

Docket No. 50-263

JAN 25 1978

Northern States Power Company  
ATTN: Mr. L. O. Mayer, Manager  
Nuclear Support Services  
414 Nicollet Mall - 8th Floor  
Minneapolis, Minnesota 55401

Gentlemen:

By letter dated November 25, 1977, we forwarded to you recommended interim technical specifications for fire protection at the Monticello Nuclear Generating Plant. These specifications revised, where necessary, the specifications submitted in your January 31, 1977 response to our September 30, 1976 letter.

You responded to our November 25, 1977 letter by letter dated December 16, 1977, in which you took exception to three particular specifications.

In order to achieve expeditious implementation of the fire protection Technical Specifications, Specification 6.1.C.6 is being issued at this time with the minimum number of on-site fire brigade members specified as 3 as you proposed. This number is less than the minimum number given in the generic staff position, "Minimum Fire Brigade Shift Size," which was an attachment to the Safety Evaluation issued with our letter of November 25, 1977. However, we are presently evaluating your justifications for this smaller brigade size, and when the evaluation is completed, the minimum number will be increased if we do not agree with your position.

The same consideration is being given to Specification 4.13.B.1.k (concerning valve position verification) and Figure 6.1.2. (concerning specification of responsible fire protection personnel). Specification 4.13.B.1.k and Figure 6.1.2 have been issued as requested in your December 16, 1977 letter. However, upon completion of our further review, we may require that the specification and figure be modified.



JAN 25 1978

The enclosed fire protection Technical Specifications for Monticello are being issued as Amendment No. 33 to Provisional Operating License No. DPR-22. A copy of the Notice of Issuance is enclosed. The related Safety Evaluation was enclosed with our November 25, 1977 letter.

Sincerely,

Original Signed by  
Don K. Davis

Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Enclosures:

1. Amendment No. 33 to DPR-22
2. Notice

cc w/enclosures: See next page

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January 25, 1978

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\*(w/ cy of NSP filings 1/31/77 &  
12/16/77)



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

NORTHERN STATES POWER COMPANY  
DOCKET NO. 50-263  
MONTICELLO NUCLEAR GENERATING PLANT  
AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 33  
License No. DPR-22

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Northern States Power Company (the licensee) dated January 31, 1977, as amended by letter dated December 16, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

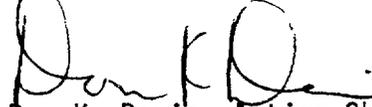
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B of Facility License No. DPR-22 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 33, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: January 25, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 33

PROVISIONAL OPERATING LICENSE NO. DPR-22

DOCKET NO. 50-263

Replace the following pages of the Technical Specifications contained in Appendix A of the above indicated license with the attached pages. The changed areas on revised pages are reflected by a marginal line.

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189T (new)  
189U (new)  
189V (new)  
189W (new)  
189X (new)  
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## INTRODUCTION

These Technical Specifications are prepared in accordance with the requirements of 10 CFR 50.36 and apply to the Monticello Nuclear Generating Plant, Unit No. 1. The bases for these Specifications are included for information and understandability purposes.

### 1.0 DEFINITIONS

The succeeding frequently used terms are explicitly defined so that a uniform interpretation of the Specifications may be achieved.

#### A. Alteration of the Reactor Core

The act of moving any component in the region above the core support plate, below the upper grid and within the shroud. (Normal operating functions such as control rod movement using the normal drive mechanism, tip scans, SRM and IRM detector movements, etc., are not to be considered core alterations.)

#### B. Hot Standby

Hot Standby means operation with the reactor critical in the startup mode at a power level just sufficient to maintain reactor pressure and temperature.

#### C. Fire Suppression Water System

The fire suppression water system consists of: water sources; pumps; and distribution piping with associated sectionalizing isolation valves. Such valves include yard hydrant valves, and the first valve ahead of the water flow alarm device on each sprinkler, hose standpipe, or spray system riser.

