

February 16, 1989

Docket No. 50-263

Mr. D. M. Musolf, Manager  
Nuclear Support Services  
Northern States Power Company  
414 Nicollet Mall  
Minneapolis, Minnesota 55401

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EJORDAN  
BGRIMES

Dear Mr. Musolf:

SUBJECT: AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-22:  
MISCELLANEOUS ADMINISTRATIVE TS CHANGES (TAC NO. 64873)

The Commission has issued the enclosed Amendment No. 59 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated February 16, 1987, as revised August 17, 1988.

The amendment revises the plant TSs to: (1) clarify the location for monitoring airborne radioiodine and particulates consistent with and as set forth in the approved Offsite Dose Calculation Manual; (2) delete page 251a, incorrectly retained, and which should have been deleted by License Amendment No. 46 (July 1, 1986); (3) delete Section 6.8 which was superseded by the publication of 10 CFR 50.49, "Environmental qualification of electric equipment important to safety for nuclear power plants"; (4) standardize reports and correspondence to reflect addressee changes in conformance with 10 CFR 50.4; and (5) reflect the current ANSI standard referenced in the updated Quality Assurance Plan for Monticello operation (i.e., replace "Paragraph 4.4 of ANSI N18.7-1972" with "ANSI N18.7-1976 as modified by the Operational Quality Assurance Plan" on page 239 of the TSs).

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

original signed by

John J. Stefano, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III, IV, V  
& Special Projects

Enclosures:

1. Amendment No. 59 to License No. DPR-22
2. Safety Evaluation

cc w/enclosures:  
See next page

5520 NAME: MONTICELLO AMEND TAC 64873

|               |               |                 |
|---------------|---------------|-----------------|
| LA/PD31: DRSP | PM/PD31: DRSP | (A)D/PD31: DRSP |
| RINGRAM       | JSTEFANO      | TQUAY TRW       |
| 1/19/89       | 1/19/89       | 2/13/89         |

|               |               |
|---------------|---------------|
| CH/LPFB: DPLQ | CH/LHFB: DPLQ |
| KPERKINS      | WREGAN        |
| 1/ /89        | 1/ /89        |

OGC  
OPW JFOI  
2/2/89 '11

PRPB/DREP  
L. CUNNINGHAM  
1/23/89

Handwritten notes and signatures: 30, @/P-1, LQAB, S Weiss



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555  
February 16, 1989

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Sincerely,

A handwritten signature in cursive script, reading "John J. Stefano".

John J. Stefano, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III, IV, V  
& Special Projects

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License No. DPR-22
2. Safety Evaluation

cc w/enclosures:  
See next page

Mr. D. M. Musolf  
Northern States Power Company

Monticello Nuclear Generating Plant

cc:

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Wright County Board of  
Commissioners  
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Buffalo, Minnesota 55313

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John J. Stefano, Project Manager  
Project Directorate III-1  
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- Safety Evaluation

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| KPERKINS     | WREGAN       |
| 1/ /89       | 1/ /89       |

OGC  
CPW  
2/2/89

PRPB/DREP  
L. CUNNINGHAM  
1/23/89

30  
S Weiss



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 59  
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northern States Power Company (the licensee) dated February 16, 1987, as revised August 17, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.59 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3: This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Theodore R Quay*

Theodore R. Quay, Acting Director  
Project Directorate III-1  
Division of Reactor Projects - III, IV, V  
& Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 16, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 59

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

INSERT

|       |       |
|-------|-------|
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| 239   | 239   |
| 248   | 248   |
| 249   | 249   |
| 251   | 251   |
| 251a  | -     |
| 252   | 252   |
| 253   | -     |

