

March 13, 1986

Docket No. 50-263

Mr. D. M. Musolf
Nuclear Support Services Department
Northern States Power Company
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401

Dear Mr. Musolf:

SUBJECT: VARIOUS ADMINISTRATIVE CHANGES (TAC 57632, 60612 THROUGH 60617)

Re: Monticello Nuclear Generating Plant

The Commission has issued the enclosed Amendment No. 39 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. This amendment is in response to your application dated April 26, 1985, as supplemented on October 16, 1985.

The amendment revises the Technical Specifications for the following items: (1) Snubber Table, (2) Section 6.5.6, Plant Operating Procedures, (3) Radiological Effluent Technical Specifications, (4) Rod Block Monitor Test Frequency, and (5) several miscellaneous administrative changes.

A copy of our related Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notices.

Sincerely,

Original signed by:

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Enclosures:

1. Amendment No. 39 to License No. DPR-22
2. Safety Evaluation

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in black ink, appearing to read "John A. Zwolinski".

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Enclosures:

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2. Safety Evaluation

cc w/enclosures:
See next page

Mr. D. M. Musolf
Northern States Power Company

Monticello Nuclear Generating Plant

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated April 26, 1985, as supplemented on October 16, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:

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2 Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 39 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 13, 1986.

ATTACHMENT TO LICENSE AMENDMENT NO. 39

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Revise Appendix "A" Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
vi	vi
61	61
129	129
130	130
130a	-
130b	-
131	131
132	132
132a	-
229i	229i
229n	229p
229q	229q
229r	229r
229s	229s
244	244
244a	244a
246b	246b

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Table 4.2.1
Minimum Test and Calibration Frequency For Core Cooling
Rod Block and Isolation Instrumentation

Instrument Channel	Test (3)	Calibration (3)	Sensor Check (3)
<u>ECCS INSTRUMENTATION</u>			
1. Reactor Low-Low Water Level (Note 7)	once/month	Once/3 months	Once/Shift
2. Drywell High Pressure (Note 7)	once/month	Once/3 months	None
3. Reactor Low Pressure (Pump Start)	Note 1	Once/3 months	None
4. Reactor Low Pressure (Valve Permissive)	Note 1	Once/3 months	None
5. Undervoltage Emergency Bus	Refueling Outage	Refueling Outage	None
6. Low Pressure Core Cooling Pumps			
Discharge Pressure Interlock	Note 1	Once/3 months	None
7. Loss of Auxiliary Power	Refueling Outage	Refueling Outage	None
8. Condensate Storage Tank Level	Refueling Outage	Refueling Outage	None
9. Reactor High Water Level	Once/month	Once/3 months	Once/day
<u>ROD BLOCKS</u>			
1. APRM Downscale	Notes (1,5)	Once/3 months	None
2. APRM Flow Variable	Notes (1,5)	Once/3 months	None
3. IRM Upscale	Notes (2,5)	Note 2	Note 2
4. IRM Downscale	Notes (2,5)	Note 2	Note 2
5. RBM Upscale	Once/month Note (5)	Once/3 months	None
6. RBM Downscale	Once/month Note (5)	Once/3 months	None
7. SRM Upscale	Notes (2,5)	Note 2	Note 2
8. SRM Detector not in Start-up Position	Note 2	Note 2	Note 2
9. Scram Discharge Volume-High Level	Once/3 months	Refueling outage	None
<u>MAIN STEAM LINE ISOLATION</u>			
1. Steam Tunnel High Temperature	Refueling Outage	Refueling Outage	None
2. Steam Line High Flow	Note 1	Once/3 months	Once/Shift

3.0 LIMITING CONDITIONS FOR OPERATION

H. Snubbers

1. Except as permitted below, all safety related snubbers shall be operable whenever the supported system is required to be Operable.
2. With one or more snubbers made or found to be inoperable for any reason when Operability is required, within 72 hours:
 - a. Replace or restore the inoperable snubbers to Operable status and perform an engineering evaluation or inspection of the supported components, or
 - b. Determine through engineering evaluation that the as-found condition of the snubber had no adverse effect on the supported components and that they would retain their structural integrity in the event of design basis seismic event, or
 - c. Declare the supported system inoperable and take the action required by the Technical Specifications for inoperability of that system.

