000	Valve RC-516 3" Gate Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
089300	RC-03-PSF-1003-07R1 Valve 1RC-516 to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
089400	RC-2501R-2-RC15 Variable Spring	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
089500	RC-2501R-2-HS51 Snubber	F-A F1.10S	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
089600	RC-03-PSF-1003-08 Pipe to Reducer	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
089700	RC-03-PSF-1003-08A Reducer to Valve RC-430	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
089800	Valve RC-430 3" Globe Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
090100	RC-04-PSF-1001-02 Safe-End to Elbow Cal Block: 6-PTB, 37-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
090200	RC-04-PSF-1001-03 Elbow to Tee Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
090300	RC-04-PSF-1001-04 Tee to Reducer Cal Block: 6-PTB	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
090500	RC-2501R-2-HS15 Snubber	F-A F1.10S	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
	Examination to be performed in conjunction with the snubber program.														

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS																			
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD											
				1	2	3	4	O U T A G E				1	2	3	4								
<u>Loop A RTD Bypass / Dwg. No. ISI-PRI-1136</u>																							
091000	RC-02-BP-1001-01 Branch Connection to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091100	RC-02-BP-1001-A Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091200	RC-02-BP-1001-B Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091300	RC-02-BP-1001-C Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091400	RC-02-BP-1001-D Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091500	RC-02-BP-1001-03 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091600	RC-02-BP-1001-04 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091700	RC-02-BP-1001-08 Pipe to RTD Manifold	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091800	RC-02-BP-1001-09 RTD Manifold to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
091900	RC-02-BP-1001-14A Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-								

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
092000	RC-02-BP-1001-14B Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
092100	RC-02-BP-1001-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
092200	RC-02-BP-1001-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
092500	RC-02-BP-1001-16A Pipe to Orifice	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
092600	RC-02-BP-1001-16B Orifice to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
092900	RC-02-BP-1001-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
093000	RC-02-BP-1001-18 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
093100	RC-02-BP-1001-19 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
093200	RC-02-BP-1001-20 Coupling to Conc Swage Reducer	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
094000	RC-02-BP-1002-01A Cross to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
094100	RC-02-BP-1002-01 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
094200	RC-02-BP-1002-02 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
094300	RC-02-BP-1002-03 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
094400	RC-02-BP-1002-04 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
094500	RC-02-BP-1002-08 Pipe to RTD Manifold	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
094700	RC-02-BP-1002-09 RTD Manifold to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
094900	RC-02-BP-1002-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
095000	RC-02-BP-1002-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
095100	RC-02-BP-1002-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
095200	RC-02-BP-1002-18 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
095300	RC-02-BP-1002-19 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
095400	RC-02-BP-1002-20 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
095500	RC-02-BP-1002-21 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
095600	RC-02-BP-1002-22 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
095800	36-22-PSSA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
096200	RC-02-BP-1002-23 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
096300	RC-02-BP-1002-24 Elbow to Conc Swage Reducer	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
097000	RC-03-BP-1001-01 Conc Swage Reducer to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
097100	H-200 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
097200	RC-03-BP-1001-02 Conc Swage Reducer to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				O U T A G E											
				1	2	3	4	1	2	3	4	1	2	3	4
097300	RC-03-BP-1001-03 Tee to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
097500	RC-03-BP-1001-04 Pipe to Orifice	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
097600	RC-03-BP-1001-05 Orifice to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
097700	RC-03-BP-1001-06 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
097800	RC-03-BP-1001-07 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
097900	RC-03-BP-1001-08 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
098000	RC-03-BP-1001-09 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
098100	RC-03-BP-1001-11 Pipe to Pipe	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-
098200	H-204 Spring Hanger	F-A F1.10C	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
098400	RC-03-BP-1001-12 Pipe to Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
098500	H-A1004 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
098600	H-A1005 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
098700	H-A1006 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
<u>Loop B RTD Bypass / Dwg. No. ISI-PRI-1137</u>															
100000	RC-02-BP-1003-01 Cross to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
100100	RC-02-BP-1003-01A Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
100200	RC-02-BP-1003-02 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
100300	RC-02-BP-1003-03 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
100400	RC-02-BP-1003-04 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
100500	37-04-PS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
100600	RC-02-BP-1003-08 Pipe to RTD Manifold	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
100700	RC-02-BP-1003-09 RTD Manifold to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
