

February 28, 1989

Docket No. 50-263

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Mr. D. M. Musolf, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

SUBJECT: AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-22:
MISCELLANEOUS ADMINISTRATIVE CHANGES (TAC NO. 69303)

The Commission has issued the enclosed Amendment No.60 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. This amendment consists of changes to the Technical Specifications (TSs) in response to your application dated August 31, 1988.

The amendment revises the plant TSs to: (1) correct an error in the existing TS action statement 3.7.B.1.b. to specify that the Standby Gas Treatment System is required, consistent with the definition of Secondary Containment Integrity contained in Section 1.0 of the TSs; (2) incorporate Specification 4.7.A.2.d which was inadvertently deleted by License Amendment No. 55; and (3) make other editorial corrections to achieve consistency throughout the TSs.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

John J. Stefano, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Enclosures:

1. Amendment No. 60 to License No. DPR-22
2. Safety Evaluation

cc w/enclosures:
See next page

DF01
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(A)D/PD31:DRSP
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C/P-1

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

February 28, 1989

Docket No. 50-263

Mr. D. M. Musolf, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

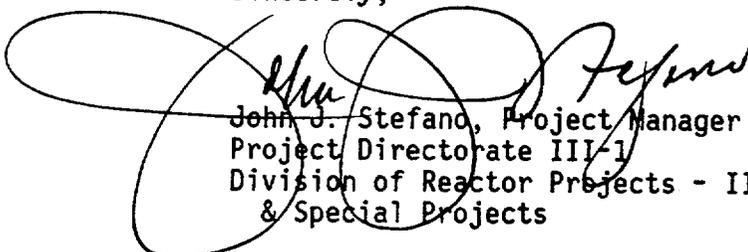
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John J. Stefano, Project Manager
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Division of Reactor Projects - III, IV, V
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2. Safety Evaluation

cc w/enclosures:
See next page

Mr. D. M. Musolf, Manager
Northern States Power Company

Monticello Nuclear Generating Plant

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 60
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated August 31, 1988, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

8903130470 890228
PDR ADCK 05000263
F PIC

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 60, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Theodore R Quay

Theodore Quay, Acting Director
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 28, 1989

ATTACHMENT TO LICENSE AMENDMENT NO. 60

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
16	16
17	--
158	158
160	160
161	161
167	167

Bases Continued:

that indicated by the neutron flux at the scram setting. Analyses demonstrate that, with a 120% scram trip setting, none of the abnormal operational transients analyzed violate the fuel Safety Limit and there is a substantial margin from fuel damage. Therefore, the use of flow referenced scram trip provides even additional margin.

For operation in the startup mode while the reactor is at low pressure, the IRM scram setting of 20% of rated power provides adequate thermal margin between the setpoint and the safety limit, 25% of rated. The margin is adequate to accommodate anticipated maneuvers associated with power plant startup. Effects of increasing pressure at zero or low void content are minor, cold water from sources available during startup is not much colder than that already in the system, temperature coefficients are small, and control rod patterns are constrained to be uniform by operating procedures backed up by the rod worth minimizer. Worth of individual rods is very low in a uniform rod pattern. Thus, of all possible sources of reactivity input, uniform control rod withdrawal is the most probable cause of significant power rise. Because the flux distribution associated with uniform rod withdrawals does not involve high local peaks, and because several rods must be moved to change power by a significant percentage of rated power, the rate of power rise is very slow. Generally, the heat flux is in near equilibrium with the fission rate. In an assumed uniform rod withdrawal approach to the scram level, the rate of power rise is no more than 5% of rated power per minute, and the IRM system would be more than adequate to assure a scram before the power could exceed the safety limit. The IRM scram remains active until the mode switch is placed in the run position and the associated APRM is not downscale. This switch occurs when reactor pressure is greater than 850 psig.

The operator will set the APRM neutron flux trip setting no greater than that stated in Specification 2.3.A.1. However, the actual setpoint can be as much as 3% greater than that stated in Specification 2.3.A.1 for recirculation driving flows less than 50% of design and 2% greater than that shown for recirculation driving flows greater than 50% of design due to the deviations discussed on page 39.

