

APR 1 1977

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Docket No. 50-263

Northern States Power Company  
ATTN: Mr. L. O. Mayer, Manager  
Nuclear Support Services  
414 Nicollet Mall - 8th Floor  
Minneapolis, Minnesota 55401

Gentlemen:

In response to your request dated March 8, 1977, the Commission has issued the enclosed Amendment No. 26 to Provisional Operating License No. DPR-22 for the Monticello Nuclear Generating Plant.

The amendment consists of changes to the Technical Specifications to incorporate a specific testing method into the surveillance requirements for the automatic pressure relief system. The specified method will verify that the safety-relief valves actually open when tested.

Copies of the related Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

*15/*  
Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Enclosures:

- 1. Amendment No. 26 to License No. DPR-22
- 2. Safety Evaluation
- 3. Notice

cc w/enclosures:  
See next page

*Amend. 26*

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SURNAME	RPSnaider:ah	RMDiggs	JMMcGough	S.H. Lewis	DRDavis
DATE	3/24/77	3/24/77	3/29/77	3/31/77	4/1/77

cc w/enclosures:

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Office of City Planner  
Grace Building  
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Roseville, Minnesota 55113

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Roisman, Kessler and Cashdan  
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The Environmental Conservation Library  
Minneapolis Public Library  
300 Nicollet Mall  
Minneapolis, Minnesota 55401

Chief, Energy Systems  
Analyses Branch (AW-459)  
Office of Radiation Programs  
U. S. Environmental Protection Agency  
Room 645, East Tower  
401 M Street, S. W.  
Washington, D. C. 20460

U. S. Environmental Protection Agency  
Federal Activities Branch  
Region V Office  
ATTN: EIS COORDINATOR  
230 South Dearborn Street  
Chicago, Illinois 60604

Mr. D. S. Douglas, Auditor  
Wright County Board of Commissioners  
Buffalo, Minnesota 55313

cc w/enclosures and cy of NSPCo  
filing dtd. 3/8/77:  
State Department of Health  
ATTN: Secretary & Executive Officer  
University Campus  
Minneapolis, Minnesota 55440

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 26  
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northern States Power Company (the licensee) dated March 8, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Provisional License No. DPR-22 is hereby amended to read as follows:

3.B Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 26, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: *April 1, 1977*

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ATTACHMENT TO LICENSE AMENDMENT NO.  
PROVISIONAL OPERATING LICENSE NO. DPR-22  
DOCKET NO. 50-263

Replace page 105 of the Technical Specifications contained in Appendix A of the above-indicated license with the attached page 105. The changed area on the revised page is reflected by a marginal line.

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### 3.0 LIMITING CONDITIONS FOR OPERATION

1. Except as specified in 3.5.E.2 and 3.5.E.3 below, the entire automatic pressure relief system shall be operable at any time the reactor pressure is above 150 psig and irradiated fuel is in the reactor vessel.
2. From and after the date that one of the automatic pressure relief system valves is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding ~~seven~~ days unless such valve is sooner made operable, provided that during such ~~seven~~ days both remaining automatic relief system valves and the HPCI system are operable.
3. From and after the date that more than one of the automatic pressure relief valves are made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding 24 hours unless repairs are made and provided that during such time the HPCI system is operable.
4. If the requirements of 3.5.E.1-3 cannot be met, an orderly reactor

### 4.0 SURVEILLANCE REQUIREMENTS

#### 1. Testing:

<u>Item</u>	<u>Frequency</u>
Valve operability	Each operating cycle
Simulated automa- tic actuation test	Each operating cycle

NOTE: Safety/relief valve operability is verified by cycling the valve and observing a compensating change in turbine bypass valve position.

2. When it is determined that one or more automatic pressure relief valves of the Automatic Pressure Relief system is inoperable, the HPCI system shall be demonstrated to be operable immediately and weekly thereafter.

3.5/4.5

105



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 26 TO PROVISIONAL OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY

MONTICELLO NUCLEAR GENERATING PLANT

DOCKET NO. 50-263

INTRODUCTION

By letter dated January 5, 1977, the NRC informed Northern States Power Company (NSP), the licensee, of a potential deficiency in the method being used to confirm valve operability during periodic testing of Boiling Water Reactor (BWR) safety-relief valves. We required that NSP propose a technical specification change, for the Monticello Nuclear Generating Plant, which would eliminate the potential deficiency in the surveillance testing program. NSP submitted the proposed change by letter dated March 8, 1977.

DISCUSSION AND EVALUATION

Some BWR licensees, not including NSP in the case of Monticello, had previously been using safety-relief valve temperature indication as a positive method of confirmation that a safety-relief valve is open when actuated during surveillance testing.

The NRC staff found that an increased temperature indication could be obtained at the safety-relief valve exit, although the safety-relief valve remained closed. The increase in indicated temperature would result from steam vented through the valve actuation mechanism during the surveillance test. Thus, we concluded that a temperature increase at the valve exit does not by itself provide a positive means of verification that the safety-relief valve has opened.

By their letter dated March 8, 1977, NSP proposed adding a note to the safety-relief valve testing specification. This note would document the established NSP manner of testing in that operability of a safety-relief valve is to be verified by cycling the valve at operating pressure and observing a compensating change in turbine bypass valve position. With the pressure regulator setpoint held constant and the

bypass valves initially open, the cycling of the safety-relief valves will cause the bypass valves to close and reopen to compensate for the steam loss. Because this proposed change would document the established practice at Monticello with regard to testing the safety-relief valves and because such practice will result in reliable positive indication of safety-relief valve operation during testing, we found the proposed change acceptable.

#### ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

#### CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: *April 1, 1977*



UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-263

NORTHERN STATES POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. *26* to Provisional Operating License No. DPR-22, issued to Northern States Power Company (the licensee), which revised Technical Specifications for operation of the Monticello Nuclear Generating Plant (the facility) located in Wright County, Minnesota. The amendment is effective as of its date of issuance.

The amendment modified the existing Monticello Technical Specifications to add a note defining the acceptable method of testing safety-relief valves in order that a positive indication of valve operation is obtained during required surveillance testing.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

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The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated March 8, 1977, (2) Amendment No. 26 to License No. DPR-22, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at The Environmental Conservation Library, Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401. A single copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this *16<sup>th</sup> day of April, 1977.*

FOR THE NUCLEAR REGULATORY COMMISSION

Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

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