### 3.0 LIMITING CONDITIONS FOR OPERATION

#### 3.13 FIRE DETECTION AND PROTECTION SYSTEMS

##### Applicability:

Applies to instrumentation and plant systems used for fire detection and protection of the nuclear safety-related structures, systems, and components of the plant.

##### Objective:

To insure that the structures, systems, and components of the plant important to nuclear safety are protected from fire damage.

##### Specification:

##### A. Fire Detection Instrumentation

1. In each fire detection zone containing equipment required to be operable, at least (N-1) of the fire detection instruments located in that zone shall be operable, where N is the total number of fire detection instruments installed in the zone.
2. If specification 3.13.A.1 cannot be met, within one hour establish a fire watch patrol to inspect the zone(s) with inoperable instrument(s) at least once per hour. Restore at least (N-1) of the instruments to operable status within 14 days or submit a 30-day written report outlining the cause of the malfunction and the plans for restoring the instruments to operable status.

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### 4.0 SURVEILLANCE REQUIREMENTS

#### 4.13 FIRE DETECTION AND PROTECTION SYSTEMS

##### Applicability:

Applies to the periodic testing of instrumentation and plant systems used for fire detection and protection of the nuclear safety related structures, systems, and components.

##### Objective:

To verify the operability of instrumentation and plant systems used for fire detection and protection of nuclear safety related structures, systems, and components.

##### Specification:

##### A. Fire Detection Instrumentation

1. Fire detectors located in the cable spreading room, standby gas treatment system room, 4.16 KV switchgear area, and intake structure shall be tested in accordance with the following schedule:  
  
A functional test shall be performed every six months.
2. Fire detector alarm circuitry shall be demonstrated operable every six months.

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3.0 LIMITING CONDITIONS FOR OPERATION

B. Fire Suppression Water System

1. Except as specified in 3.13.B.2 or 3.13.B.3 below, the system shall be operable at all times with:
  - a. At least two of the following pumps, including automatic initiation logic, operable and capable of delivering at least 1500 gpm at a discharge pressure of 90 psig:
    1. Diesel-driven fire pump
    2. Motor-driven fire pump
    3. Screen wash pump
  - b. Piping and correctly positioned valves to supply fire suppression water to all safety related structures, systems, and components.
2. If Specification 3.13.B.1 cannot be met, restore the inoperable equipment to operable status within seven days or provide a 30-day written report outlining the plans and procedures to be used to provide for the loss of redundancy in the Fire Suppression Water system.

4.0 SURVEILLANCE REQUIREMENTS

B. Fire Suppression Water System

1. The system shall be verified operable as follows:
  - a. Operability of the diesel-driven fire pump starting battery shall be demonstrated by:
    1. Once each week verify electrolyte level and voltage is within specifications.
    2. Once every three months verify the specific gravity of each cell is within specifications.
    3. Once every 18 months inspect the batteries, battery racks, and electrical connections for damage or abnormal deterioration.
  - b. The motor-driven fire pump shall be started every month and run for at least 15 minutes on recirculation flow.
  - c. The diesel-driven fire pump shall be started every month from ambient conditions and run for at least 20 minutes on recirculation flow.
  - d. The level in the diesel-driven fire pump day tank shall be checked every month and verified to contain at least 65 gallons of fuel.

### 3.0 LIMITING CONDITIONS FOR OPERATION

### 4.0 SURVEILLANCE REQUIREMENTS

- e. Every three months verify that a sample of fuel from the diesel oil storage tank, obtained in accordance with ASTM-D270-65, is within the acceptable limits specified in Table 1 of ASTM D475-74 when checked for viscosity, water, and sediment.
- f. Every 18 months subject the diesel-driven fire pump engine to an inspection in accordance with procedures prepared in conjunction with the manufacturer's recommendations for this class of standby service.
- g. A simulated automatic actuation of each fire pump and the screen wash pump, including verification of pump capability, shall be conducted every 18 months.
- h. The yard main and the reactor building heater shall be flushed every 12 months.
- i. System flow tests shall be performed every three years.
- j. Valves in flow paths supplying fire suppression water to safety related structures, systems, and components shall be cycled every 12 months.
- k. Each valve (manual, power operated, or automatic) in the flow path that is not electrically supervised, locked, sealed, or otherwise secured in position, shall be verified to be in its correct position every month.

