

June 5, 1990

Docket No. 50-263

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Mr. T. M. Parker, Manager  
Nuclear Support Services  
Northern States Power Company  
414 Nicollet Mall  
Minneapolis, Minnesota 55401

Dear Mr. Parker:

SUBJECT: AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. DPR-22:  
(TAC NO. 76407)

The Commission has issued the enclosed Amendment No. 74 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. This amendment consists of changes to the Technical Specifications (TS) in response to your application dated March 28, 1990.

The amendment deletes the requirement for the Reactor Building Wide Range Gas Monitors to provide a secondary containment isolation initiation input signal.

A copy of our related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

William O. Long, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 74 to License No. DPR-22
2. Safety Evaluation

cc w/enclosures:  
See next page

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\*See previous concurrence

LA/PD31:DRSP  
\*MRShuttleworth  
06/5/90 *MWR*

PM/PD31:DRSP  
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LCunningham  
05/11/90

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OGC  
LDewey  
05/16/90

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Docket No. 50-263

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Sincerely,

William O. Long, Project Manager Project  
Directorate III-1 Division of Reactor  
Projects - III, IV, V  
& Special Projects

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*MA*  
LA/PD31:DRSP  
PShuttleworth  
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*[Signature]*  
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*QCD*  
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*LD*  
OGC  
L. Dewey  
05/16/90

*X*



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

June 5, 1990

Docket No. 50-263

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Northern States Power Company  
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Sincerely,

*William O. Long*

William O. Long, Project Manager  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

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License No. DPR-22
2. Safety Evaluation

cc w/enclosures:

See next page

Mr. T. M. Parker, Manager  
Northern States Power Company

Monticello Nuclear Generating Plant

cc:

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Monticello Nuclear Generating Plant  
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Commissioners  
10 NW Second Street  
Buffalo, Minnesota 55313



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY  
DOCKET NO. 50-263  
MONTICELLO NUCLEAR GENERATING PLANT  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 74  
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northern States Power Company (the licensee) dated March 28, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

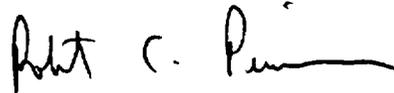
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Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 74, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert C. Pierson, Acting Director  
Project Directorate III-1  
Division of Reactor Projects - III,  
IV, V & Special Projects  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: June 5, 1990

ATTACHMENT TO LICENSE AMENDMENT NO. 74

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE

59  
62  
68  
198-1  
198y

INSERT

59  
62  
68  
198-1  
198y

Table 3.2.4  
Instrumentation That Initiates Reactor Building Ventilation Isolation  
And Standby Gas Treatment System Initiation

Function	Trip Settings	Total No. of Instrument Channels Per Trip System	Min. No. of Operable or Operating Instrument Channels Per Trip System (Notes 1, 2)	Required Conditions*
1. Low Reactor Water Level (Note 3)	$\geq 10'6"$ above the top of the active fuel	2	2	A. or B.
2. High Drywell Pressure (Note 3)	$\leq 2$ psig	2	2	A. or B.
3. Reactor Building Plenum Radiation Monitors	$\leq 100$ mR/hr	1	1 (Note 4)	A. or B.
4. Refueling Floor Radiation Monitors	$\leq 100$ mR/hr	1	1 (Note 4)	A. or B.

Notes:

- (1) There shall be two operable or tripped trip systems for each function with two instrument channels per trip system and there shall be one operable or tripped trip system for each function with one instrument channel per trip system.
  - (2) Upon discovery that minimum requirements for the number of operable or operating trip systems or instrument channels are not satisfied action shall be initiated to:
    - (a) Satisfy the requirements by placing appropriate channels or systems in the tripped condition, or
    - (b) Place the plant under the specified required conditions using normal operating procedures.
  - (3) Need not be operable when primary containment integrity is not required-
  - (4) One of the two monitors may be bypassed for maintenance and/or testing.
- \* Required Conditions when minimum conditions for operation are not satisfied.
- A. The reactor building ventilation system isolated and the standby gas treatment system operating.
  - B. Establish conditions where secondary containment is not required.

