July 14, 1989

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Docket No. 50-263

Mr. T. M. Parker
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall
Minneapolis, Minnesota 55401

Dear Mr. Parker:

SUBJECT: ERRATA FOR AMENDMENT NO. 63 TO FACILITY OPERATING LICENSE DPR-22 - (TAC NO. ~~61161~~) 61661

The changes to the plant Technical Specifications (TS) implemented by License Amendment No. 63, which were transmitted to you by letter dated April 18, 1989, contained an incorrect TS page. Specifically, page 17 which was included in the amendment application became superseded by a later application that was issued prior to Amendment No. 63. Page 16 is the correct TS page for Amendment NO. 63. Page 16 was amended at the same time as the deletion of page 17 (Amendment NO. 60) and is amended again in Amendment No. 63.

The enclosed errata to the TS changes implemented by License Amendment No. 63 includes the corrected pages.

Sincerely,

John Stefano, Project Manager
Project Directorate III-1
Division of Reactor Projects - III, IV, V
& Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

LA:PD31;DRSP	PM:PD31:DRSP	(A)D:PD31:DRSP
PShuttleworth	JStefano	LYandell
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Division of Reactor Projects - III, IV, V
& Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
As stated

cc w/enclosure:
See next page

Mr. D. M. Musolf
Northern States Power Company

Monticello Nuclear Generating Plant

cc:
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Wright County Board of
Commissioners
10 NW Second Street
Buffalo, Minnesota 55313

ERRATA *

ATTACHMENT TO LICENSE AMENDMENT NO. 63

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by amendment number and contain marginal lines indicating the area of change.

REMOVE	INSERT
v	v
1	1
16*	16 *
28	28
29	29
30	30
32	32
33	33
41	41
42	42
43	-
44	-
60	60
61	61
62	62
63	63
63a	63a
72	72
73	-
74	-
75	-
82	82
-	82a
108	108
109	109
110	110
111	111
156	156
169	169
208	208
229a	229a
229b	229b
229c	229c
229d	229d

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Bases Continued:

that indicated by the neutron flux at the scram setting. Analyses demonstrate that, with a 120% scram trip setting, none of the abnormal operational transients analyzed violate the fuel Safety Limit and there is a substantial margin from fuel damage. Therefore, the use of flow referenced scram trip provides even additional margin.

For operation in the startup mode while the reactor is at low pressure, the IRM scram setting of 20% of rated power provides adequate thermal margin between the setpoint and the safety limit, 25% of rated. The margin is adequate to accommodate anticipated maneuvers associated with power plant startup. Effects of increasing pressure at zero or low void content are minor, cold water from sources available during startup is not much colder than that already in the system, temperature coefficients are small, and control rod patterns are constrained to be uniform by operating procedures. Worth of individual rods is very low in a uniform rod pattern. Thus, of all possible sources of reactivity input, uniform control rod withdrawal is the most probable cause of significant power rise. Because the flux distribution associated with uniform rod withdrawals does not involve high local peaks, and because several rods must be moved to change power by a significant percentage of rated power, the rate of power rise is very slow. Generally, the heat flux is in near equilibrium with the fission rate. In an assumed uniform rod withdrawal approach to the scram level, the rate of power rise is no more than 5% of rated power per minute, and the IRM system would be more than adequate to assure a scram before the power could exceed the safety limit. The IRM scram remains active until the mode switch is placed in the run position and the associated APRM is not downscale. This switch occurs when reactor pressure is greater than 850 psig.

The operator will set the APRM neutron flux trip setting no greater than that stated in Specification 2.3.A.1. However, the actual setpoint can be as much as 3% greater than that stated in Specification 2.3.A.1 for recirculation driving flows less than 50% fo design and 2% greater than that shown for recirculation driving flows greater than 50% of design due to the deviations discussed on page 39.

B. Deleted