3.13/4.13

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3. From and after the date that the supply of fire suppression water to any safety-related structure, system, or component is made or found to be interrupted for any reason, within 24 hours:
  - a. Establish a backup Fire Suppression Water System.
  - b. Provide prompt notification with a written followup report outlining the actions taken and the plans and schedule for restoring the system to operable status.
  - c. If Specification 3.13.B.3.a cannot be met, the reactor shall be placed in hot standby within 6 hours and in cold shutdown within 30 hours.

#### C. Hose Stations

1. Whenever equipment protected by hose stations in the following areas is required to be operable, the hose station(s) protecting those areas shall be operable:
  - a. Diesel generator rooms
  - b. Safety related switchgear areas
  - c. Safety related areas of the screenhouse
  - d. Reactor building
  - e. Control room
  - f. Cable spreading room
  - g. Safety related battery rooms

2. When it is determined that one of the two pumps required by specification 3.13.B.1.a is inoperable, the remaining operable pump shall be started daily and run for at least 20 minutes on recirculation flow until specification 3.13.B.1.a can be met.

#### C. Hose Stations

The hose stations specified in 3.13.C.1 shall be demonstrated operable as follows:

1. Each month a visual inspection shall be conducted to assure all equipment is available.
2. Every 18 months the hose shall be removed for inspection and re-racking and all gaskets in the couplings shall be inspected and replaced if necessary.
3. Every 3 years each hose station valve shall be partially opened to verify valve operability and no flow blockage.
4. Every 3 years each hose shall be hydrostatically tested at a pressure at least 50 psig greater than the maximum pressure available at that hose station. (This specification is effective as of January 1981).

3.0 LIMITING CONDITIONS FOR OPERATION

2. If Specification 3.13.C.1 cannot be met, hoses supplied from operable hose stations shall be made available for routing within one hour to each area with an inoperable hose station.

D. Fire Barrier Penetration Fire Seals

1. All penetration fire barriers protecting areas having equipment required to be operable shall be operable.
2. If Specification 3.14.D.1 cannot be met, a continuous fire watch shall be established on at least one side of the affected fire barrier(s) within one hour.

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4.0 SURVEILLANCE REQUIREMENTS

D. Fire Barrier Penetration Fire Seals

1. A visual inspection of fire barrier penetration fire seals shall be conducted every 18 months.
2. Following repair of a fire barrier penetration fire seal, a visual inspection of the seal shall be conducted.

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### 3.13 BASES:

Elements of the fire detection and protection system are required to be operable to protect safety related structures, systems, and components whenever those structures, systems, or components are required to be operable. Fire detection and protection systems will normally be maintained operable at all times except for periods of maintenance and testing.

Ionization type fire detectors are located in the following areas of the plant to protect safety-related structures, systems, and components:

Cable spreading room  
Standby gas treatment system room  
4.16 KV switchgear area  
Intake structure

These detectors sense the airborne products of combustion during the very early stages of a fire. The detectors in each area initiate a local alarm and an alarm in the Control Room. The specifications require all but one detector in each area to be operable. Since there are at least three detectors in each area, the loss of one detector does not significantly degrade the ability to detect fires in any area. If more than one detector is inoperable a patrolling fire watch is established in the affected area until the required number of detectors are restored to operable status.