4.0 SURVEILLANCE REQUIREMENTS

3. The diffuser to lower plenum differential pressure reading on an individual jet pump is 10% or more, less than the mean of all jet pump differential pressures.

H. Snubbers

The following surveillance requirements apply to all safety related snubbers.

1. Visual inspection of snubbers shall be conducted in accordance with the following schedule:

<u>No. of Snubbers Found Inoperable per Inspection Period</u>	<u>Next Required Inspection Period</u>
0	18 months ± 25%
1	12 months ± 25%
2	6 months ± 25%
3,4	124 days ± 25%
5,6,7	62 days ± 25%
8 or more	31 days ± 25%

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized in two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

3.0 LIMITING CONDITIONS FOR OPERATION

3. All safety-related snubbers installed or planned for use at Monticello are hydraulic snubbers. No mechanical snubbers are used on safety-related systems at Monticello. If installed in the future, appropriate Technical Specifications changes will be proposed within 60 days of installation.

4.0 SURVEILLANCE REQUIREMENTS

2. Visual inspections shall verify (1) that there are no visible indications of damage or impaired operability and (2) attachments to the supporting structure are secure. Snubbers which appear inoperable as a result of visual inspection may be determined Operable for the purpose of establishing the next visual inspection interval by:
 - a. Clearly establishing the cause of the rejection for that particular snubber and for others that may be generically susceptible; and
 - b. Functionally testing the affected snubber in the as-found condition and finding it Operable per Specification 4.6.H.4.

However, when the fluid plunger gauge of a hydraulic snubber is below low range, the snubber shall be considered inoperable for the purposes of establishing the next visual inspection interval.

3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

3. Functional testing of snubbers shall be conducted at least once per 18 months \pm 25% during cold shutdown. Ten percent of the total number of each brand of snubber shall be functionally tested either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria in Specification 4.6.H.4 below, an additional ten percent of that brand shall be functionally tested until no more failures are found or all snubbers of that brand have been tested.

The representative sample selected for functional testing shall include the various configurations, operating environments, and the range of size and capacity of the snubbers.

In addition to the regular sample and specified re-samples, snubbers which failed the previous functional test shall be retested during the next test period if they were reinstalled as a safety-related snubber. If a spare snubber has been installed in place of a failed safety related snubber, it shall be tested during the next period.

If any snubber selected for functional testing either fails to lockup or fails to move (i.e. frozen in place) the cause shall be evaluated and if caused by manufacturer or design deficiency, all snubbers of the same design subject to the same defect shall be functionally tested.

3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

4. Hydraulic snubber functional tests shall verify that:
 - a. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression
 - b. Snubber bleed, or release rate, where required, is within the specified range in compression or tension.
5. For any snubbers found inoperable, an engineering evaluation or inspection shall be performed on the components which are supported by the snubbers. The purpose of this engineering evaluation or inspection shall be to determine if the components supported by the snubbers were adversely affected by the inoperability of the snubbers in order to ensure that the supported component remains capable of meeting the designed service.
6. The installation and maintenance records for each safety related snubber shall be reviewed at least once every 18 months to verify that the indicated service life will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded, the snubber service life shall be re-evaluated or the snubber shall be replaced or reconditioned to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement, or reconditioning shall be indicated in the records.