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**MONTICELLO NUCLEAR GENERATING PLANT  
RADIATION ENVIRONMENTAL MONITORING PROGRAM  
SAMPLE COLLECTION AND ANALYSIS**

| <u>Exposure Pathway<br/>and/or Sample</u>           | <u>Number of Samples<br/>and<br/>Sample Locations**</u>   | <u>Sampling and<br/>Collection Frequency</u>                       | <u>Type and Frequency<br/>of Analysis</u>  |
|---|---|--|--|
| 1. <u>Airborne</u><br>Radioiodine &<br>Particulates | Samples from 5 locations:<br>3 samples from offsite<br>locations (in different<br>sectors) of the highest<br>calculated annual average<br>ground level D/Q, 1 sample<br>from the vicinity of a com-<br>munity having the highest<br>calculated annual average<br>ground-level D/Q, and<br>1 sample from a control<br>location specified in the<br>ODCM. | Continuous Sampler<br>operation with sampler<br>collection weekly. | Radioiodine analysis<br>Weekly for I-131<br><br>Particulate:<br>Gross beta activity<br>on each filter weekly*.<br>Analyses shall be per-<br>formed more than 24<br>hours following filter<br>change. Perform gamma<br>isotopic analysis on<br>composite (by location)<br>sample quarterly. |
| 2. <u>Direct</u><br><u>Radiation</u>                | 37 TLD stations established<br>with duplicate dosimeters<br>placed at the following<br>locations:   | Quarterly  | Gamma Dose quarterly   |

\* If gross beta activity in any indication sample exceeds 10 times the yearly average of the control sample, a gamma isotopic analysis is required.

\*\* Sample locations are given on the figure and table in the ODCM.

- f. Investigation of all Reportable Events and Events requiring Special Reports to the Commission.
  - g. Revisions to the Facility Emergency Plan, the Facility Security Plan, and the Fire Protection Program.
  - h. Operations Committee minutes to determine if matters considered by that Committee involve unreviewed or unresolved safety questions.
  - i. Other nuclear safety matters referred to the SAC by the Operations Committee, plant management or company management.
  - j. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems, or components.
  - k. Reports of special inspections and audits conducted in accordance with specification 6.3.
  - l. Changes to the Offsite Dose Calculation Manual (ODCM).
  - m. Review of investigative reports of unplanned releases of radioactive material to the environs.
6. Audit - The operation of the nuclear power plant shall be audited formally under the cognizance of the SAC to assure safe facility operation.
- a. Audits of selected aspects of plant operation, as delineated in ANSI N18.7-1976 as modified by the Operational Quality Assurance Plan, shall be performed with a frequency commensurate with their nuclear safety significance and in a manner to assure that an audit of all nuclear safety-related activities is completed within a period of two years. The audits shall be performed in accordance with appropriate written instructions and procedures.
  - b. Audits of aspects of plant radioactive effluent treatment and radiological environmental monitoring shall be performed as follows:
    - 1. Implementation of the Offsite Dose Calculation Manual and quality controls for effluent monitoring at least once every two years.
    - 2. Implementation of the Process Control Program for solidification of radioactive waste at least once every two years.
    - 3. The Radiological Environmental Monitoring Program and the results thereof, including quality controls, at least once every year.
  - c. Periodic review of the audit program should be performed by the SAC at least twice a year to assure its adequacy.
  - d. Written reports of the audits shall be reviewed by the Vice President Nuclear Generation, by the SAC at a scheduled meeting, and by members of Management having responsibility in the areas audited.

## 6.7 REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the U.S. Nuclear Regulatory Commission, Attn: Document Control Desk, Washington, DC 20555, unless otherwise noted.

### A. Routine Reports

#### 1. Startup Report

A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

2. Occupational Exposure Report. (1) An annual report of occupational exposure covering the previous calendar year shall be submitted prior to March 1 of each year.

The report should tabulate on an annual basis the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man-rem exposure according to work and job functions, e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing, and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

3. Monthly Operating Report. A monthly report of operating statistics and shutdown experience covering the previous month shall be submitted by the 15th of the following month.

4. Semiannual Radioactive Effluent Release Report. Routine radioactive effluent release reports covering the operation of the unit during the previous six months of operation shall be submitted within 60 days after January 1st and July 1st of each year.

The radioactive effluent release reports shall include a summary of the quantities of radioactive liquid and gaseous effluents as outlined in Appendix B of Regulatory Guide 1.21, Revision 1, June, 1974, with data summarized on a quarterly basis.

