



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 20, 2002

OFFICE OF THE
GENERAL COUNSEL

Mary Olson
Southeast Director of NIRS
P.O. Box 7586
Asheville, NC 20882

Dear Ms. Olson:

Pursuant to the Staff's agreement to forward Daily Event Reports (DERs) relating to McGuire Nuclear Station, Units 1 & 2, and Catawba Nuclear Station, Units 1 & 2, enclosed please find three DERs.

Sincerely,

A handwritten signature in black ink, appearing to read "S. Uttal", is written over the typed name.

Susan L. Uttal
Counsel for NRC Staff

cc w/encls: J. Zeller
D. Repka

cc w/o encls: Service list

POWER REACTOR

EVENT NUMBER: 38557

| | | | | | | |
|--------------------------------------|------------|-----------|----------|-------------------------------|----------|---------------|
| FACILITY: CATAWBA | | REGION: 2 | | NOTIFICATION DATE: 12/11/01 | | |
| UNIT: [] [2] [] | | STATE: SC | | NOTIFICATION TIME: 17:16 [ET] | | |
| RX TYPE: [1] W-4-LP, [2] W-4-LP | | | | EVENT DATE: 10/17/01 | | |
| NRC NOTIFIED BY: ED BREWER | | | | EVENT TIME: 10:39 [EST] | | |
| HQ OPS OFFICER: LEIGH TROCINE | | | | LAST UPDATE DATE: 12/11/01 | | |
| EMERGENCY CLASS: NON EMERGENCY | | | | NOTIFICATIONS | | |
| 10 CFR SECTION: | | | | LEONARD WERT R2 | | |
| AINV INVALID SPECIF SYSTEM ACTUATION | | | | | | |
| UNIT | SCRAM CODE | RX CRIT | INIT PWR | INIT RX MODE | CURR PWR | CURR RX MODE |
| 2 | A | N | 0 | Cold Shutdown | 0 | Cold Shutdown |

EVENT TEXT

INVALID ACTUATION OF THE REACTOR PROTECTION SYSTEM WHILE THE UNIT WAS IN COLD SHUTDOWN (60-Day Report)

The following text is a portion of a facsimile received from the licensee:

"This 60-day optional report, as allowed by 10CFR50.73(a)(1), is being made under the reporting requirement in 10CFR50.73(a)(2)(iv)(A) to describe an invalid actuation of a specified system, specifically the reactor protection system (RPS). On October 17, 2001, at 1039 with Unit 2 in MODE 5 with a boron concentration of 2531 ppm, an invalid reactor trip signal was generated from 2 out of 4 logic for overtemperature delta-T. The reactor trip breakers opened as designed upon receipt of the invalid reactor trip signal. All control rods and shutdown rods were already fully inserted before the invalid reactor trip signal trip signal was generated. At the time of the invalid reactor trip signal, reactor protection channel III had been properly removed from service for maintenance testing. Other maintenance technicians were in the process of setting up test equipment in reactor protection channel I for upcoming resistance temperature detector cross calibrations. The technicians thought that the place where they were connecting the test equipment was isolated from the circuit and would have no effect on the plant. During the connections, a momentary spike occurred on reactor protection channel I. This completed the 2 out of 4 logic for an overtemperature delta-T reactor trip, and a reactor trip occurred. The control room operators entered appropriate procedures and verified proper equipment operation for the existing plant conditions. Both trains A [and] B of the reactor protection system actuated. This was a complete actuation, and the system performed as designed for the existing plant conditions. This event has been entered into the site-specific corrective

(Continued on next page)

action program for resolution."

The licensee notified the NRC resident inspector and plans to notify applicable state, county, and local agencies.

POWER REACTOR

EVENT NUMBER: 38630

FACILITY: MCGUIRE
UNIT: [] [2] []
RX TYPE: [1] W-4-LP, [2] W-4-LP

REGION: 2
STATE: NC

NOTIFICATION DATE: 01/15/02
NOTIFICATION TIME: 10:14 [ET]
EVENT DATE: 01/15/02
EVENT TIME: 09:55 [EST]
LAST UPDATE DATE: 01/15/02

NRC NOTIFIED BY: WAYNE HOYLE
HQ OPS OFFICER: BOB STRANSKY

NOTIFICATIONS

EMERGENCY CLASS: UNUSUAL EVENT
10 CFR SECTION:
AAEC EMERGENCY DECLARED

BRIAN BONSER R2
ROBERT DENNIG NRR
TONY CEGIELSKI FEMA

| UNIT | SCRAM CODE | RX CRIT | INIT PWR | INIT RX MODE | CURR PWR | CURR RX MODE |
|------|------------|---------|----------|-----------------|----------|-----------------|
| 2 | N | Y | 100 | Power Operation | 100 | Power Operation |

EVENT TEXT

UNUSUAL EVENT DECLARED AND EXITED

The licensee entered and exited an Unusual Event due to a leak from the Chemical and Volume Control System (CVCS) that was caused by the lifting of a relief valve. During valve maintenance on a line tagged out as isolated, the opening of a valve caused pressure in the line to exceed that of a relief valve, causing flow to the Unit 1 Boric Acid Storage Tank (BAST). The licensee is sampling the tank to ensure operability of the tank. The NRC resident inspector has been informed of this event by the licensee.

POWER REACTOR

EVENT NUMBER: 38748

FACILITY: MCGUIRE
UNIT: [1] [] []
RX TYPE: [1] W-4-LP, [2] W-4-LP

REGION: 2
STATE: NC

NOTIFICATION DATE: 03/04/02
NOTIFICATION TIME: 10:13 [ET]
EVENT DATE: 03/04/02
EVENT TIME: 08:41 [EST]
LAST UPDATE DATE: 03/04/02

NRC NOTIFIED BY: PHILLIP THOMPSON
HQ OPS OFFICER: MIKE NORRIS

NOTIFICATIONS

EMERGENCY CLASS: NON EMERGENCY
10 CFR SECTION:
ARPS RPS ACTUATION - CRITICAL
AESF VALID SPECIF SYS ACTUATION

CAUDLE JULIAN R2

| UNIT | SCRAM CODE | RX CRIT | INIT PWR | INIT RX MODE | CURR PWR | CURR RX MODE |
|------|------------|---------|----------|-----------------|----------|--------------|
| 1 | M/R | Y | 100 | Power Operation | 0 | Hot Standby |

EVENT TEXT

MANUAL REACTOR TRIP DUE TO LOSS OF FEEDWATER

"Manual reactor trip due to falling level in 1A Steam Generator (S/G), (RPS Activation). Level decrease occurred after the Feedwater Regulating Valve (FRV) for the 1A S/G failed closed upon loss of the normal and backup electrical power to the valve solenoid. After the reactor trip, the 1A S/G experienced a LO-LO Level Alarm which auto started both motor driven Auxiliary Feedwater pumps, (ESF Actuation). All S/G levels are now at normal levels and Unit 1 is stable. An investigation is in progress to determine the loss of electrical power to the FRV solenoid for the 1A S/G."

The plant is dumping steam to the Main Condenser with no known Primary/Secondary leakage. There was no actuation of primary safety valves. All safety systems are operable and normal electrical distribution is available.

The NRC Resident Inspector has been notified.