3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

3. Deviations are permitted from the required sampling schedule if samples are unobtainable due to hazardous conditions, seasonable unavailability, or to malfunction of automatic sampling equipment. If the latter occurs, every effort shall be made to complete corrective action prior to the end of the next sampling period.
4. With the level of radioactivity in an environmental sampling medium exceeding the reporting levels of Table 4.16.3 when averaged over any calendar quarter, in lieu of any other report, prepare and submit to the Commission within 30 days from the end of the affected calendar quarter a Report pursuant to Specification 6.7.C.3. When more than one of the radionuclides in Table 4.16.3 are detected in the sampling medium, this report shall be submitted if:

$$\frac{\text{concentration (1)}}{\text{limit level (1)}} + \frac{\text{concentration (2)}}{\text{limit level (2)}} + \dots > 1.0$$

When radionuclides other than those in Table 4.16-3 are detected and are the result of plant effluents, this report shall be submitted if the potential annual dose to an individual is equal to or greater than the calendar year limits of Specifications 3.8.A.2, 3.8.B.2, or 3.8.B.3. This report is not required if the measured level of radioactivity was not the result of plant effluents; however, in such an event, the condition shall be reported and described in the Annual Radiation Environmental Monitoring Report.

3.16/4.16

Amendment No. 18, 37, 39

2291

Table 4.16.1
(Page 5 of 5)

MONTICELLO NUCLEAR GENERATING PLANT
RADIATION ENVIRONMENTAL MONITORING PROGRAM
SAMPLE COLLECTION AND ANALYSIS

<u>Exposure Pathway and/or Sample</u>	<u>Number of Samples and Sample Locations**</u>	<u>Sampling and Collection Frequency</u>	<u>Type and Frequency of Analysis</u>
c. Food Products	One sample of corn and potatoes from any area that is irrigated by water in which liquid radioactive effluent has been discharged.***	At time of harvest	Gamma isotopic analysis of edible portion of each sample
	One sample of broad leaf vegetation from highest D/Q garden and one sample from 10-20 miles	At time of harvest	I-131 analysis of edible portion of each sample

** Sample locations are given on the figure and table in the ODCM.

*** As determined by methods outlined in the ODCM.

Table 4.16.2
(Page 1 of 2)

MAXIMUM VALUES FOR THE LOWER LIMITS OF DETECTION (LLD)^{a,e}

Analysis	Water (pCi/l)	Airborne Particulate or Gas (pCi/m ³)	Fish (pCi/kg, wet)	Milk (pCi/l)	Food Products (pCi/kg, wet)	Sediment (pCi/kg, dry)
gross beta	4 ^b	1 x 10 ⁻²				
³ H	2000(1000 ^b)					
⁵⁴ Mn	15		130			
⁵⁹ Fe	30		260			
^{58, 60} Co	15		130			
⁶⁵ Zn	30		260			
⁹⁵ Zr-Nb	15 ^c					
¹³¹ I ^d	1 ^b	7 x 10 ⁻²		1	60	
^{134, 137} Cs	15(10 ^b), 18	1 x 10 ⁻²	130	15	60	150
¹⁴⁰ Ba-La	15 ^c			15 ^c		

3.16/4.16

Amendment No. 15, 25, 37, 39

229q

TABLE 4.16.2
(Page 2 of 2)

TABLE NOTATION

- a - The LLD is the smallest concentration of radioactive material in a sample that will be detected with 95% probability with 5% probability of falsely concluding that a blank observation represents a "real" signal.

For a particular measurement system (which may include radiochemical separation):

$$\text{LLD} = \frac{4.66 s_b}{E \cdot V \cdot 2.22 \cdot Y \cdot \exp(-\lambda \Delta t)}$$

where

LLD is the apriori lower limit of detection as defined above (as picocurie per unit mass or volume),

s_b is the standard deviation of the background counting rate or of the counting rate of a blank sample as appropriate (as counts per minute). Typical values of E, V, Y and Δt shall be used in the calculations.

E is the counting efficiency (as counts per transformation)

V is the sample size (in units of mass or volume)

2.22 is the number of transformations per minute per picocurie

Y is the fraction radiochemical yield (when applicable)

λ is the radioactive decay constant for the particular radionuclide

Δt is the elapsed time between sample collection (or end of the sample collection period) and time of counting

- b - LLD for drinking water.
c - Total for parent and daughter.
d - These LLDs apply only where "I-131 analysis" is specified.
e - Where "Gamma Isotopic Analysis" is specified, the LLD specifications applies to the following radionuclides: H-3, MN-54, Fe-59, Co-58, Co-60, Zn-65, Zr-Nb-95, Cs-134, Cs-137 and Ba-La-140. Other peaks which are measurable and identifiable, together with the above nuclides shall be identified and reported.