The fire suppression water system is supplied by three vertical centrifugal pumps rated at 1500 gpm at 100 psig each. Two pumps, one motor driven and one diesel driven, are the assigned fire pumps. The third pump is motor driven and normally assigned as a screen wash pump. Transfer from screen wash duty to fire duty occurs automatically. All pumps are started automatically by instrumentation sensing header pressure. Any one pump is capable of supplying all fire fighting water requirements. Two of the three pumps are required to be operable. If two pumps are inoperable, at least one must be repaired within seven days or a report must be submitted to the Commission. If all pumps are inoperable, or if other circumstances interrupt the supply of water to any safety related area, a backup source of water must be provided within 24 hours and the Commission notified.

Fire protection for all safety related areas is provided by hose stations supplied from the fire suppression water system. If the water supply to these areas is interrupted, continued operation is permitted only if a hose supplied from an operable hose station is made available to protect the area having the inoperable station.

Piping and cabling penetrating fire barriers are provided with fire seals at each fire barrier. If a seal is made or found to be inoperable for any reason, the penetration area is continuously attended until an effective fire seal is restored.

### 3.13 BASES

#### 4.13 BASES:

Fire detectors are tested in accordance with the manufacturer's recommendations. All tests and inspections are performed by the plant staff. Every six months each detector is functionally tested. Combustion generated smoke is not used in these tests. Alarm circuits are functionally checked every six months. In addition, all circuitry is automatically supervised for open wiring and ground faults.

Fire pumps are tested each month to verify operability. Test starting of the screen wash pump is not required since it is normally in service. Each fire pump is manually started and operated for at least 15 minutes with pump flow directed through the recirculation test line. Every 18 months the operability of the automatic actuation logic for the fire pumps and the screen wash pump is verified and the performance of each pump is verified to meet system requirements. The specified flush and valve checks provide assurance that the piping system is capable of supplying fire suppression water to all safety related areas.

When one of the two required pumps is inoperable, the operable pump is started daily to verify operability until two pumps are once again available.

A system flow test is specified every three years. This test verifies the hydraulic performance of the fire suppression fire water header system. The testing will be performed using Section 11, Chapter 5 of the Fire Protection Handbook, 14th Edition, as a procedural guide. This test is generally performed in conjunction with a visit from insurance company inspectors.

Hose stations in safety related areas are inspected monthly to verify that all required equipment is in place. Hose station gaskets in hose couplings and the hose are inspected every 18 months. Operability of hose station isolation valves is verified every three years by partially opening each valve to verify flow. All of these tests provide a high degree of assurance that each hose station will perform satisfactorily after periods of standby service.

Plant fire barrier walls are provided with seals for pipes and cables. Where such seals are installed, they must be maintained intact to perform their function. Visual inspection of each installed seal is required every 18 months and after seal repair. A visual inspection following repair of a seal in the secondary containment boundary is sufficient to assure that seal leakage will be within acceptable limits.