Table 4.2.1 - Continued  
 Minimum Test and Calibration Frequency For Core Cooling  
 Rod Block and Isolation Instrumentation

Instrument Channel	Test (3)	Calibration (3)	Sensor Check (3)
3. Steam Line Low Pressure	Once/month	Once/3 months	None
4. Steam Line High Radiation	Once/week (Note 5)	Note 6	Once/shift
<u>CONTAINMENT ISOLATION (GROUPS 2 &amp; 3)</u>			
1. Reactor Low Water Level (Note 10)	-	-	-
2. Drywell High Pressure (Note 10)	-	-	-
<u>HPCI (GROUP 4) ISOLATION</u>			
1. Steam Line High Flow	Once/month	Once/3 months	None
2. Steam Line High Temperature	Once/month	Once/3 months	None
<u>RCIC (GROUP 5) ISOLATION</u>			
1. Steam Line High Flow	Once/month	Once/3 months	None
2. Steam Line High Temperature	Once/month	Once/3 months	None
<u>REACTOR BUILDING VENTILATION</u>			
1. Radiation Monitors (Plenum)	Once/month	Once/3 months	Once/day
2. Radiation Monitors (Refueling Floor)	Once/month	Once/3 months	Note 4
<u>RECIRCULATION PUMP TRIP AND ALTERNATE ROD INJECTION</u>			
1. Reactor High Pressure	Once/month (Note 5)	Once/Operating Cycle- Transmitter Once/3 Months-Trip Unit	Once/Day
2. Reactor Low Low Water Level	Once/month (Note 5)	Once/Operating Cycle- Transmitter Once/3 Months-Trip Unit	Once/shift
<u>SHUTDOWN COOLING SUPPLY ISOLATION</u>			
1. Reactor Pressure Interlock	Once/month	Once/3 Months	None

Bases Continued:

- 3.2 The RBM bypass time delay is set low enough to assure minimum rod movement while upscale trips are bypassed.

The IRM rod block function provides local as well as gross core protection. The scaling arrangement is such that trip setting is less than a factor of 10 above the indicated level. Analysis of the worst case accident results in rod block action before MCPR approaches the Safety Limit (T.S.2.1.A).

A downscale indication of an APRM or IRM is an indication the instrument has failed or the instrument is not sensitive enough. In either case the instrument will not respond to changes in control rod motion and thus control rod motion is prevented. The downscale rod blocks assure that there will be proper overlap between the neutron monitoring systems and thus, that adequate coverage is provided for all ranges of reactor operation. The downscale trips are set at 3/125 of full scale.

For effective emergency core cooling for the small pipe break the HPCI or Automatic Pressure Relief system must function since for these breaks, reactor pressure does not decrease rapidly enough to allow either core spray or LPCI to operate in time. The arrangement of the tripping contacts is such as to provide this function when necessary and minimize spurious operation. The trip settings given in the specification are adequate to assure the above criteria is met. Reference Section 6.2.4 and 6.2.6 FSAR. The specification preserves the effectiveness of the system during periods of maintenance, testing, or calibration, and also minimizes the risk of inadvertent operation; i.e., only one instrument channel out of service.

Four radiation monitors (two reactor building vent plenum and two refueling floor) are provided which initiate isolation of the reactor building and operation of the standby gas treatment system following a refueling accident. The monitors measure radioactivity in the reactor building ventilation exhaust and on the refueling floor. One upscale trip signal or two downscale/inoperable trip signals, from a pair of monitors performing the same function, will cause the desired action. Trip settings of 100 mR/hr for the reactor building vent plenum monitors and the refueling floor monitors are based upon initiating normal ventilation isolation and standby gas treatment system operation so that none of the activity released during the refueling accident leaves the reactor building via the normal ventilation stack but that all the activity is processed by the standby gas treatment system.

The recirculation pump trip description and performance analysis is discussed in Topical Report NEDO-25016, September 1976, "Evaluation of Anticipated Transients Without Scram for the Monticello Nuclear Generating Plant". (See September 15, 1976 letter from Mr L O Mayer, NSP, to Mr D L Ziemann, USNRC.) The pump trip is provided to minimize reactor pressure in the highly unlikely event of a plant transient coincident with the failure of all control rods to scram. The rapid flow reduction

TABLE 3.8.2 - RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION  
(Page 2 of 2)

Instrument	Minimum Channels Operable	Applicability	Action if Minimum Channels not Operable
Reactor Building Vent (includes Turbine Building & Radwaste Building releases)			
Wide Range Noble Gas Activity Monitors**	1	At all times	Releases via this pathway may continue for up to 30 days provided grab samples are taken and analyzed at least once every 8 hours.
Iodine Sampler Cartridge	1	At all times	Releases via the pathway may continue for up to 30 days provided within 8 hours samples are continuously collected with auxiliary sampling equipment as required by Table 4.8.4.
Particulate Sampler Cartridge	1	At all times	Releases via the pathway may continue for up to 30 days provided within 8 hours samples are continuously collected with auxiliary sampling equipment as required by Table 4.8.4.
Duct Flow Monitors	1	At all times	Releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once every 4 hours.
Sample Flow Instruments	1	At all times	Releases via this pathway may continue for up to 30 days provided the flow rate is estimated at least once every 4 hours.