100800	37-09-PS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
100900	RC-02-BP-1003-13 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
101000	RC-02-BP-1003-14 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
101100	RC-02-BP-1003-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
101200	RC-02-BP-1003-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
101300	RC-02-BP-1003-16A Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
101400	RC-02-BP-1003-16B Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
101800	37-16-PSA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
102000	RC-02-BP-1003-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
102100	RC-02-BP-1003-18 Elbow to Conc Swage Reducer	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103000	RC-02-BP-1004-E Branch Connection to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103100	RC-02-BP-1004-A Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103200	RC-02-BP-1004-B Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103300	RC-02-BP-1004-C Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103400	RC-02-BP-1004-D Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103500	RC-02-BP-1004-03 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103600	RC-02-BP-1004-04 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103700	RC-02-BP-1004-05 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
103800	RC-02-BP-1004-06 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
103900	RC-02-BP-1004-09 Pipe to RTD Manifold	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104000	RC-02-BP-1004-10 RTD Manifold to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104100	RC-02-BP-1004-13 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104200	RC-02-BP-1004-14 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104300	RC-02-BP-1004-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104400	RC-02-BP-1004-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104500	RC-02-BP-1004-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104600	RC-02-BP-1004-18 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104700	RC-02-BP-1004-19 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
104800	RC-02-BP-1004-20 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
105200	RC-02-BP-1004-21 Pipe to Orifice	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
105300	RC-02-BP-1004-22 Orifice to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
105600	RC-02-BP-1004-23 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
105700	RC-02-BP-1004-24 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
105800	RC-02-BP-1004-24A Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
105900	RC-02-BP-1004-25 Coupling to Conc Swage Reducer	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
107000	RC-03-BP-1002-01 Conc Swage Reducer to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
107100	RC-03-BP-1002-02 Conc Swage Reducer to Tee	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
107300	RC-03-BP-1002-03 Tee to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
107400	RC-03-BP-1002-04 Pipe to Orifice	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
107500	RC-03-BP-1002-05 Orifice to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
107600	RC-03-BP-1002-06 Pipe to Elbow	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
107700	RC-03-BP-1002-07 Elbow to Pipe	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
107800	RC-03-BP-1002-08 Pipe to Elbow	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
107900	RC-03-BP-1002-09 Elbow to Pipe	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
108000	RC-03-BP-1002-10 Pipe to Pipe	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
108100	RC-03-BP-1002-12 Pipe to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
108200	RC-03-BP-1002-13 Pipe to Branch Connection	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
108300	H-105G Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
108400	H-106G Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
108500	H-112G Rigid Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
108600	H-201 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
<u>Charging Line to/from Regenerative Heat Exchanger / Dwg. No. ISI-PRI-1138</u>															
109000	Valve CV-1298 3" Gate Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
109300	CVC-03-CH-1001-02M Pipe to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
109400	CVC-03-CH-1001-02R1 Valve CV-1298 to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
109500	CH-2501R-4-CH1 Spring Hanger	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
109600	CVC-03-CH-1001-03 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
109700	CVC-03-CH-1001-04 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
109800	CH-2501R-4-R84 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
110100	CVC-03-CH-1001-05 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
110200	CH-2501R-4-R81 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
110300	CVC-03-CH-1001-06 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
110400	CVC-03-CH-1001-07 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
110500	CVC-03-CH-1001-08 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
110800	CVC-03-CH-1001-09 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
110900	CVC-03-CH-1001-10 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
111000	CH-2501R-4-R78 Component Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
Ganged with RC-2501R-1-R78. Examination will be performed at the same time for both supports.															
111100	CH-2501R-4-R78-IWA Welded Attachment	B-K B10.20	SUR	X	-	-	-	-	-	-	-	-	-	-	-
111200	CVC-03-CH-1001-11 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
111300	CVC-03-CH-1001-12 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
111400	CVC-03-CH-1001-13 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
111500	CVC-03-CH-1001-14 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
111600	CVC-03-CH-1001-15 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
111700	CVC-03-CH-1001-16 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
111800	CH-2501R-4-R73 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Ganged with RC-2501R-1-R73. Examination will be performed at the same time for both supports.															
111900	CH-2501R-4-R73-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
112100	CH-2501R-4-R70 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Ganged with RC-2501R-1-R70. Examination will be performed at the same time for both supports.															