#### 4.13 BASES

## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 Organization

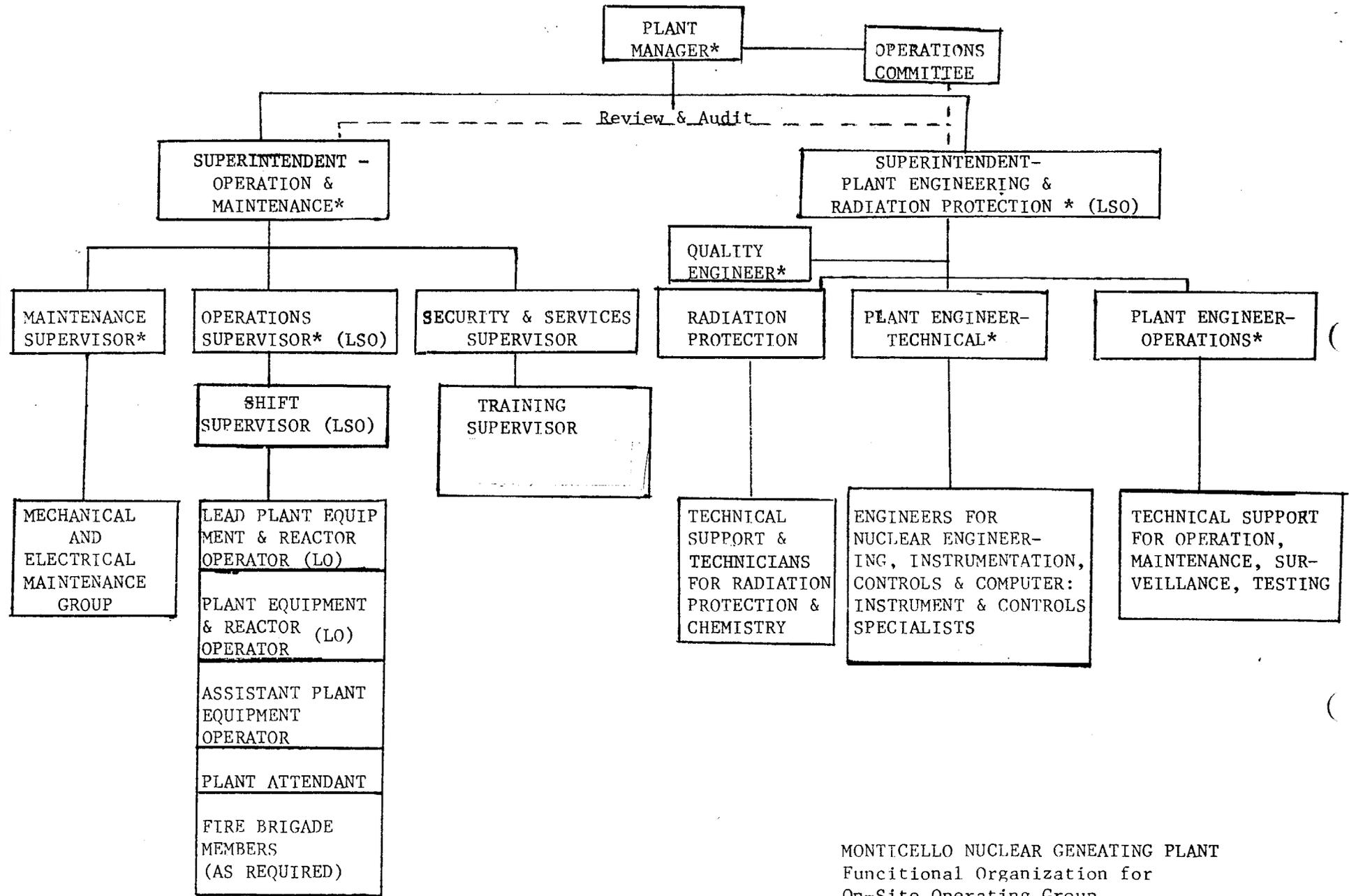
- A. The Plant Manager has the overall full-time onsite responsibility for safe operation of the facility. During periods when the Plant Manager is unavailable, he may delegate this responsibility to other qualified supervisory personnel.
- B. The Northern States Power corporate organizational structure relating to the operation of this plant is shown in Figure 6.1.1.
- C. The minimum functional organization for operation of the plant shall be as shown in Figure 6.1.2 and:
  - 1. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.1.1.
  - 2. At least once licensed operator shall be in the control room when fuel is in the reactor.
  - 3. At least two licensed operators shall be present in the control room during cold startup, scheduled reactor shutdown, and during recovery from reactor trips.
  - 4. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
  - 5. All alterations of the reactor core shall be directly supervised by a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
  - 6. A fire brigade of at least three members shall be maintained on site at all times. The fire brigade shall not include the four members of the shift organization required for safe shutdown of the reactor or more than one member of the site security force.
- D. Minimum qualifications, training, replacement training and retraining of plant personnel shall be in accordance with that stated in the "Standard for Selection and Training of Personnel for Nuclear Power Plants" ANSI N18.1-1971. The Radiation Protection supervisor shall meet the minimum qualifications in Regulatory Guide 1.8, September, 1975. The minimum frequency of the retraining program shall be every two years. The training program shall be under the direction of a designated member of the plant staff.