Notes:

# - Indicates number of channels required per operating recombiner train.

\* - Provides automatic termination of offgas treatment system releases.

\*\* - Provides control room indication prior to exceeding 10CFR Part 20 release limits.

### 3.8 and 4.8 Bases: (continued)

Specification 3.8.B.4.c is provided to ensure that the concentration of potentially explosive gas mixtures contained in the compressed storage subsystem is maintained below the flammability limits of hydrogen and oxygen. Automatic control features are included in the system to prevent the hydrogen and oxygen concentrations from reaching these flammability limits. Maintaining the concentration of hydrogen below the flammability limit provides assurance that the releases of radioactive materials will be controlled in conformance with the requirements of General Design Criterion 60 of Appendix A to 10 CFR Part 50.

Specification 3.8.B.4.e is provided to limit the radioactivity which can be stored in one decay tank. Restricting the quantity of radioactivity contained in each gas storage tank provides assurance that in the event of an uncontrolled release of the tanks contents, the resulting total body exposure to an individual at the site restricted area boundary will not exceed 20 mrem. A flow restrictor in the discharge line of the decay tanks prevents a tank from being discharged at an uncontrolled rate. In addition, interlocks prevent the contents of a tank from being released with less than 12 hours of holdup.

Specification 3.8.B.5 establishes a maximum activity at the steam jet air ejector. Restricting the gross radioactivity rate of noble gases from the main condenser provides reasonable assurance that the total body exposure to an individual at the restricted area boundary will not exceed the limits of 10 CFR Part 20 in the event this effluent is inadvertently discharged directly to the environment with minimal treatment. This specification implements the requirements of General Design Criteria 60 and 64 of Appendix A to 10 CFR Part 50.

Specification 3.8.B.6 requires the containment to be purged and vented through the standby gas treatment system except during inerting and deinerting operations. This provides for iodine and particulate removal from the containment atmosphere. During outages when the containment is opened for maintenance, the containment ventilation exhaust is directed to the monitored reactor building vent. Use of the 2-inch flow path prevents damage to the standby gas treatment system in the event of a loss of coolant accident during purging or venting. Use of the reactor building plenum and vent flow path for inerting and deinerting operations permits the control room operators to monitor the activity level of the resulting effluent by use of the Reactor Building Vent Wide Range Gas Monitors. In the event that the reactor building release rate exceeds the Reactor Building Vent Wide Range Gas Monitor alarm settings, the monitors will alarm in the control room alerting the operators to take actions to limit the release of gaseous radioactive effluents. The alarm settings for the reactor building vent wide range gas monitors are calculated in accordance with the NRC approved methods in the ODCM to ensure that alarms will alert control room operators prior to the limits of specification 3.8.B.1 being exceeded.

### C. Solid Radioactive Waste

Specification 3.8.C.1 provides assurance that the solid radwaste system will be used whenever solid radwastes require processing and packaging prior to being shipped offsite. This specification implements the requirements of 10 CFR Part 50.36a and General Design Criteria 60 of Appendix A to 10 CFR Part 50.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 74 TO FACILITY OPERATING LICENSE NO. DPR-22  
NORTHERN STATES POWER COMPANY  
MONTICELLO NUCLEAR GENERATING PLANT  
DOCKET NO. 50-263

1.0 INTRODUCTION

By letter dated March 28, 1990, the Northern States Power Company (the licensee) requested an amendment to the Technical Specifications (TS) appended to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The proposed amendment would delete a current operability requirement for the Reactor Building Wide Range Gas Monitors to provide a secondary containment isolation signal. A secondary containment isolation is an Engineered Safety Feature (ESF) actuation which results in reactor building isolation and initiation of the standby gas treatment system (SGTS).

A discussion of the proposed change, and the NRC staff's evaluation and findings are provided below.

2.0 DISCUSSION AND EVALUATION

Following the Three Mile Island accident, two Wide Range Gas Monitors (WRGMs) were installed in the Reactor Building Ventilation Exhaust System (Refs: NUREG-0578 "TMI Lessons Learned," Section 2.1.8.b and NUREG-0737, Item II.F.1.1). The purpose of the instruments was to provide expanded range capability (twelve decades) for continuous monitoring of radioactive gaseous effluents under both normal and post-accident operating conditions. (Note: Similar instruments were provided in the plant off-gas stack.)