112200	CH-2501R-4-R70-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
112300	CVC-03-CH-1001-16A Pipe to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
112400	CH-2501R-4-R69 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				O U T A G E											
				1	2	3	4	1	2	3	4	1	2	3	4
112450	CH-2501R-4-R69-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
112500	CH-2501R-4-R68 Component Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Ganged with RC-2501R-1-R68. Examination will be performed at the same time for both supports.															
112550	CH-2501R-4-R68-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
112600	CVC-03-CH-1001-17 Pipe to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
112700	CVC-03-CH-1001-18 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
112900	CH-2501R-4-A8 Anchor	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
113000	CH-2501R-4-A8-IWA Welded Attachment	B-K B10.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
113100	CVC-03-CH-1001-19 Pipe to Valve RC-295	R-A R1.20	VOL	-	-	-	-	-	-	-	-	-	-	-	-
113200	Valve RC-295 3" Check Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
113300	Valve RC-295-BLT Valve Bolting 3" Check		VT-1	-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
113400	RC-03-CH-1001-20 Valve RC-295 to Elbow	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
113500	RC-03-CH-1001-21 Elbow to Pipe	R-A R1.20	VOL	-	-	-	-	-	-	-	-	X	-	-	-
113700	RC-03-CH-1001-22 Pipe to Elbow	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
113800	RC-03-CH-1001-23 Elbow to Pipe	R-A R1.11	VOL	-	-	-	-	-	-	-	-	X	-	-	-
114000	RC-03-CH-1001-24 Pipe to Elbow	R-A R1.11	VOL	-	-	-	-	-	-	-	-	X	-	-	-
114100	RC-03-CH-1001-25 Elbow to Pipe	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
114300	RC-03-CH-1001-26 Pipe to Elbow	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
114400	RC-03-CH-1001-27 Elbow to Branch Connection	R-A R1.11	VOL	-	-	-	-	-	-	-	-	-	-	-	-
<u>Auxiliary Charging / Dwg. No. ISI-PRI-1139</u>															
115000	39-16-PSSD Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
115100	CVC-02-ACH-1001-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	1	2	3	4	1	2	3	4
115200	CVC-02-ACH-1001-18 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
115300	39-18-PRA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
115400	39-18-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
115500	CVC-02-ACH-1001-19 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
115600	CVC-02-ACH-1001-20 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
115700	CVC-02-ACH-1001-21 Pipe to Valve CV-383	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
115800	Valve CV-383 2" Check Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
115900	CVC-02-ACH-1001-22 Valve CV-383 to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
116000	CVC-02-ACH-1001-23 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
116100	CVC-02-ACH-1001-24 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
116200	CVC-02-ACH-1001-25 Pipe to Branch Connection	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
-----															
<u>Auxiliary Spray / Dwg. No. ISI-PRI-1140</u>															
117000	Valve CV-296 2" Globe Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
-----															
117200	CVC-02-AS-1001-06 Valve CV-296 to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
-----															
117300	CVC-02-AS-1001-07 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
-----															
117400	CVC-02-AS-1001-08 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
-----															
117500	CVC-02-AS-1001-09 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
-----															
117600	SK2 Rigid Support	F-A F1.10B	VT-3	-	-	-	-	X	-	-	-	-	-	-	-
-----															
117700	CVC-02-AS-1001-10 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
-----															
117800	CVC-02-AS-1001-11 Pipe to Valve CV-297	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
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117900	Valve CV-297 2" Check Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
118000	CVC-02-AS-1001-12 Valve CV-297 to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
118100	CVC-02-AS-1001-13 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
118200	CVC-02-AS-1001-14 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
118300	CVC-02-AS-1001-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
118400	CVC-02-AS-1001-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
118500	40-16-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
118600	40-16-PRA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
118900	40-16-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
119000	CVC-02-AS-1001-17 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
119100	CVC-02-AS-1001-18 Coupling to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
119200	40-18-PRA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
119500	40-18-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
119800	40-18-PRC Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
120000	CVC-02-AS-1001-19 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
120100	CVC-02-AS-1001-20 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
120200	40-20-PSS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
120300	CVC-02-AS-1001-25 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
120400	CVC-02-AS-1001-26 Elbow to Pipe  Weld is inaccessible.	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
120500	40-26-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
120600	CVC-02-AS-1001-27 Pipe to Reducer  2" butt weld.	R-A R1.20	VOL	-	-	-	-	X	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS																			
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD											
				1	2	3	4	O U T A G E				1	2	3	4								
<u>Letdown / Dwg. No. ISI-PRI-1141</u>																							
121000	CVC-02-LD-1002-01A Nozzle to Pipe	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
121100	CVC-02-LD-1002-01 Pipe to Elbow	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
121200	CVC-02-LD-1002-02 Elbow to Pipe	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
121300	41-02-PR Rigid Support	F-A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-								
121400	CVC-02-LD-1002-03 Pipe to Valve CV-392	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
121500	Valve CV-392 2" Valve			-	-	-	-	-	-	-	-	-	-	-	-								
	No examination required due to size.																						
