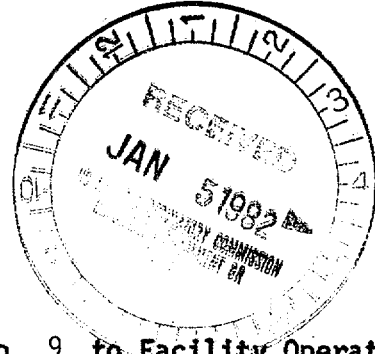


December 28, 1981

Docket No. 50-263

Mr. L. O. Mayer, Manager
Nuclear Support Services
Northern States Power Company
414 Nicollet Mall - 8th Floor
Minneapolis, Minnesota 55401



Dear Mr. Mayer:

The Commission has issued the enclosed Amendment No. 9 to Facility Operating License No. DPR-22 for the Monticello Nuclear Generating Plant. The amendment consists of changes to the Technical Specifications in response to your application dated October 9, 1981 and subsequent discussions between the NRC staff and your staff.

The Technical Specification changes pertain to inservice surveillance requirements for snubbers. We have made certain modifications to your proposed Technical Specification revisions. These changes were discussed with, and agreed to, by your staff.

Your amendment application includes proposed Technical Specifications in response to our letter of November 20, 1980 concerning Technical Specifications for snubber surveillance. The staff has reviewed your application and determined that it is in conformance with the most recent Standard Technical Specifications (STS-NUREG-0123, Rev. 3). Consequently, the Technical Specification changes associated with snubber surveillance are acceptable as an administrative change implementing a previously reviewed and approved action by the Commission.

We have evaluated the potential for environmental impact of plant operation in accordance with the enclosed amendment and have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

OFFICE	8201110046 811228						
SURNAME	PDR ADOCK 05000263						
DATE	PDR						

Mr. L. O. Mayer

2

Since the amendment applies only to administrative details, it does not involve significant new safety information of a type not considered by a previous Commission safety review of the facility. It does not involve a significant increase in the probability or consequences of an accident, does not involve a significant decrease in a safety margin, and therefore does not involve a significant hazards consideration. We have also concluded that there is reasonable assurance that the health and safety of the public will not be endangered by this action.

A copy of the related Notice of Issuance is also enclosed.

Sincerely,

Byron L. Siegel, Project Manager
Operating Reactors Branch #2
Division of Licensing

Enclosures:

- 1. Amendment No. 9 to DPR-22
- 2. Notice

cc: w/enclosures
See next page

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- Gray File
- H. Shaw
- h. Engle
- J. Hegner (5 extra)

Amendment + FR Notices (attached)

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DATE	12/10/81	12/15/81	12/10/81	12/17/81	12/17/81	12/23/81	12/21/81

Mr. L. O. Mayer
Northern States Power Company

cc:

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230 South Dearborn Street
Chicago, Illinois 60604



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated October 9, 1981 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2 of Facility Operating License No. DPR-22 is hereby amended to read as follows:


2. Technical Specifications

The Technical Specifications contained in Appendices A and B as revised through Amendment No. 9 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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PDR ADDCK 05000263
PDR

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing

Attachment:
Changes to the Technical
Specifications.

Date of Issuance: December 28, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 9

FACILITY OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Insert</u>
ii	ii
129	129
130	130
--	130a
--	130b
131	131
132	132
132a	132a
154	154
155	155
247	247
--	274a

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3.0 LIMITING CONDITIONS FOR OPERATION

II. Snubbers

1. Except as permitted below, all snubbers listed in Table 3.6.1 shall be operable above Cold Shutdown. Snubbers may be inoperable in Cold Shutdown and Refueling Shutdown whenever the supported system is not required to be Operable.
2. With one or more snubbers made or found to be inoperable for any reason when Operability is required, within 72 hours:
 - a. Replace or restore the inoperable snubbers to Operable status and perform an engineering evaluation or inspection of the supported components, or
 - b. Determine through engineering evaluation that the as-found condition of the snubber had no adverse effect on the supported components and that they would retain their structural integrity in the event of the design basis seismic event, or
 - c. Declare the supported system inoperable and take the action required by the Technical Specifications for Inoperability of that system.

4.0 SURVEILLANCE REQUIREMENTS

3. The diffuser to lower plenum differential pressure reading on an individual jet pump is 10% or more, less than the mean of all jet pump differential pressures.