Table 4.16.3

REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES

Reporting Levels

Analysis	Water (pCi/l)	Airborne Particulate or Gas (pCi/m ³)	Fish (pCi/kg, wet)	Milk (pCi/l)	Vegetables (pCi/kg, wet)
H-3	2×10^4 ^(a)				
Mn-54	1×10^3		3×10^4		
Fe-59	4×10^2		1×10^4		
Co-58	1×10^3		3×10^4		
Co-60	3×10^2		1×10^4		
Zn-65	3×10^2		2×10^4		
Zr-Nb-95	4×10^2 ^(b)				
I-131	2 ^(c)	0.9		3	1×10^2
Cs-134	30	10	1×10^3	60	1×10^3
Cs-137	50	20	2×10^3	70	2×10^3
Ba-La-140	2×10^2 ^(b)			3×10^2 ^(b)	

a - For drinking water samples

b - Total for parent and daughter

c - If no drinking water pathways exist, a value of 20 pCi/l may be used.

6.5 Plant Operating Procedures

Detailed written procedures, including the applicable check-off lists and instructions, covering areas listed below shall be prepared and followed. These procedures and changes thereto, except as specified in 6.5.G shall be reviewed by the Operations Committee and approved by a member of plant management designated by the Plant Manager.

A. Plant Operations

1. Integrated and system procedures for normal startup, operation and shutdown of the reactor and all systems and components involving nuclear safety of the facility.
2. Fuel handling operations.
3. Actions to be taken to correct specific and foreseen potential or actual malfunction of systems or components including responses to alarms, primary system leaks and abnormal reactivity changes and including follow-up actions required after plant protective system actions have initiated.
4. Surveillance and testing requirements that could have an effect on nuclear safety.
5. Implementing procedures of the emergency plan, including procedures for coping with emergency conditions involving potential or actual releases of radioactivity.
6. Implementing procedures of the fire protection program.
7. Implementing procedures for the Process Control Program and Offsite Dose Calculation Manual including quality control measures.

Drills on the procedures specified in A.3 above shall be conducted as a part of the retraining program. Drills on the procedures specified in A.5 above shall be conducted at least semi-annually, including a check of communications with offsite support groups.

B. Radiological

1. a. A Radiation Protection Program, consistent with the requirements of 10 CFR 20, shall be developed and followed. The Radiation Protection Program shall consist of the following:

(1) A Radiation Protection Plan, which shall be a complete definition of radiation protection policy and program

(2) Procedures which implement the requirements of the Radiation Protection Plan

The Radiation Protection Plan and implementing procedures, with the exception of those non-safety related procedures governing work activities exclusively applicable to or performed by health physics personnel, shall be reviewed by the Operations Committee and approved by a member of plant management designated by the Plant Manager. Health physics procedures not reviewed by the Operations Committee shall be reviewed and approved by the Superintendent, Radiation Protection.

b. Paragraph 20.203 "Caution signs, labels, signals and controls." In lieu of the "Control device" or alarm signal required by paragraph 20.203(c)(2), each high radiation area in which the intensity of radiation is 1000 mrem/hr or less shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.

c. The above procedure shall also apply to each high radiation area in which the intensity of radiation is greater than 1000 mrem/hr, except that doors shall be locked or attended to prevent unauthorized entry into these areas and the keys or key devices for locked doors shall be maintained under the administrative control of the Plant Manager.