121600	CVC-02-LD-1002-04 Valve CV-392 to Pipe	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
121700	CVC-02-LD-1002-07 Pipe to Tee	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
121800	CVC-02-LD-1002-07A Tee to Reducing Insert	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								
121900	CVC-02-LD-1002-08 Tee to Pipe	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-								

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
122000	41-08-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
122100	CVC-02-LD-1002-09 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
122200	CVC-02-LD-1002-10 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
122250	CVC-02-LD-1002-10PS Component Support	F-A F1.10A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
Located in the Regenerative Heat Exchanger room. No drawings available.															
122300	CVC-02-LD-1002-11 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
122400	CVC-02-LD-1002-12 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
122500	41-12-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
122600	CVC-02-LD-1002-13 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
122700	CVC-02-LD-1002-14 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
122800	41-14-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
122900	CVC-02-LD-1002-15 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
123000	41-15-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
123100	CVC-02-LD-1002-16 Coupling to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
123200	41-16-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
123300	CVC-02-LD-1002-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
123400	CVC-02-LD-1002-18 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
123500	CVC-02-LD-1002-19 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
123600	CVC-02-LD-1002-20 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
123700	41-20-PR Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
123800	CVC-02-LD-1002-21 Pipe to Valve CV-200A	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
123900	Valve CV-200A 2" Globe Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
125000	CVC-02-LD-1003-01 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
125100	41-01-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
125200	CVC-02-LD-1003-02 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
125300	41-02-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
125400	CVC-02-LD-1003-03 Coupling to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
125500	41-03-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
125600	CVC-02-LD-1003-04 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
125700	CVC-02-LD-1003-05 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
125800	CVC-02-LD-1003-06 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
125900	CVC-02-LD-1003-07 Elbow to Pipe	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-
126000	41-07-PRB Rigid Support	F-A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
126100	CVC-02-LD-1003-08 Pipe to Valve CV-200B	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-
126200	Valve CV-200B 2" Globe Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
127000	CVC-02-LD-1004-01 Tee to Pipe	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-
127100	41-01-PRC Rigid Support	F-A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
127200	CVC-02-LD-1004-02 Pipe to Coupling	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-
127300	41-02-PRC Rigid Support	F-A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
127400	CVC-02-LD-1004-03 Coupling to Pipe	R-A	SUR	-	-	-	-	-	-	-	-	-	-	-	-
127500	41-03-PRC Rigid Support	F-A	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
127600	CVC-02-LD-1004-04 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
127700	CVC-02-LD-1004-05 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
127800	CVC-02-LD-1004-06 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
127900	CVC-02-LD-1004-07 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
128000	41-07-PRC Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
128100	CVC-02-LD-1004-08 Pipe to Valve CV-200C	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
128200	Valve CV-200C 2" Globe Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
<u>Reactor Coolant Pump A Seal Water Supply / Dwg. No. ISI-PRI-1142</u>															
129000	CVC-02-PSI-1001-39 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
129100	CVC-02-PSI-1001-40 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
129200	CVC-02-PSI-1001-41 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

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 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
129300	CVC-02-PSI-1001-42 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
129400	CVC-02-PSI-1001-43 Pipe to Flange	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
129500	CVC-02-PSI-1001-43-FB Flange Bolting	B-G-2 B7.50	VT-1	-	-	-	-	X	-	-	-	-	-	-	-
129600	CVC-02-PSI-1001-44 Flange to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
129700	CVC-02-PSI-1001-45 Pipe to Valve CV-304A	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
129800	Valve CV-304A 2" Check Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
