December 23, 1985

Docket No. 50-263

Mr. D. M. Musolf
Nuclear Support Services Department
Northern States Power Company
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

The Commission has issued the enclosed Amendment No. 36 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The amendment consists of changes to the Technical Specifications in response to your application dated September 24, 1982, as supplemented on December 3, 1985.

The amendment revises the Technical Specifications to allow cycling of the drywell-to-wetwell vacuum breakers during inerting and deinerting to aid in changing the gas mixture in the drywell-to-torus vent pipes and vent header.

A copy of the Safety Evaluation is enclosed.

Sincerely,

ORIGINAL SIGNED BY

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Enclosures:
1. Amendment No. 36 to License No. DPR-22
2. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION	FEltawila		
Docket File	CJamerson	BGrimes	OPA, CMiles
NRC PDR	RAuluck	TBarnhart (4)	RDiggs
Local PDR	OELD	WJones	Gray File
BWD#1 Reading	LJHarmon	DVassallo	
RBernero	ELJordan	ACRS (10)	JPartlow

DBL:PD#2	DBL:PD#1	DBL:PD#1	OELD
SNorris:rc*	RAuluck*	JZwolinski	*
11/05/85	11/06/85	12/21/85	11/21/85

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Northern States Power Company
414 Nicollet Mall - 8th Floor
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Sincerely,

Rajender Auluck, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:
1. Amendment No. to License No. DPR-22
2. Safety Evaluation

cc w/enclosures:
See next page

DISTRIBUTION	FEltawila		
Docket File	SNorris	BGrimes	OPA, CMiles
NRC PDR	RAuluck	TBarnhart (4)	RDiggs
Local PDR	OELD	WJones	Gray File
ORB#2 Reading	LJHarmon	MVirgilio	Extra - 5
HThompson	ELJordan	ACRS (10)	JPartlow

DL:ORB#2	DL:ORB#2	DL:ORB#2	OELD	DL:AD-OR
SNorris:rc	RAuluck	DVassallo		GLainas
11/5/85	11/6/85	11/14/85	11/21/85	11/ /85

cc:
Gerald Charnoff, Esquire
Shaw, Pittman, Potts and
Trowbridge
1800 M Street, N. W.
Washington, D. C. 20036

U. S. Nuclear Regulatory Commission
Resident Inspector's Office
Box 1200
Monticello, Minnesota 55362

Plant Manager
Monticello Nuclear Generating Plant
Northern States Power Company
Monticello, Minnesota 55362

Russell J. Hatling
Minnesota Environmental Control
Citizens Association (MECCA)
Energy Task Force
144 Melbourne Avenue, S. E.
Minneapolis, Minnesota 55113

Executive Director
Minnesota Pollution Control Agency
1935 W. County Road B2
Roseville, Minnesota 55113

John W. Ferman, Ph.D.
Nuclear Engineer
Minnesota Pollution Control Agency
1935 W. County Road B2
Roseville, Minnesota 55113

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

Commissioner of Health
Minnesota Department of Health
717 Delaware Street, S. E.
Minneapolis, Minnesota 55440

O. J. Arlien, Auditor
Wright County Board of
Commissioners
10 NW Second Street
Buffalo, Minnesota 55313

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Northern States Power Company (the licensee) dated September 24, 1982, as supplemented on December 3, 1985 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

2 Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 36 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 23, 1985.