B. Deleted

3.0 LIMITING CONDITIONS FOR OPERATION

2. Primary Containment Integrity

- a. Primary Containment Integrity, as defined in Section 1, shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212°F and fuel is in the reactor vessel, except when performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed 5 MW(t). Without Primary Containment Integrity, restore Primary Containment Integrity within one hour or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

4.0 SURVEILLANCE REQUIREMENTS

2. Primary Containment Integrity

- a. Primary Containment Integrity shall be demonstrated after each closing of each penetration subject to Type B testing, if opened following a Type A or Type B test, by leak rate testing the seal with gas at \geq Pa, 42 psig, and verifying that when the measured leakage rate for these seals is added to the leakage rates determined pursuant to Surveillance Requirement 4.7.A.2.b.5 for all other Type B and C penetrations, the combined leakage rate is less than or equal to 0.6La.

3.7/4.7

Amendment No. 30,55,60

3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

4. The accuracy of each Type A test shall be verified by a supplemental test which:
 - a. Confirms the accuracy of the test by verifying that the difference between the supplemental data and the Type A test data is within $0.25L_a$, and
 - b. Has duration sufficient to establish accurately the change in leakage rate between the Type A test and the supplemental test, and
 - c. Requires the rate of gas injected into the containment or bled from the containment during the supplemental test to be limited between 75 to 125% of L_a .

5. Type B and C tests shall be conducted with gas at $\geq P_a$ at each refueling shutdown (maximum interval of 24 months), except for tests involving the main steam line isolation valves. Main steam isolation valve tests shall be conducted with gas at ≥ 25 psig each 18 months. A combined leakage rate of $\leq 0.6 L_a$ shall be demonstrated for all penetrations and valves, except for main steam line isolation valves, subject to Type B and C tests. A leakage rate of ≤ 11.5 scf per hour shall be demonstrated for each main steam line isolation valve.

3.7/4.7

3.0 LIMITING CONDITIONS FOR OPERATION

- c. When Primary Containment Integrity is required, the primary containment airlock shall be operable with:
1. Both doors closed except when the airlock is being used, then at least one airlock door shall be closed, and
 2. An overall airlock leakage rate of less than or equal to 0.05 La at Pa or 0.007La at 10 psig.

With the primary containment airlock inoperable, maintain at least one airlock door closed and restore the airlock to Operable status within 24 hours or be in at least Hot Shutdown within the next 12 hours and in Cold Shutdown within the following 24 hours.

4.0 SURVEILLANCE REQUIREMENTS

- c. The primary containment airlock shall be demonstrated operable:
1. At each refueling shutdown, and at six month intervals thereafter, by conducting an overall airlock leakage test at $\geq P_a$ and demonstrating that overall airlock leakage rate is $\leq 0.05 L_a$. This test interval may be extended up to the next refueling outage (up to a maximum interval between tests at P_a of 24 months) if there have been no air lock openings since the last successful test at P_a .
 2. After each opening by conducting an overall airlock leakage test at ≥ 10 psig and verifying the leakage rate is $\leq 0.007 L_a$. If the airlock is being used for multiple openings, this test is not required after each opening, but shall be performed at least once per 72 hours.
 3. At six month intervals by verifying that only one door can be opened at a time. If the airlock has not been used since the last door interlock test, this test is not required.
- d. The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration.

3.0 LIMITING CONDITIONS FOR OPERATION

- b. If both standby gas treatment system circuits are not operable, within 36 hours the reactor shall be placed in a condition for which the standby gas treatment system is not required in accordance with Specification 3.7.C.2.(a) through (d).

2. Performance Requirements

a. Periodic Requirements

- (1) The results of the in-place DOP tests at 3500 cfm ($\pm 10\%$) on HEPA filters shall show $\leq 1\%$ DOP penetration.
- (2) The results of in-place halogenated hydrocarbon tests at 3500 cfm ($\pm 10\%$) on charcoal banks shall show $\leq 1\%$ penetration.
- (3) The results of laboratory carbon sample analysis shall show $>90\%$ methyl iodine removal efficiency when tested at 130°C , 95% R.H.