II. Snubbers

The following surveillance requirements apply to all snubbers listed in Table 3.6.1.

1. Visual inspection of snubbers shall be conducted in accordance with the following schedule:

No. of Snubbers Found Inoperable per Inspection Period	Next Required Inspection Period
0	18 months + 25%
1	12 months + 25%
2	6 months + 25%
3,4	124 days + 25%
5,6,7	62 days + 25%
8 or more	31 days + 25%

The required inspection interval shall not be lengthened more than one step at a time.

Snubbers may be categorized in two groups, "accessible" or "inaccessible" based on their accessibility for inspection during reactor operation. These two groups may be inspected independently according to the above schedule.

3.0. LIMITING CONDITIONS FOR OPERATION

3. Snubber modifications may be made to safety related systems without prior License Amendment to Table 3.6.1 provided that a revision to the Table is included with the next License Amendment Request.
4. All safety-related snubbers installed or planned for use at Monticello are hydraulic snubbers. No mechanical snubbers are used on safety-related systems at Monticello. If installed in the future, appropriate Technical Specifications changes will be proposed within 60 days of installation.

4.0 SURVEILLANCE REQUIREMENTS

2. Visual inspections shall verify (1) that there are no visible indications of damage or impaired operability and (2) attachments to the supporting structure are secure. Snubbers which appear inoperable as a result of visual inspection may be determined Operable for the purpose of establishing the next visual inspection interval by:
 - a. Clearly establishing the cause of the rejection for that particular snubber and for others that may be generically susceptible; and
 - b. Functionally testing the affected snubber in the as-found condition and finding it Operable per Specification 4.6.11.4.

However, when the fluid plunger gauge of a hydraulic snubber is below low range, the snubber shall be considered inoperable for the purposes of establishing the next visual inspection interval.

3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

3. Functional testing of snubbers shall be conducted at least once per 18 months + 25% during cold shutdown. Ten percent of the total number of each brand of snubber shall be functionally tested either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria in Specification 4.6.11.4 below, an additional ten percent of that brand shall be functionally tested until no more failures are found or all snubbers of that brand have been tested.

The representative sample selected for functional testing shall include the various configurations, operating environments, and the range of size and capacity of the snubbers.

In addition to the regular sample and specified re-samples, snubbers which failed the previous functional test shall be retested during the next test period if they were reinstalled as a safety-related snubber. If a spare snubber has been installed in place of a failed safety related snubber, it shall be tested during the next period.

If any snubber selected for functional testing either fails to lockup or fails to move (i.e. frozen in place) the cause shall be evaluated and if caused by manufacturer or design deficiency, all snubbers of the same design subject to the same defect shall be functionally tested.

3.0 LIMITING CONDITIONS FOR OPERATION

4.0 SURVEILLANCE REQUIREMENTS

4. Hydraulic snubber functional tests shall verify that:
- a. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
 - *b. Snubber bleed, or release rate, where required, is within the specified range in compression or tension.
5. For any snubbers found inoperable, an engineering evaluation or inspection shall be performed on the components which are supported by the snubbers. The purpose of this engineering evaluation or inspection shall be to determine if the components supported by the snubbers were adversely affected by the inoperability of the snubbers in order to ensure that the supported component remains capable of meeting the designed service.
6. The installation and maintenance records for each snubber listed in Table 3.6.1 shall be reviewed at least once every 18 months to verify that the indicated service life will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded, the snubber service life shall be re-evaluated or the snubber shall be replaced or reconditioned to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement, or reconditioning shall be indicated in the records.

*This requirement is effective on July 1, 1983. Until that time, only snubber bleed, or release, shall be demonstrated.