- E. A training program for individuals serving in the fire brigade shall be maintained under the direction of a designated member of the plant staff. This program shall meet the requirements of Section 27 of the NFPA Code-1976 with the exception of training scheduling. Fire brigade training shall be scheduled as set forth in the plant training program.

6.1

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MONTICELLO NUCLEAR GENERATING PLANT  
 Functional Organization for  
 On-Site Operating Group

CODE: \* Key Supervisor  
 LO Licensed Operator  
 LSO Licensed Senior Operator

FIGURE 6.1.2

TABLE 6.1.1

## MINIMUM SHIFT CREW COMPOSITION (Note 1)

CATEGORY	APPLICABLE PLANT CONDITIONS	
	COLD SHUTDOWN OR REFUELING OPERATION	ABOVE COLD SHUTDOWN
No. Licensed Senior Operators (LSO)	1 (Note 2)	1
Total No. Licensed Operators (LSO & LO)	2	3
Total No. Licensed and Unlicensed personnel	3	5

NOTES:

- Shift crew composition may be one less than the minimum requirements for a period of time not to exceed two hours in order to accommodate an unexpected absence of one duty shift crew member provided immediate action is taken to restore the shift crew composition to within the minimum requirements specified.
- Does not include the licensed Senior Reactor Operator, or Senior Reactor Operator Limited to Fuel Handling, supervising alterations of the reactor core.

- f. Investigation of all events which are required by regulation or technical specifications to be reported to NRC in writing within 24 hours.
  - g. Revisions to the Facility Emergency Plan, the Facility Security Plan, and the Fire Protection Program.
  - h. Operations Committee minutes to determine if matters considered by that Committee involve unreviewed or unresolved safety questions.
  - i. Other nuclear safety matters referred to the SAC by the Operations Committee, plant management or company management.
  - j. All recognized indications of an unanticipated deficiency in some aspect of design or operation of safety-related structures, systems, or components.
  - k. Reports of special inspections and audits conducted in accordance with specification 6.3.
6. Audit - The operation of the nuclear power plant shall be audited formally under the cognizance of the SAC to assure safe facility operation.
- a. Audits of selected aspects of plant operation, as delineated in Paragraph 4.4 of ANSI N18.7-1972, shall be performed with a frequency commensurate with their nuclear safety significance and in a manner to assure that an audit of all nuclear safety-related activities is completed within a period of two years. The audits shall be performed in accordance with appropriate written instructions and procedures.
  - b. Periodic review of the audit program should be performed by the SAC at least twice a year to assure its adequacy.
  - c. Written reports of the audits shall be reviewed by the Vice President - Power Production & System Operation, by the SAC at a scheduled meeting, and by members of management having responsibility in the areas audited.

7. Authority

The SAC shall be advisory to the Vice President - Power Production & System Operation.

8. Records

Minutes shall be prepared and retained for all scheduled meetings of the Safety Audit Committee. The minutes shall be distributed to the Vice President-Power Production & System Operation, the General Superintendent of Nuclear Power Plant Operation, each member of the SAC and others designated by the Chairman or Vice Chairman within one month of the meeting. There shall be a formal approval of the minutes.

9. Procedures

A written charter for the SAC shall be prepared that contains:

- a. Subjects within the purview of the group.
- b. Responsibility and authority of the group.
- c. Mechanisms for convening meetings.
- d. Provisions of use of specialists or subgroups.
- e. Authority to obtain access to the nuclear power plant operating record files and operating personnel when assigned audit functions.
- f. Requirements for distribution of reports and minutes prepared by the group to others in the NSP Organization.