129900	CVC-02-PSI-1001-46 Valve CV-304A to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
130000	CVC-02-PSI-1001-47 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
130100	CVC-02-PSI-1001-47A Tee to Insert	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
130200	CVC-02-PSI-1001-48 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	X	-	-	-	-	-	-	-



POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
131500	CVC-02-PSI-1001-05 Pipe to Valve CV-320A	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
131600	Valve CV-320A 2" Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
131700	CVC-02-PSI-1001-06 Valve CV-320A to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
131800	CVC-02-PSI-1001-07 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
131900	CVC-02-PSI-1001-08 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
132000	CVC-02-PSI-1001-09 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
132100	CVC-02-PSI-1001-10 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
132200	42-10-PSSA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
132300	42-10-PSSB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
132400	42-10-PSSC Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
132500	42-10-PSSD Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
132600	CVC-02-PSI-1001-11 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
132700	CVC-02-PSI-1001-12 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
132800	CVC-02-PSI-1001-13 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
132900	CVC-02-PSI-1001-14 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
133000	42-14-PSSA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
133100	42-14-PSSB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
133200	CVC-02-PSI-1001-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
133300	CVC-02-PSI-1001-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
133400	42-16-PS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
133500	CVC-02-PSI-1001-17 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
133600	CVC-02-PSI-1001-18 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
133700	42-18-PSS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
133800	CVC-02-PSI-1001-19 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
133900	CVC-02-PSI-1001-20 Coupling to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
134000	42-20-PSSA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
134100	42-20-PSSB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
134200	42-20-PSSC Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
134300	42-20-PSSD Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
134400	42-20-PSS E Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
134500	CVC-02-PSI-1001-21 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
134600	CVC-02-PSI-1001-21A Tee to Insert	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
134700	CVC-02-PSI-1001-22 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
134800	CVC-02-PSI-1001-23 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
134900	CVC-02-PSI-1001-24 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
135000	CVC-02-PSI-1001-25 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
135100	CVC-02-PSI-1001-26 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
135200	CVC-02-PSI-1001-27 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
135300	CVC-02-PSI-1001-28 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
135400	42-28-PRA Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
135500	42-28-PRB Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
135600	CVC-02-PSI-1001-29 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
Weld is inaccessible due to support 42-28-PRB.															
135700	CVC-02-PSI-1001-30 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
135800	CVC-02-PSI-1001-31 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
135900	CVC-02-PSI-1001-32 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
136000	42-32-PS Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
136100	CVC-02-PSI-1001-33 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
136200	CVC-02-PSI-1001-34 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
136300	CVC-02-PSI-1001-35 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
136400	CVC-02-PSI-1001-36 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
136500	CVC-02-PSI-1001-37 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
136600	CVC-02-PSI-1001-38 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
<u>Reactor Coolant Pump B Seal Water Supply / Dwg. No. ISI-PRI-1144</u>															
137000	Valve CV-304D 2" Check Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
137100	CVC-02-PSI-1002-02 Valve CV-304D to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
137200	CVC-02-PSI-1002-03 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
137300	CVC-02-PSI-1002-03A Tee to Insert	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
137400	CVC-02-PSI-1002-04 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
137500	CVC-02-PSI-1002-05 Pipe to Valve CV-320B	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
137600	Valve CV-320B 2" Valve			-	-	-	-	-	-	-	-	-	-	-	-
No examination required due to size.															