4.0 SURVEILLANCE REQUIREMENTS

- b. If both standby gas treatment system circuits are not operable within 7 days, within 36 hours verify that the conditions of Specification 3.7.C.2.(a) through (d) are satisfied.

2. Performance Requirement Tests

- a. At least once per 720 hours of system operation; or once per operating cycle, but not to exceed 18 months, whichever occurs first; or following painting, fire, or chemical release in any ventilation zone communicating with the system while the system is operating that could contaminate the HEPA filters or charcoal absorbers, perform the following:
 - (1) In-place DOP test the HEPA filter banks.
 - (2) In-place test the charcoal adsorber banks with halogenated hydrocarbon tracer.
 - (3) Remove one carbon test canister from the charcoal adsorber. Subject this sample to a laboratory analysis to verify methyl iodine removal efficiency.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 60 TO FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY
MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

By letter dated August 31, 1988, Northern States Power Company (NSP or the licensee) submitted an application for amendment of the Technical Specifications (TSs) appended to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The application proposed several administrative type changes. The specific changes and the reason for each change are as follows:

- a. Delete the period on the last line of page 16 of the existing TSs, relocate the contents on TS page 17 to page 16, and add a note on page 16 that the next page is page 18 (i.e., page 17 of the TSs is being deleted).

The reason for this change is that a period was inadvertently added to the last line on page 16 making an incomplete sentence; i.e., the sentence in question continues on page 17 of the existing TSs. In addition, the entire contents on page 17 are being moved to page 16 in the interest of continuity and to make the paragraph easier to read, with page 17 being deleted, accordingly.

- b. In Paragraph 4.7.A.2.a, on page 158 of the TS, line 10, change "4.7.A.2.b.4" to "4.7.A.2.b.5," and on page 160, renumber Paragraph 4.7.A.2.b.4.d to Paragraph 4.7.A.2.b.5.

The reason for this change is that Paragraph 4.7.A.2.b.4.d is not related to the other items in Paragraph 4.7.A.2.b.4 and should be made a separate subparagraph to 4.7.A.2.b. Paragraph 4.7.A.2.b.4 and subitems a., b. and c. describe the surveillance requirements for Type A tests to verify primary containment integrity, where existing Paragraph 4.7.A.2.b.4.d describes the surveillance requirements for Type B and Type C tests for verifying primary containment integrity. Therefore, subitem d. of Paragraph 4.7.A.2.b.4 is being renumbered as Paragraph 4.7.A.2.b.5 to effect this separation in specifying the Type A versus the Type B and Type C test requirements.

- c. Add Paragraph 4.7.A.2.d to TS page 161 to read: "The interior surfaces of the drywell shall be visually inspected each operating cycle for evidence of deterioration."

The reason for this addition is that Paragraph 4.7.A.2.d was inadvertently deleted from the TS revision implemented by License Amendment No. 55.

- d. In Paragraphs 3.7.B.1.b and 4.7.B.1.b, on page 167 of the existing TSs, change "Specification 3.7.C.1.(a) through (d)" to "Specification 3.7.C.2.(a) through d".

The reason for this change is that Specification 3.7.C.1.(a) through (d) do not exist. The Standby Gas Treatment System (SGTS) is required to be operable when Secondary Containment Integrity is required (per the definition in Section 1.0 of the existing TSs), and the correct action statement and surveillance requirement reference should be 3.7.C.2 and 4.7.C.2, respectively, which describe when Secondary Containment Integrity is needed.

2.0 EVALUATION

We have reviewed each of the proposed changes discussed above, including the reason for each of the proposed changes, and agree that they represent changes that are purely administrative for achieving technical consistency in the TS sections (paragraphs) changed and for correcting obvious errors. Accordingly, we find that all of the TS changes proposed and described above are acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves administrative changes to correct errors in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and in surveillance requirements. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: John J. Stefano

Dated: February 28, 1989