ATTACHMENT TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

Page
164
165

3.0 LIMITING CONDITIONS FOR OPERATION	4.0 SURVEILLANCE REQUIREMENTS
4. Pressure Suppression Chamber-Drywell Vacuum Breakers	4. Pressure Suppression Chamber-Drywell Vacuum Breakers
a. When primary containment is required, all eight drywell-suppression chamber vacuum breakers shall be operable and positioned in the closed position as indicated by the position indication system, except during testing and except as specified in 3.7.A.4.b through 3.7.A.4.d below.	a. Operability and full closure of the drywell-suppression chamber vacuum breakers shall be verified by performance of the following:
b. Any drywell-suppression chamber vacuum breaker may be nonfully closed as indicated by the position indication and alarm systems provided that drywell to suppression chamber differential pressure decay does not exceed that shown on Figure 3.7.1	(1) Monthly each operable drywell-suppression chamber vacuum breaker shall be exercised through an opening-closing cycle.
c. Up to two drywell-suppression chamber vacuum breakers may be inoperable provided that: (1) the vacuum breakers are determined to be fully closed and at least one position alarm circuit is operable or (2) the vacuum breaker is secured in the closed position or replaced by a blank flange.	(2) Once each operating cycle, drywell to suppression chamber leakage shall be demonstrated to be less than that equivalent to a one-inch diameter orifice and each vacuum breaker shall be visually inspected. (Containment access required)
d. Drywell-suppression chamber vacuum breakers may be cycled, one at a time using the exercise test push-button, during containment inerting and deinerting operations to assist in purging air or nitrogen from the suppression chamber vent header.	(3) Once each operating cycle, vacuum breaker position indication and alarm systems shall be calibrated and functionally tested. (Containment access required)
	(4) Once each operating cycle, the vacuum breakers shall be tested to determine that the force required to open each valve from fully closed to fully open does not exceed that equivalent to 0.5 psi acting on the suppression chamber face of the valve disc. (Containment access required)

Amendment No. 8, 36

3.0 LIMITING CONDITIONS FOR OPERATION	4.0 SURVEILLANCE REQUIREMENTS
e. One position alarm circuit can be inoperable providing that the redundant position alarm circuit is operable. Both position alarm circuits may be inoperable for a period not to exceed seven days provided that all vacuum breakers are operable.	b. When the position of any drywell-suppression chamber vacuum breaker valve is indicated to be not fully closed at a time when such closure is required, the drywell to suppression chamber differential pressure decay shall be demonstrated to be less than that shown on Figure 3.7.1 immediately and following any evidence of subsequent operation of the inoperable valve until the inoperable valve is restored to a normal condition. c. When both position alarm circuits are made or found to be inoperable, the control panel indicator light status shall be recorded daily to detect changes in the vacuum breaker position.
5. Oxygen Concentration a. The primary containment atmosphere shall be reduced to less than 4% oxygen by volume with nitrogen gas whenever the reactor is in the run mode, except as specified in 3.7.A.5.b. b. Within the 24-hour period subsequent to placing the reactor in the run mode following shutdown, the containment atmosphere oxygen concentration shall be reduced to less that 4% by volume, and maintained in this condition. Deinerting may commence 24 hours prior to leaving the run mode for a reactor shutdown.	5. Oxygen Concentration Whenever inerting is required, the primary containment oxygen concentration shall be measured and recorded on a weekly basis.

Amendment No. 35, 36

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 36 TO FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

1.0 INTRODUCTION

By letter dated September 24, 1982 Northern States Power Company (NSP/the licensee) proposed revised Technical Specifications (TSs) to permit vacuum breaker cycling (i.e. opening, holding open for up to one minute, and closing), one vacuum breaker at a time during containment inerting and deinerting operations.

2.0 EVALUATION

The proposed change will allow cycling of the drywell-to-wetwell vacuum breakers during inerting and deinerting operation to aid in changing the gas mixture in the drywell-to-torus vent pipes and vent header. This cycling operation is quite similar to the operability test currently required every 31 days by the Standard Technical Specification.

The operation of the vacuum breakers (no more than one vacuum breaker at a time) in these modes will be accomplished from the control room with an operator present to assure closure of the vacuum breaker following the completion of the operation. Based on our evaluation of the licensee submittal, we agree with their conclusion that cycling the vacuum breakers during inerting will further assure that oxygen concentration in the vent pipe and vent header remains below the Technical Specification limits. We also find that the cycling of the vacuum breakers during deinerting operation will eliminate potential pockets of inert gas. As stated above, this operation is not much different than the monthly valve exercise test. If a vacuum breaker would stick open, the plant would be placed in cold sutdown as required by the existing TSs. Therefore, the proposed Technical Specifications are acceptable. Subsequent to the initial notice in the Federal Register, the Northern States Power Company, by letter dated December 3, 1985, corrected the typographical error. These revisions do not change the substance of the amendment and do not alter the staff's initial determination that this amendment would involve no significant hazards considerations.

3.0 ENVIRONMENTAL CONSIDERATIONS

This amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: F. Eltawila .

Dated: December 23, 1985.