TABLE 3.6.1
SAFETY RELATED HYDRAULIC SNUDDERS

SNUGGLER NO.	SYSTEM	LOCATION	ELEVATION	AZIMUTH (AIRLOCK O REF)	ACCESSIBLE -A (INACCESSIBLE -I)
PS1-H2	MAIN STEAM	DRYWELL	959	071	
PS1-H3	MAIN STEAM	DRYWELL	950	148	
PS2-H2	MAIN STEAM	DRYWELL	950	120	
PS3-H2	MAIN STEAM	DRYWELL	950	240	
PS4-H3	MAIN STEAM	DRYWELL	950	212	
RV29-H3	SAFETY-RELIEF	DRYWELL	950	110	
RV29-H4	SAFETY-RELIEF	DRYWELL	935	100	
RV29-H5	SAFETY-RELIEF	DRYWELL	935	110	
RV29-N1	SAFETY-RELIEF	DRYWELL	935	110	
RV29-N1	SAFETY-RELIEF	DRYWELL	935	090	
RV29-N1	SAFETY-RELIEF	DRYWELL	947	048	
RV29-N1	SAFETY-RELIEF	DRYWELL	939	115	
RV29-N1	SAFETY-RELIEF	DRYWELL	939	052	
RV29-N1	SAFETY-RELIEF	DRYWELL	950	086	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	160	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	180	
RV29-N1	SAFETY-RELIEF	DRYWELL	948	190	
RV29-N1	SAFETY-RELIEF	DRYWELL	946	190	
RV29-N1	SAFETY-RELIEF	DRYWELL	934	180	
RV29-N1	SAFETY-RELIEF	DRYWELL	945	120	
RV29-N1	SAFETY-RELIEF	DRYWELL	945	120	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	135	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	200	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	200	
RV29-N1	SAFETY-RELIEF	DRYWELL	947	200	
RV29-N1	SAFETY-RELIEF	DRYWELL	947	200	
RV29-N1	SAFETY-RELIEF	DRYWELL	956	200	
RV29-N1	SAFETY-RELIEF	DRYWELL	940	250	
RV29-N1	SAFETY-RELIEF	DRYWELL	935	250	
RV29-N1	SAFETY-RELIEF	DRYWELL	950	250	
RV29-N1	SAFETY-RELIEF	DRYWELL	951	250	
RV29-N1	SAFETY-RELIEF	DRYWELL	950	320	
RV29-N1	SAFETY-RELIEF	DRYWELL	950	230	
RV29-N1	SAFETY-RELIEF	DRYWELL	945	270	
RV29-N1	SAFETY-RELIEF	DRYWELL	945	270	
RV29-N1	SAFETY-RELIEF	DRYWELL	956	270	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	290	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	290	
RV29-N1	SAFETY-RELIEF	DRYWELL	938	290	
RV29-N1	SAFETY-RELIEF	DRYWELL	950	270	
RV29-N1	SAFETY-RELIEF	DRYWELL	953	270	
RV29-N1	SAFETY-RELIEF	DRYWELL	922	270	
RV29-N1	SAFETY-RELIEF	DRYWELL	922	315	
RV29-N1	SAFETY-RELIEF	DRYWELL	922	135	
RV29-N1	SAFETY-RELIEF	DRYWELL	952	302	
RV29-N1	SAFETY-RELIEF	DRYWELL	952	058	
RV29-N1	SAFETY-RELIEF	DRYWELL	952	258	
RV29-N1	SAFETY-RELIEF	DRYWELL	952	096	
SS-1	MAIN STEAM	DRYWELL	953	270	
SS-1A	RECIRCULATION	DRYWELL	922	270	
SS-1B	RECIRCULATION	DRYWELL	922	315	
SS-11	FEEDWATER	DRYWELL	952	135	
SS-12	FEEDWATER	DRYWELL	952	302	
SS-13	FEEDWATER	DRYWELL	952	058	
SS-14	FEEDWATER	DRYWELL	952	258	
SS-14	FEEDWATER	DRYWELL	952	096	

