

December 23, 1987

Docket No. 50-263

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Mr. D. M. Musolf, Manager  
Nuclear Support Services  
Northern States Power Company  
414 Nicollet Mall  
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

On November 25, 1987, the Commission issued Amendment No. 54 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The amendment revised the Technical Specifications to reflect the changes supported by analysis for the reload justifying Cycle 13 operation.

Page 230 of the Technical Specifications, section 5.0 describing design features issued with the amendment contained an error of omission of words regarding control rod material, as follows: "The control rod material shall be boron carbide powder ... or hafnium." The words "or hafnium" were inadvertently omitted in the Technical Specification page submitted with your application. This fact was called to our attention by your staff by telephone call on December 10, 1987.

Technical Specification page 230, as corrected, is enclosed.

Sincerely,




Dominic C. Dilanni, Project Manager  
Project Directorate III-3  
Division of Reactor Projects


Enclosure:  
TS page 230

cc: See next page

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Date: 12/14/87



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Mr. D. M. Musolf  
Northern States Power Company

Monticello Nuclear Generating Plant

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## 5.0 DESIGN FEATURES

### 5.1 Site

- A. The reactor center line is located at approximately 850,810 feet North and 2,038,920 feet East as determined on the Minnesota State Grid, South Zone. The nearest site boundary is approximately 1630 feet S 30° W of the reactor center line and the exclusion area is defined by the minimum fenced area shown in FSAR Figure 2.2.2a. Due to the prevailing wind pattern, the direction of maximum integrated dosage is SSE. The southern property line generally follows the northern boundary of the right-of-way for the Burlington Northern Railway. More details on the current property lines can be found in USAR Figure 2.2-1.

### 5.2 Reactor

- A. The reactor core shall consist of not more than 484 fuel assemblies.
- B. The reactor core shall contain 121 cruciform-shaped control rods whose design has been reviewed and approved for BWR use by an NRC Safety Evaluation Report. The control rod material shall be boron carbide powder ( $B_4C$ ) compacted to approximately 70% of theoretical density or hafnium.

### 5.3 Reactor Vessel

- A. The pressure vessel shall be designed for a pressure of 1250 psig and a temperature of 562°F. The coolant recirculation system shall be designed for a pressure of 1148 psig on suction side of pump and 1248 psig at pump discharge. The applicable design codes shall be as described in Sections 4.2.3 and 4.3.1 of the Monticello Final Safety Analysis Report.

### 5.4 Containment

- A. The primary containment shall be of the pressure suppression type having a drywell and an absorption chamber constructed of steel. The drywell shall have a volume of approximately 134,200 ft<sup>3</sup> and is designed to conform to ASME Boiler and Pressure Vessel Code Section III Class B for an internal pressure of 56 psig at 281°F and an external pressure of 2 psig at 281°F. The absorption chamber shall have a total volume of approximately 176,250 ft<sup>3</sup>.

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