- c. Mechanism for scheduling meetings
- d. Meeting agenda
- e. Use of subcommittee
- f. Review and approval, by members, of OC actions
- g. Distribution of minutes

### 6.3 Special Inspections and Audits

- A. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified off-site Northern States Power Company personnel or an outside fire protection consultant.
- B. An inspection and audit by an outside qualified fire protection consultant shall be performed at intervals no greater than three years.

### 6.4 Action to be Taken if a Safety Limit is Exceeded

If a Safety Limit is exceeded, the reactor shall be shut down immediately. An immediate report shall be made to the General Superintendent of Nuclear Power Plant Operation, or his designated alternate in his absence, and reported as specified in Section 6.7. A complete analysis of the circumstances leading up to and resulting from the situation, together with recommendations by the Operations Committee, shall also be prepared. This report shall be submitted to the General Superintendent of Nuclear Power Plant Operation and the Chairman of the Safety Audit Committee.

Reactor operation shall not be resumed until authorized by the U. S. Nuclear Regulatory Commission.

## 6.5 Plant Operating Procedures

Detailed written procedures, including the applicable check-off and instructions, covering areas listed below shall be prepared and followed. These procedures and changes thereto, except as specified in 6.5.D shall be reviewed by the Operation Committee and approved by a member of plant management designated by the Plant Manager.

### A. Plant Operations

1. Integrated and system procedures for normal startup, operation and shutdown of the reactor and all systems and components involving nuclear safety of the facility.
2. Fuel handling operations.
3. Actions to be taken to correct specific and foreseen potential or actual malfunction of systems or components including responses to alarms, primary system leaks and abnormal reactivity changes and including follow-up actions required after plant protective system actions have initiated.
4. Surveillance and testing requirements that could have an effect on nuclear safety.
5. Implementing procedures of the security plan.
6. Implementing procedures of the emergency plan, including procedures for coping with emergency conditions involving potential or actual releases of radioactivity.
7. Implementing procedures of the fire protection program.

Drills on the procedures specified in A.3 above shall be conducted as a part of the retraining program. Drills on the procedures specified in A.6 above shall be conducted at least semi-annually, including a check of communications with offsite support groups.

### B. Radiological

Radiation control procedures shall be maintained and made available to all plant personnel. These procedures shall show permissible radiation exposure and shall be consistent with the requirements of 10 CFR 20. This radiation protection program shall be organized to meet the requirements of 10 CFR 20.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NO. 50-263NORTHERN STATES POWER COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO PROVISIONAL  
OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 33 to Provisional Operating License No. DPR-22, issued to Northern States Power Company (the licensee), which revised the Technical Specifications for operation of the Monticello Nuclear Generating Plant (the facility) located in Wright County, Minnesota. The amendment is effective as of its date of issuance.

The amendment revised the existing Monticello Technical Specifications to incorporate fire protection specifications consistent with the Commission's requirements. These specifications are as suggested by the Commission with three exceptions taken by the licensee which the Commission's Staff has temporarily accepted pending its review and final resolution. The three exceptions relate to the reduction from five to three in the minimum number of on-site fire brigade members, valve position verification, and responsibility for training of fire protection personnel.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the

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license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated January 31, 1977 and supplement thereto dated December 16, 1977, (2) the Commission's letters dated September 30, 1976 and November 25, 1977, (3) Amendment No. 33 to License No. DPR-22, and (4) the Commission's related Safety Evaluation issued as an enclosure to the Commission's November 25, 1977 letter. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Environmental Conservation Library, Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401. A single copy of items (2), (3) and (4) above may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission,

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Washington, D. C., Attention: Director, Division of Operating  
Reactors.

Dated at Bethesda, Maryland, this 25th day of January, 1978.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Don K. Davis". The signature is written in a cursive style with a large initial "D" and "K".

Don K. Davis, Acting Chief  
Operating Reactors Branch #2  
Division of Operating Reactors