TABLE 3.6-1
SAFETY RELATED HYDRAULIC SNUDDERS

SNUDDER NO.	SYSTEM	LOCATION	ELEVATION	ALTIHUB (AIRLOCK 0 REF)	ACCESSIBLE - INACCESSIBLE-1
SS-17A	RHM	DRYWELL	505	072	1
SS-17B	RHM	DRYWELL	505	072	1
SS-17A	RHM	DRYWELL	505	072	1
SS-17B	RHM	DRYWELL	505	072	1
SS-18	RHM	DRYWELL	505	072	1
SS-19	RHM	DRYWELL	505	072	1
SS-20	RHM	DRYWELL	505	072	1
SS-21	RHM	DRYWELL	505	072	1
SS-22	RHM	DRYWELL	505	072	1
SS-23	RHM	DRYWELL	505	072	1
SS-24	RHM	DRYWELL	505	072	1
SS-25	RHM	DRYWELL	505	072	1
SS-26	RHM	DRYWELL	505	072	1
SS-27	RHM	DRYWELL	505	072	1
SS-28A	RHM	DRYWELL	505	072	1
SS-28B	RHM	DRYWELL	505	072	1
SS-29	RHM	DRYWELL	505	072	1
SS-30	RHM	DRYWELL	505	072	1
SS-31	RHM	DRYWELL	505	072	1
SS-32A	RHM	DRYWELL	505	072	1
SS-32B	RHM	DRYWELL	505	072	1
SS-33	RHM	DRYWELL	505	072	1
SS-34	RHM	DRYWELL	505	072	1
SS-35	RHM	DRYWELL	505	072	1
SS-36A	RHM	DRYWELL	505	072	1
SS-36B	RHM	DRYWELL	505	072	1
SS-37	RHM	DRYWELL	505	072	1
SS-38	RHM	DRYWELL	505	072	1
SS-39	RHM	DRYWELL	505	072	1
SS-40	RHM	DRYWELL	505	072	1
SS-41A	RHM	DRYWELL	505	072	1
SS-41B	RHM	DRYWELL	505	072	1
SS-42A	RHM	DRYWELL	505	072	1
SS-42B	RHM	DRYWELL	505	072	1
SS-43	RHM	DRYWELL	505	072	1
SS-44	RHM	DRYWELL	505	072	1
SS-45	RHM	DRYWELL	505	072	1
SS-46	RHM	DRYWELL	505	072	1
SS-47	RHM	DRYWELL	505	072	1
SS-48	RHM	DRYWELL	505	072	1
SS-49	RHM	DRYWELL	505	072	1
SS-50	RHM	DRYWELL	505	072	1
SS-51	RHM	DRYWELL	505	072	1
SS-52	RHM	DRYWELL	505	072	1
SS-53	RHM	DRYWELL	505	072	1
SS-54	RHM	DRYWELL	505	072	1
SS-55	RHM	DRYWELL	505	072	1
SS-56	RHM	DRYWELL	505	072	1
SS-57	RHM	DRYWELL	505	072	1
SS-58	RHM	DRYWELL	505	072	1
SS-59	RHM	DRYWELL	505	072	1
SS-60	RHM	DRYWELL	505	072	1
SS-61	RHM	DRYWELL	505	072	1
SS-62	RHM	DRYWELL	505	072	1
SS-63	RHM	DRYWELL	505	072	1
SS-64	RHM	DRYWELL	505	072	1
SS-65	RHM	DRYWELL	505	072	1
SS-66	RHM	DRYWELL	505	072	1
SS-67	RHM	DRYWELL	505	072	1
SS-68	RHM	DRYWELL	505	072	1
SS-69	RHM	DRYWELL	505	072	1
SS-70	RHM	DRYWELL	505	072	1
SS-71	RHM	DRYWELL	505	072	1
SS-72	RHM	DRYWELL	505	072	1
SS-73	RHM	DRYWELL	505	072	1
SS-74	RHM	DRYWELL	505	072	1
SS-75	RHM	DRYWELL	505	072	1
SS-76	RHM	DRYWELL	505	072	1
SS-77	RHM	DRYWELL	505	072	1
SS-78	RHM	DRYWELL	505	072	1
SS-79	RHM	DRYWELL	505	072	1
SS-80	RHM	DRYWELL	505	072	1
SS-81	RHM	DRYWELL	505	072	1
SS-82	RHM	DRYWELL	505	072	1
SS-83	RHM	DRYWELL	505	072	1
SS-84	RHM	DRYWELL	505	072	1
SS-85	RHM	DRYWELL	505	072	1
SS-86	RHM	DRYWELL	505	072	1
SS-87	RHM	DRYWELL	505	072	1
SS-88	RHM	DRYWELL	505	072	1
SS-89	RHM	DRYWELL	505	072	1
SS-90	RHM	DRYWELL	505	072	1
SS-91	RHM	DRYWELL	505	072	1
SS-92	RHM	DRYWELL	505	072	1
SS-93	RHM	DRYWELL	505	072	1
SS-94	RHM	DRYWELL	505	072	1
SS-95	RHM	DRYWELL	505	072	1
SS-96	RHM	DRYWELL	505	072	1
SS-97	RHM	DRYWELL	505	072	1
SS-98	RHM	DRYWELL	505	072	1
SS-99	RHM	DRYWELL	505	072	1
SS-100	RHM	DRYWELL	505	072	1