137700	CVC-02-PSI-1002-06 Valve CV-320B to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
137800	CVC-02-PSI-1002-07 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
137900	CVC-02-PSI-1002-08 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
138000	CVC-02-PSI-1002-09 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
138100	CVC-02-PSI-1002-10 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
138200	43-10-PHA Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
138300	43-10-PHB Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
138400	43-10-PHC Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
138500	43-10-PHD Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
138600	CVC-02-PSI-1002-11 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
138700	CVC-02-PSI-1002-12 Coupling to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
138800	43-12-PH Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
138900	CVC-02-PSI-1002-13 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139000	CVC-02-PSI-1002-14 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139100	CVC-02-PSI-1002-15 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139200	CVC-02-PSI-1002-16 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139300	CVC-02-PSI-1002-17 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139400	CVC-02-PSI-1002-17A Tee to Insert	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139500	CVC-02-PSI-1002-18 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139600	43-18-PH Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
139700	CVC-02-PSI-1002-19 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
139800	CVC-02-PSI-1002-20 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
139900	43-20-PH Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
140000	CVC-02-PSI-1002-21 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
140100	CVC-02-PSI-1002-22 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
140200	CVC-02-PSI-1002-23 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
140300	CVC-02-PSI-1002-24 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
140400	CVC-02-PSI-1002-25 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
140500	CVC-02-PSI-1002-26 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
<u>Reactor Coolant Pump B Seal Water Supply / Dwg. No. ISI-PRI-1145</u>															
141000	CVC-02-PSI-1002-27 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
141100	CVC-02-PSI-1002-28 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
INSERVICE INSPECTION LONG TERM PLAN  
CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
141200	CVC-02-PSI-1002-29 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
141300	CVC-02-PSI-1002-30 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
141400	43-30-PHA Rigid Support	F-A F1.10B	VT-3	X	-	-	-	-	-	-	-	-	-	-	-
141500	43-30-PHB Spring Hanger	F-A F1.10C	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
141700	CVC-02-PSI-1002-31 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
141800	CVC-02-PSI-1002-32 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
141900	43-32-PRA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
142000	43-32-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
142100	43-32-PRC Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
142200	CVC-02-PSI-1002-33 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
142300	CVC-02-PSI-1002-34 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
142400	43-34-PH Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
142500	CVC-02-PSI-1002-35 Pipe to Flange	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
142600	CVC-02-PSI-1002-35-FB Flange Bolting	B-G-2 B7.50	VT-1	-	-	-	-	X	-	-	-	-	-	-	-
142700	CVC-02-PSI-1002-36 Flange to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
142800	CVC-02-PSI-1002-37 Pipe to Valve CV-304B	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
142900	Valve CV-304B 2" Check Valve			-	-	-	-	-	-	-	-	-	-	-	-
	No examination required due to size.														
143000	CVC-02-PSI-1002-38 Valve CV-304B to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
143100	CVC-02-PSI-1002-39 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
143200	CVC-02-PSI-1002-40 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
143300	CVC-02-PSI-1002-41 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
143400	CVC-02-PSI-1002-42 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
143500	CVC-02-PSI-1002-43 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
143600	CVC-02-PSI-1002-44 Pipe to Reducer	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
<u>Letdown from Loop B Cold Leg / Dwg. No. ISI-PRI-1146</u>															
144000	RC-08-DR-1001-01 Branch Connection to Pipe Cal Block: 17-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
144100	RC-08-DR-1001-02 Pipe to Reducer Cal Block: 17-PTB	R-A R1.20	VOL	X	-	-	-	-	-	-	-	-	-	-	-
144200	RC-02-LD-1001-01 Reducer to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
144300	RC-02-LD-1001-02 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
144400	RC-02-LD-1001-03 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
144500	RC-02-LD-1001-04 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
144600	RC-02-LD-1001-05 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
145000	44-05-PR Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	X	-	-	-
145200	RC-02-LD-1001-06 Pipe to Tee	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
145300	RC-02-LD-1001-06A Tee to Insert	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
145400	RC-02-LD-1001-07 Tee to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
145700	44-07-PRA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
145900	44-07-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
146000	RC-02-LD-1001-08 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
146100	RC-02-LD-1001-09 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
146400	44-09-PRA Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-

POINT BEACH NUCLEAR PLANT - UNIT 1  
 INSERVICE INSPECTION LONG TERM PLAN  
 CLASS 1 COMPONENTS

SUMMARY NUMBER	EXAMINATION AREA IDENTIFICATION	CATEGORY ITEM NO	NDE METH	PLAN STATUS											
				FIRST PERIOD				SECOND PERIOD				THIRD PERIOD			
				1	2	3	4	O U T A G E				1	2	3	4
146700	44-09-PRB Rigid Support	F-A F1.10B	VT-3	-	-	-	-	-	-	-	-	-	-	-	-
147100	RC-02-LD-1001-10 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
147200	RC-02-LD-1001-11 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
147300	RC-02-LD-1001-12 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
147400	RC-02-LD-1001-13R1 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
147500	RC-02-LD-1001-14R1 Pipe to Elbow	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
147600	RC-02-LD-1001-15R1 Elbow to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
147700	RC-02-LD-1001-16R1 Pipe to Coupling	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
147800	RC-02-LD-1001-17R1 Coupling to Pipe	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-
148000	RC-02-LD-1001-18 Pipe to Valve RC-427	R-A R1.20	SUR	-	-	-	-	-	-	-	-	-	-	-	-