The report to be submitted 60 days after January 1st of each year shall include an assessment of the radiation doses from radioactive effluents released from the plant during the previous calendar year. This same report shall also include an assessment of the radiation doses from radioactive liquid and gaseous effluents to individuals due to their activities inside the site boundary (Figures 3.8.1 and 3.8.2) during the report period. All assumptions used in making these assessments (i.e., specific activity, exposure time and location) shall be included in these reports. The assessment of radiation doses shall be performed in accordance with the Offsite Dose Calculation Manual (ODCM) or standard NRC computer codes.

1/ This report supplements the requirements of 10CFR20, Section 20.407. If 10CFR20, Section 20.407 is revised to include such information, this Specification is unnecessary.

**C. Environmental Reports**

**1. Annual Radiation Environmental Monitoring Report**

- a. Annual Radiation Environmental Monitoring Reports covering the operation of the program during the previous calendar year shall be submitted prior to May 1 of each year.
- b. The Annual Radiation Environmental Monitoring Reports shall include summaries, interpretations, and an analysis of trends of the results of the radiological environmental surveillance activities for the report period, including a comparison with preoperational studies, operational controls (as appropriate), and previous environmental surveillance reports and an assessment of the observed impacts of the plant operation on the environment. The reports shall also include the results of land use census required by Specification 4.16.B.1. If harmful effects or evidence of irreversible damage are detected by the monitoring, the report shall provide an analysis of the problem and a planned course of action to alleviate the problem.
- c. The Annual Radiation Environmental Monitoring Reports shall include summarized and tabulated results in the format of Regulatory Guide 4.8, December 1975 of all radiological environmental samples taken during the report period. In the event that some results are not available for inclusion with the report, the report shall be submitted noting and explaining the reasons for the missing results. The missing data shall be submitted as soon as possible in a supplementary report.
- d. The reports shall also include the following: a summary description of the radiological environmental monitoring program; a map of all sampling locations keyed to a table giving distances and directions from the reactor; and the results of licensee participation in the Interlaboratory Comparison Program, required by Specification 4.16.C.1.

## 2. Environmental Special Reports

When radioactivity levels in samples exceed limits specified in Table 4.16.3 an Environmental Special Report shall be submitted within 30 days from the end of the affected calendar quarter. For certain cases involving long analysis time, determination of quarterly averages may extend beyond the 30 day period. In these cases the potential for exceeding the quarterly limits will be reported within the 30 day period to be followed by the Environmental Special Report as soon as practicable.

## 3. Other Environmental Reports (non-radiological, non-aquatic)

- a. Environmental events that indicate or could result in a significant environmental impact causally related to plant operation. The following are examples: excessive bird impact; onsite plant or animal disease outbreaks; unusual mortality of any species protected by the Endangered Species Act of 1973; increase in nuisance organisms or conditions; or excessive environmental impact caused by herbicide application to transmission corridors associated with the plant. This report shall be submitted within 30 days of the event and shall (a) describe, analyze, and evaluate the event, including extent and magnitude of the impact and plant operating characteristics, (b) describe the probable cause of the event, (c) indicate the action taken to correct the reported event, (d) indicate the corrective action taken to preclude repetition of the event and to prevent similar occurrences involving similar components or systems, and (e) indicate the agencies notified and their preliminary responses.
- b. Proposed changes, tests or experiments which may result in a significant increase in any adverse environmental impact which was not previously reviewed or evaluated in the Final Environmental Statement or supplements thereto. This report shall include an evaluation of the environmental impact of the proposed activity and shall be submitted 30 days prior to implementing the proposed change, test or experiment.

## D. Special Reports

Unless otherwise indicated, special reports required by the Technical Specification shall be submitted within the time period specified for each report.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 59 TO FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

By letter dated February 16, 1987, the licensee proposed changes to the plant Technical Specifications (TSs) which would: (1) reference the NRC approved Offsite Dose Calculation Manual (ODCM) for Monticello in lieu of specifying the radiation environmental program control locations for monitoring airborne radioiodine and particulates; (2) delete page 251a, incorrectly retained, and which should have been removed by the issuance of License Amendment No. 46 on July 1, 1986; (3) delete Section 6.8 which was superseded by the publication of 10 CFR 50.49, "Environmental qualification of electric equipment important to safety for nuclear power plants"; (4) standardize reports and correspondence to reflect addressees set forth in 10 CFR 50.4; and (5) identify the current standard for operations personnel qualifications referenced in the updated Monticello Operational Quality Assurance (QA) Plan, replacing "Paragraph 4.4 of ANSI N18.7-1972" with "ANSI N18.7-1976 as modified by the Operational Quality Assurance Plan", on page 239 of the TSs.