3.6/4.6 Amendment No. 2, 9

TABLE 3.6.1
SAFETY RELATED HYDRAULIC SHIPPERS

SHIPPER NO.	SYSTEM	LOCATION	ELEVATION	ALTIMETER (AIRLOCK OR REF.)	ACCESSIBLE - INACCESSIBLE-1
SS-37	HPCL	HPCL ROOM - W WALL	505		A
SS-38A	KLIC	KLIC ROOM - W WALL	506		A
SS-38B	KLIC	KLIC ROOM - W WALL	506		A
SS-41	LDME SHAW	ABOVE TONS CRANE	523		A
SS-42	HPCL	ABOVE TONS KING HDR	508		A

Bases Continued 3.6 and 4.6:

A nozzle-riser system failure could also generate the coincident failure of a jet pump body; however, the converse is not true. The lack of any substantial stress in the jet pump body makes failure impossible without an initial nozzle-riser system failure.

II. Snubbers

All snubbers are required to be operable above Cold Shutdown to ensure that the structural integrity of the reactor coolant system and all other safety related systems is maintained during and following a seismic or other event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on non-safety related systems and then only if their failure or failure of the system on which they are installed would have no adverse effect on any safety-related system.

The visual inspection frequency is based upon maintaining a constant level of snubber protection to systems. Therefore, the required inspection interval varies inversely with the observed snubber failures and is determined by the number of inoperable snubbers found during an inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, and verified by inservice functional testing, that snubber may be exempted from being counted as inoperable. Generically susceptible snubbers are those which are of a specific make or model and have the same design feature directly related to rejection of the snubber.

When a snubber is found inoperable, an engineering evaluation or inspection is performed, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber. The evaluation or inspection will determine whether or not the snubber mode of failure has imparted a significant effect or degradation on the supported component or system.

II. Snubbers (continued)

To provide assurance of snubber functional reliability, a representative sample of 10% of the installed snubbers will be functionally tested during plant shutdowns at intervals of no more than 18 months \pm 25%. Observed failures of these sample snubbers will require functional testing of additional units.

The service life of a snubber is evaluated via manufacturer input and through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc. . .). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life.

5. Principal maintenance activities, including inspection, repairs and substitution or replacement of principal items of equipment pertaining to nuclear safety.
6. Records of changes to plant procedures and records of special tests and experiments.
7. Records of wind speed and direction.
8. Records of individual plant staff members showing qualifications, training and retraining.
9. Reportable Occurrences.

B. Records Retained for Plant Life

Records and logs relative to the following items shall be retained for the life of the plant:

1. Liquid and gaseous radioactive releases to the environs
2. Radiation exposures for all plant, visitor and contractor personnel
3. Off-site environmental monitoring surveys
4. Fuel accountability including new and spent fuel inventories and transfers, and fuel assembly histories
5. Radioactive shipments
6. Plant radiation and contamination surveys
7. Changes made to the plant as it is described in the Final Safety Analysis Report, reflected in updated, corrected and as-built drawings
8. Cycling beyond normal limits for those components that have been designed to operate safely for a limited number of cycles beyond such limits
9. Reactor coolant system in-service inspections
10. Minutes of meetings of the Safety Audit Committee

B. Records Retained for Plant Life (continued)

- 11. Records for Environmental Qualification which are covered under the provisions of paragraph 6.8.**
- 12. Records of the service lives of all safety-related snubbers, including the date at which the service life commences and associated installation and maintenance records.**

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-263NORTHERN STATES POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSE

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 9 to Facility Operating License No. DPR-22 issued to Northern States Power Company which revises the Technical Specifications for operation of the Monticello Nuclear Generating Plant located in Wright County, Minnesota. The amendment is effective as of its date of issuance.

The amendment modifies the Technical Specifications which pertain to inservice inspection requirements for snubbers.

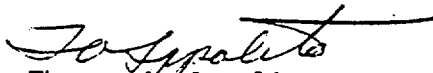
The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated October 9, 1981, (2) Amendment No. 9 to License No. DPR-22, and (3) the Commission's letter to Northern States Power Company dated December 28, 1981. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental Conservation Library, Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota. A copy of items (2) and (3) may be obtained upon request addressed to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 28th day of December 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Thomas A. Ippolito, Chief
Operating Reactors Branch #2
Division of Licensing