Prior to installation of the WRGM's, the function of reactor building vent exhaust gaseous radioactive effluent monitoring was performed by two narrow range monitors mounted in the Reactor Building exhaust plenum (Reactor Building Plenum Monitors). These plenum monitors were installed with the capability to initiate automatic reactor building ventilation isolation and standby gas treatment system initiation, to limit the off-site release of radioactive gases to Part 20 limits. At that time, the plenum monitor set point was 3 mrem/hr. Concurrent with the installation of the new WRGM's during the 1982 outage, Amendment No. 15 was issued providing new Radiological Effluent Technical Specifications. The new WRGMs were specified by TS 3/4.8.B.1 to have a set point to be calculated in accordance with the Offsite Dose Calculation Manual (ODCM)

and to provide automatic termination of gaseous release at a set point that limits the off-site dose rate to  $\leq 500$  mrem/year at all times. Subsequently, confusion arose as to which instruments provided what function for which purpose. This resulted in a new amendment application.

Amendment No. 40 was issued March 18, 1986. This amendment raised the plenum monitor set points to 100 mR/hr and clarified that they (and not the WRGMs) serve to provide protection required by 10 CFR Part 100 (the ESF signal for mitigation of Design Basis Accidents). The WRGMs were stated to serve the purpose of providing protection required by 10 CFR Part 20. However, the licensee, in designing the WRGM system instrumentation logic, provided the WRGMs with the capability to initiate the ESF function of secondary containment isolation; a capability already provided by the plenum monitors at a higher set point. The intent served two purposes: (1) the plenum monitors could possibly be eliminated in the future, and (2) it anticipated proposed requirement of NUREG-0133 "Preparation of Radiological Effluent Technical Specifications for Nuclear Power Plants" which specifies capability for automatic termination of (non-accident related) gaseous releases in accordance with "Table 3.3-12." This Table was not included in NUREG-0133, but as a later, separate document draft NUREG-0473 "Standard Radiological Effluent Technical Specifications for Boiling Water Reactors." The final draft NUREG-0473 and all subsequently issued power reactor Technical Specifications did not require capability for automatic isolation of Reactor Building vent exhaust gaseous effluent releases upon reaching Part 20 limits, only off-gas stack releases. This has remained the staff position.

The current configuration at Monticello is thus that both the WRGM's and plenum monitors have the capability (and operability requirements), to monitor RB ventilation exhaust activity and initiate reactor building isolation with SGTS initiation. This configuration is believed to be unique to Monticello. It is now the licensee's desire to eliminate the unique operability requirement that the WRGMs provide in the capability of initiating the secondary containment isolation ESF function. This would bring the Monticello design into consistency with other similar facilities and eliminate the potential for reportable ESF actuations due to spurious WRGM trip signals.

The staff has evaluated the amendment application with the objective of determining if the amendment would adversely affect the capability of the facility to meet (1) design basis accident (DBA) dose limits and (2) the gaseous effluent limits of Section 3.8.B.1 of the facility Technical Specifications.

The proposed amendment would not modify the instrumentation associated with the plenum monitors. The plenum monitors would retain their capability to isolate secondary containment for the purpose of mitigating a DBA. The plenum monitor set point for this action is 100 mR/hr which assures that 10 CFR 100 dose limits would not be exceeded.

TS 3.8.B.1 specifies dose rate limits due to release of gaseous effluents. These limits are based on 10 CFR Part 20 and provide the basis for instrumentation requirements to monitor and isolate gaseous effluents. For noble gases the dose rate limit is  $\leq 500$  mrem/year to the

whole body and  $\leq 3000$  mrem/year to the skin, applicable at all times. To evaluate the capability for compliance with these limits, the proposed amendment was compared to the guidance of draft NUREG-0473. NUREG-0473 does not address the need for BWR facilities to have provisions for automatic termination of reactor building ventilation releases for purposes of compliance with 10 CFR Part 20 or 10 CFR Part 50, Appendix I. This feature is required for the stack off-gas release path, a requirement with which Monticello complies, and which would not be affected by this amendment. For reactor building vent exhaust gaseous releases, the WRGMs will provide monitoring alarm capability to enable operators to take immediate action to terminate a release which could result in exceeding the limits of TS 3.8.B.1.

Based on our findings that (1) the capability to mitigate a design basis accident would not be adversely affected, and (2) the capability to limit off-site gaseous effluent release dose rates to the limits of TS 3.8.B.1 would likewise not be adversely affected, we conclude the licensee's proposed Technical Specifications amendment is acceptable.

### 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 or change an inspection or surveillance requirement. We have determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released off-site, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously published a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. Long

Concurrence Review: J. Lee

Date: June 5, 1990