The change proposed in the February 16, 1987, application to revise the plant organization chart figures in Section 6 of the TSs was withdrawn by the licensee's letter dated August 17, 1988, and is not being considered for this license amendment. The August 17, 1988, letter also furnished additional information in support of the change noted above pertaining to the referencing of the ODCM in the TSs.

2.0 DISCUSSION AND EVALUATION

Proposed Change No. 1

In Table 4.16.1 of the TSs, under the column entitled "Number of Samples and Sample Locations," replace the statement "...and 1 sample from a control location 8-20 miles distance and in the least prevalent wind direction" with "... and 1 sample from a control location specified in the ODCM."

The control location in question for monitoring airborne radioiodine and particulates is identified in Figure 5.1-1 of the ODCM as "Air Station M-1," which is about 11 miles northwest of the plant. The prevailing wind is from

the northwest direction. Since the control location is about 11 miles from the plant in the general direction of the prevailing wind, the licensee believes that the current control sample meets the intent of NRC requirements; however, since wind patterns may change slightly from year to year, the licensee maintains that a literal interpretation of the existing requirement as specified in the TSs could result in an unnecessary change in control location each year. Since the control location should remain fixed, the location specified in the ODCM is a "fixed" location, and referencing the ODCM in the TSs is a change which serves to clarify the control location for monitoring airborne radioiodine and particulates. The change in no way alters the intent of the TSs to be consistent with the plant's environmental monitoring program, nor does it reduce the level of protection provided to the environment.

We agree with the licensee's basis for this proposed change. The sampling control location should be consistent with past sampling and not be dependent upon prevailing wind direction, which historically has been found to change slightly from year to year. Therefore, we find this change to be acceptable.

#### Proposed Change No. 2

Delete page 251a from the TSs.

We have reviewed the licensee's amendment request dated September 14, 1984, which resulted in the issuance of license Amendment No. 46 on July 1, 1986, and agree that the reporting requirements delineated on page 251a are no longer in force due to the issuance of license Amendment No. 46 and should have been deleted. We therefore find the deletion of page 251a to be acceptable.

#### Proposed Change No. 3

Delete Section 6.8, page 253, from the TSs.

We agree with the licensee that the publication of 10 CFR 50.49, "Environmental qualification of electric equipment important to safety for nuclear power plants" addresses the requirements set forth in Section 6.8 of the TSs and that deletion of Section 6.8 from the TSs is therefore considered acceptable.

#### Proposed Change No. 4

Incorporate the following changes to pages 248 through 252 of the TSs:

- Page 248 - replace "Director of the appropriate Regional Office of Inspection and Enforcement" with "U.S. Nuclear Regulatory Commission, Attn: Document Control Desk, Washington, DC 20555",
- Page 249 - delete "to the Office of Management Information and Program Control, U.S. Nuclear Regulatory Commission, Washington, DC 20555",
- Page 251 - delete "The reports listed below shall be submitted to the Administrator of the appropriate Regional Office or designate:",

- ° Page 252 - delete "Written reports for the following items shall be submitted to the appropriate NRC Regional Administrator:", and
- ° Page 252 - delete "to the appropriate NRC Regional Administration".

The changes proposed are acceptable since they are consistent with the standardized mailing address for any correspondence with the NRC and the transmittal of TS required reports as reflected in 10 CFR 50.4.

#### Proposed Change No. 5

On page 239 of the TSs, replace "Paragraph 4 of ANSI N18.7-1972" with "ANSI N18.7-1976 as modified by the Operational Quality Assurance Plan."

The proposed change is acceptable to us since it updates the TSs to indicate and be consistent with the more stringent and current ANSI standard implemented in the Monticello Operational Quality Assurance Plan regarding operating personnel qualifications.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes in surveillance requirements. The amendment also relates to changes in recordkeeping, reporting, or administrative procedures or requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

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Dated: February 16, 1989