November 18, 1987

Docket No. 50-263

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Mr. D. M. Musolf, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

The Commission has issued the enclosed Amendment No. 52 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. This amendment is in response to your application dated April 9, 1987, as supplemented by letter dated June 15, 1987.

The amendment revises the Technical Specifications to reflect the extension by approximately one-month of the interval for performance of the containment integrated leak rate test in order to coincide with the 1989 refueling cycle.

A copy of our related Safety Evaluation is also enclosed. The notice of issuance will be included in the Commission's next regular biweekly Federal Register notice.

The issuance of this amendment completes our work effort under TAC Numbers 65322 and 65323.

Sincerely,

Dominic C. DiIanni, Project Manager
Project Directorate III-3
Division of Reactor Projects

Enclosures:
1. Amendment No. 52 to License No. DPR-22
2. Safety Evaluation

cc w/enclosures:
See next page

*See previous concurrence

Office:	LA/PDIII-3	PM/PDIII-3	PD/PDIII-3	OGC
Surname:	*PKreutzer	*DDiIanni/tg	KEPerkins	*CBarth
Date:	11/2 /87	11/02/87	11/18/87	11/05/87

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Mr. D. M. Musolf
Northern States Power Company

Monticello Nuclear Generating Plant

cc:
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated April 9, 1987, as supplemented June 15, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Kenneth E. Perkins, Director
Project Directorate III-3
Division of Reactor Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 18, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 52

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
157	157

3.0 LIMITING CONDITIONS FOR OPERATION	4.0 SURVEILLANCE REQUIREMENTS
d. During reactor isolation conditions the reactor pressure vessel shall be depressurized to < 200 psig at normal cooldown rates if the suppression pool temperature exceeds 120°F.	d. Whenever there is indication of relief valve operation with a suppression pool temperature ≥ 160°F and the primary coolant system pressure > 200 psig, an extended visual examination of the suppression chamber shall be conducted before resuming power operation.
e. The suppression chamber water volume shall be ≥ 68,000 and ≤ 72,910 cubic feet.	e. The suppression chamber water volume shall be checked once per day.
f. Two channels of torus water level instrumentation shall be operable. From and after the date that one channel is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding 30 days unless such channel is sooner made operable. If both channels are made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding six hours unless at least one channel is sooner made operable.	f. The suppression chamber water volume indicators shall be calibrated semi-annually.
2. Primary Containment Integrity	2. Primary Containment Integrity
Primary containment integrity, as defined in Section 1, shall be maintained at all times when the reactor is critical or when the reactor water temperature is above 212°F and fuel is in the reactor vessel except while performing low power physics tests at atmospheric pressure during or after refueling at power levels not to exceed 5 Mw(t).	a. Integrated Primary Containment Leak Test (IPCLT) The containment leakage rates shall be demonstrated at the following test schedule and shall be determined in conformance with the criteria specified in Appendix J of 10 CFR 50 using the methods and provisions of ANSI N45.4-1972: 1. Three Type A Overall Integrated Containment Leakage Rate tests shall be conducted at 40 ± 10 month intervals* during shutdown at $\bar{P}_a$ (41 psig) during each 10-year service period. The third test of each set shall be conducted during the shutdown for the 10-year plant inservice inspection. *The second test of the second 10-year service period may be conducted during the 1989 refueling outage.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 52 TO FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY
MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263

1.0 INTRODUCTION

By letter dated April 9, 1987, Northern States Power Company (NSP), the licensee for Monticello Nuclear Generating Plant, requested changes to the plant Technical Specifications (TS) concerning approximately a 3-month extension in the maximum specified interval between the containment integrated leak rate (CILR) tests of 50 months. A second proposed change dealing with snubber surveillance frequency requested by the April 9, 1987 application was deleted by the licensee's June 15, 1987 submittal. Therefore, this evaluation addresses only the proposed change to extend the interval of the containment integrated leak rate tests by 3 months. Currently, the TS requires a test interval of 40 ± 10 months for overall integrated containment leakage rate tests. It is also required that the last test of the 10-year service period coincide with the refueling outage during which the 10-year ASME Code Section XI inservice inspection is conducted. The licensee states that he is unable to comply with both of these requirements during the remainder of the current 10-year service period without performing an unnecessary additional CILR test.

2.0 DISCUSSION AND EVALUATION

At Monticello, refueling outages are currently scheduled in October 1987 and May 1989, followed by the 10-year inspection outage in June 1992. The first 10-year inspection was performed during the January 1985 outage. The licensee proposed that the second and third CILR tests be conducted in the 1989 and 1992 refueling outages. This would make the interval between the last test (January 1985) and the 1989 test (which will occur in the April 1989 outage) approximately 51 months. This exceeds the maximum allowable interval by approximately 1 month between the first and second scheduled CILR tests, in the second 10-year inspection outage. Since the above test can be performed only during refueling outages, a TS change on a one-time basis is requested to permit a short extension between the first and second 10-year inspection test intervals.

The current TS is based on the requirements of Section III.D.1.(a) of Appendix J to 10 CFR Part 50. The regulation states that a set of three CILR tests shall be performed at approximately equal intervals during each 10-year service period. The third test of each set shall be conducted when the plant is shutdown for the 10-year plant inservice inspection. Since CILR tests are performed only during refueling outages, and a refueling outage is unlikely to occur exactly

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at intervals of 40 months, the regulation was made flexible with the word "approximately." The TS interval of 40 ± 10 months reflects this. Therefore, the staff concludes that the short extension of a 1-month interval to allow testing during the 1989 outage is reasonable and satisfies the requirements of 10 CFR Part 50, Appendix J. However, the licensee's request for approximately a 3-month extension is excessive and not necessary based on the 1989 refueling outage schedule. On this basis we find an extension of approximately 1 month to perform the second CILR test acceptable.

Based on the above, the staff concludes that the proposed changes to the plant TS, on a one-time basis, concerning a 1-month extension in the maximum specified interval in the TS, between the first and second containment integrated leak rate tests, in the second 10-year inspection refueling outage, complies with the intent of 10 CFR Part 50, Appendix J, and is, therefore, acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or changes a surveillance requirement. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Raj Armand

Dated: November 18, 1987