E. Offsite Dose Calculation Manual (ODCM)

The ODCM shall be approved by the Commission prior to initial implementation. Changes to the ODCM shall satisfy the following requirements:

1. Shall be submitted to the Commission with the Semi-Annual Radioactive Effluent release report for the period in which the change(s) were made effective. This submittal shall contain:
 - a. sufficiently detailed information to totally support the rationale for the change without benefit of additional or supplemental information. Information submitted should consist of a package of those pages of the ODCM to be changed with each page numbered and provided with a revision date, together with appropriate analyses or evaluations justifying the change(s).
 - b. a determination that the change will not reduce the accuracy or reliability of dose calculations or setpoint determinations; and
 - c. documentation of the fact that the change has been reviewed and found acceptable by the Operations Committee.
2. Shall become effective upon review and acceptance by the Operations Committee.

F. Security

Procedures shall be developed to implement the requirements of the Security Plan and the Security Contingency Plan. These implementing procedures, with the exception of those non-safety related procedures governing work activities exclusively applicable to or performed by security personnel, shall be reviewed by the Operations Committee and approved by a member of plant management designated by the Plant Manager. Security procedures not reviewed by the Operations Committee shall be reviewed and approved by the Superintendent, Security and Services.

G. Temporary Changes to Procedures

Temporary changes to those procedures which are required to be reviewed by the Operations Committee described in A, B, C, D, E and F above, which do not change the intent of the original procedures may be made with the concurrence of two individuals holding senior operator licenses. Such changes should be documented, reviewed by the Operations Committee and approved by a member of plant management designated by the Plant Manager within one month. Temporary changes to health physics and security procedures not reviewed by the Operations Committee shall be reviewed by the Superintendent, Radiation Protection for health physics procedures and the Superintendent, Security and Services for security procedures.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. DPR-22
NORTHERN STATES POWER COMPANY
MONTICELLO NUCLEAR GENERATING PLANT
DOCKET NO. 50-263

1.0 INTRODUCTION

By letter dated April 26, 1985, and as supplemented on October 16, 1985, Northern States Power Company (NSP, the licensee) proposed to change the Technical Specifications (TS) to incorporate the recommendations of Generic Letter 84-13, to satisfy a previous commitment and to incorporate miscellaneous administrative changes. The changes were as follows:

1. Snubber Table. Delete Table 3.6.1 and all references to Table 3.6.1 from pages vi, 129, 130, 131 and 132. Delete paragraph 3.6.H.3. The proposed changes implement the recommendations of Generic Letter 84-13 dated May 3, 1984. In paragraph 4.6.H.2, change "impared" to "impaired."
2. Section 6.5.G, Plant Operating Procedures - Temporary Changes to Procedures. Delete the requirements for the Operations Committee to review the temporary changes to non-safety-related health physics and security procedures.
3. Section 6.5.B, Plant Operating Procedures - Radiological. The phrase "and concise statement" is replaced by "definition" to clarify the Radiation Protection Plan. The Monticello Radiation Protection Program and policy is defined in the Plant Administrative Control Directives (ACDs), consistent with the requirements of 10 CFR Part 20.
4. Rod Block Monitor (RBM). In Table 4.2.1, increase the RBM surveillance test frequency to once per month from once per 3 months for exposure hours (M) greater than 2.0×10^5 . The licensee had committed to this change in its letter of October 17, 1984.
5. Section 4.16, Radiation Environmental Monitoring Program. In Table 4.15.1 (page 5 of 5), corn and potato environmental radiation sampling will be limited to the areas that are irrigated by water in which liquid radioactive effluent has been discharged.
6. Miscellaneous Administrative Changes.
 - a) Page 229i. Correct references in TS 4.16.A.4.

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- b) Pages 229q and 229r. Clarify notes "d" and "e" to specify the radionuclides(s) to which the low level doses (LLDs) are acceptable.
 - c) Page 244. Correct drill frequency referenced in procedures from A.6 to A.5.
 - d) Page 244a. Correct typographical errors.
7. I-131 Reportability. In Table 4.16.3, a footnote C is added to clarify the I-131 reportability requirements in environmental samples.

2.0 EVALUATION

Item No. 1 above implements the recommendations of Generic Letter (GL) 84-13 dated May 3, 1984. It provides an option to eliminate snubber listings within the TS provided the snubber TS are modified to specify which snubbers are required to be operable. The recordkeeping requirements of snubbers TS are not altered by this recommendation. The change requested in Item No. 1 does not alter any TS requirements such as snubber quantities, types, or locations. It only eliminates Table 3.6.1 and all references to this table. The staff has reviewed the licensee's proposed changes and concludes that they are in conformance with the guidance provided in GL 84-13 and therefore, are acceptable.

Item No. 2 above deletes the requirements for the Operations Committee to review the temporary changes to non-safety-related health physics and security procedures. In letters dated September 24, 1982 and March 30, 1984, the licensee requested elimination of Operations Committee review of non-safety-related procedures performed by health physics and guard force personnel. This was approved by the staff in Amendment No. 25 dated August 15, 1984. Technical Specification 6.5.G requires Operations Committee review of the temporary changes to procedures. The Operations Committee should not be required to review temporary changes to procedures covering health physics or guard force functions that did not require Operations Committee review when issued. The licensee further states that this review procedure will be omitted only for non-safety-related procedures associated with the activities performed exclusively by health physics or security personnel. The NRC staff has reviewed the licensee's submittal and concludes that the proposed change is acceptable.

Items 3, 5, 6 and 7 above modify portions of the licensee's Radiological Effluent Technical Specifications (RETS). Changes proposed in Items 3, 6, and 7 are strictly administrative in nature and incorporate clarifications contained in the NRC TS guidance for environmental monitoring developed since the issuance of Amendment No. 15. The staff has reviewed the licensee's proposed changes and concludes that these changes do not affect the technical content of the TS and as stated they meet the intent of the NRC model RETS for boiling water reactors (BWRs), NUREG-0743, Revision 2, February 1, 1980 and are, therefore, acceptable.

Item No. 5 above modifies the requirement for potato and corn environmental radiation sampling. Over the period of several years, a uniform process of deep well irrigation has been developed in the area and the use of river water for irrigating potato and corn fields has been discontinued. The collection of potato and corn samples is no longer a valid monitor to determine the impact of liquid release of radioactivity into the river. The land use census specified in the TS does not identify high D/Q corn and potato locations. Leafy green vegetable samples from a garden in the highest D/Q sector are used to determine radioisotope buildup due to air release from the plant.

The sampling of corn and potatoes will be continued but limited to areas that are irrigated by water in which liquid radioactive effluents have been discharged. This change is due to a shift in land use in the vicinity of the plant. In addition, the licensee has agreed to revise Section 5.1 of the Off-site Dose Calculations Manual (ODCM). Revised ODCM Section 5.1 will include, "If the plant begins routine discharges of liquid radioactive effluent into the Mississippi River, a land use survey will be conducted to determine whether any crops are irrigated with water taken from the Mississippi River between the plant discharge canal and a point 5 miles downstream. If edible crops are being irrigated from Mississippi River water, appropriate samples will be collected and analyzed per Technical Specification Table 4.15.1".

The staff has reviewed the licensee's proposed changes to the requirement for potato and corn environmental radiation sampling and to Section 5.1 of the ODCM. These changes meet the intent of the NRC model RETS for BWRs, NUREG-0673, Revision 2 and are, therefore, acceptable.

In Item No. 4, the licensee proposes to increase the RBM test frequency as shown in Table 4.2.1 of the TS from once per 3 months to once per month for exposure hours (M) greater than 2.0×10^5 . This change satisfies the commitment made by the licensee in its letter dated October 17, 1984, "Supplementary Information Related to License Amendment Request dated May 30, 1984, ARTS". The staff had reviewed this item at that time and found this change to be acceptable.

3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes to requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20, changes to the surveillance requirements, and changes in recordkeeping, reporting, and administrative procedures and requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there

has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (10). Pursuant to 10 CFR 51.22(b) no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

4.0 CONCLUSION

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

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Dated: March 13, 1986.