

October 23, 1998

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES AND OPPORTUNITY FOR A HEARING - BYRON STATION, UNITS 1 AND 2 AND BRAIDWOOD STATION, UNITS 1 AND 2 (TAC NOS. M97546, M97547, M97548 AND M97549)

Dear Mr. Kingsley:

Enclosed is a copy of a "Notice of Consideration of Issuance of Amendments to Facility Operating Licenses and Opportunity for a Hearing" for your information. This notice relates to your application for amendments dated December 13, 1996, as supplemented by letters dated February 24, September 2, October 10, October 28 and December 8, 1997 and January 27, January 29, February 6, February 13, February 24, February 26, April 13, April 16, June 1, June 2, July 2, July 8, July 30, July 31, August 11, August 12, September 21, September 25, October 1, October 2, October 5 and October 15, 1998. In your application, you propose to convert the current Technical Specifications (TSs) for Byron Station, Units 1 and 2 and Braidwood Station, Units 1 and 2, to improved Standard Technical Specifications (ISTS), consistent with the provisions of NUREG-1431, Revision 1, "Standard Technical Specifications - Westinghouse Plants," dated April 1995.

This notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

ORIG. SIGNED BY

Ramin R. Assa, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

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Docket Nos. STN 50-454, STN 50-455,
STN 50-456, STN 50-457

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Enclosure: Notice

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 23, 1998

Mr. Oliver D. Kingsley, President
Nuclear Generation Group
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO
FACILITY OPERATING LICENSES AND OPPORTUNITY FOR A HEARING -
BYRON STATION, UNITS 1 AND 2 AND BRAIDWOOD STATION, UNITS 1
AND 2 (TAC NOS. M97546, M97547, M97548 AND M97549)

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This notice has been forwarded to the Office of the Federal Register for publication.

Sincerely,

A handwritten signature in black ink, appearing to read "Ramin R. Assa".

Ramin R. Assa, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,
STN 50-456, STN 50-457

Enclosure: Notice

cc w/encl: See next page

O. Kingsley
Commonwealth Edison Company

cc:

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Byron Resident Inspectors Office
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Byron, Illinois 61010-9750

Regional Administrator, Region III
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Chairman, Ogle County Board
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Oregon, Illinois 61061

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Illinois Emergency Services
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Chairman
Will County Board of Supervisors
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Braidwood Station Manager
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O. Kingsley
Commonwealth Edison Company

- 2 -

Byron/Braidwood Stations

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Mr. Michael J. Wallace
Senior Vice President
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1400 Opus Place, Suite 900
Downers Grove, IL 60515

UNITED STATES NUCLEAR REGULATORY COMMISSIONCOMMONWEALTH EDISON COMPANYDOCKET NOS. STN 50-454, STN 50-455, STN 50-456, STN 50-457NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENTS TO
FACILITY OPERATING LICENSES AND OPPORTUNITY FOR A HEARING

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. NPF-37 and NPF-66, issued to Commonwealth Edison Company (ComEd, the licensee) for operation of Byron Station, Units 1 and 2, located in Ogle County, Illinois and Facility Operating License Nos. NPF-72 and NPF-77, issued to ComEd for operation of Braidwood Station, Units 1 and 2, located in Will County, Illinois.

The proposed amendments present a full conversion from the current Technical Specifications (TS) to TS based on NUREG-1431, Revision 1, "Standard Technical Specifications - Westinghouse Plants," dated April 1995. NUREG-1431 has been developed through working groups composed of both NRC staff members and industry representatives and has been endorsed by the staff as part of an industry-wide initiative to standardize and improve TS. The December 13, 1996, application was supplemented by letters dated February 24, September 2, October 10, October 28 and December 8, 1997 and January 27, January 29, February 6, February 13, February 24, February 26, April 13, April 16, June 1, June 2, July 2, July 8, July 30, July 31, August 11, August 12, September 21, September 25, October 1, October 2, October 5 and October 15, 1998. As part of this submittal, ComEd has applied the criteria contained in the Commission's "Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors (final policy statement)," published in the Federal Register on July 22, 1993 (58 FR 39132), to the current Byron and Braidwood TSs

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and using NUREG-1431 as a basis, developed a proposed set of improved TSs for Byron and Braidwood. The criteria in the final policy statement were subsequently added to 10 CFR 50.36, "Technical Specifications," in a rule change which was published in the Federal Register on July 19, 1995 (60 FR 36953) and became effective on August 18, 1995.

The licensee has categorized the proposed changes to the existing TSs into five general groupings. These groupings are characterized as administrative changes, relocated changes, more restrictive changes, less restrictive changes, and removed detail.

Administrative changes are those that involve restructuring, renumbering, rewording, interpretation and complex rearranging of requirements and other changes not affecting technical content or substantially revising an operational requirement. The reformatting, renumbering and rewording process reflects the attributes of NUREG-1431 and do not involve technical changes to the existing TSs. The proposed changes include: (a) providing the appropriate numbers, etc., for NUREG-1431 bracketed information (information which must be supplied on a plant-specific basis, and which may change from plant to plant), (b) identifying plant-specific wording for system names, etc., and (c) changing NUREG-1431 section wording to conform to existing licensee practices. Such changes are administrative in nature and do not impact initiators of analyzed events or assumed mitigation of accident or transient events.

More restrictive changes are those involving more stringent requirements for operation of the facility or eliminate existing flexibility. These more stringent requirements do not result in operation that will alter assumptions relative to mitigation of an accident or transient event. The more restrictive requirements will not alter the operation of process variables, structures, systems and components described in the safety analyses. For each requirement in the current Byron and Braidwood TSs that is more restrictive than the corresponding requirement in

NUREG-1431 which the licensee proposes to retain in the improved Technical Specifications (ITSs), they have provided an explanation of why they have concluded that retaining the more restrictive requirement is desirable to ensure safe operation of the facilities because of specific design features of the plant.

Less restrictive changes are those where current requirements are relaxed or eliminated, or new flexibility is provided. The more significant "less restrictive" requirements are justified on a case-by-case basis. When requirements have been shown to provide little or no safety benefit, their removal from the TSs may be appropriate. In most cases, relaxations previously granted to individual plants on a plant-specific basis were the result of (a) generic NRC actions, (b) new NRC staff positions that have evolved from technological advancements and operating experience, or (c) resolution of the Owners Groups' comments on the improved Standard Technical Specifications (ISTS). Generic relaxations contained in NUREG-1431 were reviewed by the staff and found to be acceptable because they are consistent with current licensing practices and NRC regulations. The licensee's design will be reviewed to determine if the specific design basis and licensing basis are consistent with the technical basis for the model requirements in NUREG-1431 and, thus, provides a basis for these revised TSs or if relaxation of the requirements in the current TSs is warranted based on the justification provided by the licensee.

Some less restrictive changes involve removal of detail from the current TS to a licensee-controlled document. The details being removed from the current TS are not assumed to be an initiator of any analyzed event and are not assumed to mitigate accidents or transients. Therefore, the relocation does not involve a significant increase in the probability or consequences of an accident previously evaluated. Moving some details to a

licensee-controlled document will not involve a significant change in design or operation of the plant and no hardware is being added to the plant as part of the proposed changes to the current TS. The changes will not alter assumptions made in the safety analysis and licensing basis. Therefore, the changes will not create the possibility of a new or different kind of accident from any accident previously evaluated. The changes do not reduce the margin of safety since they have no impact on any safety analysis assumptions.

Some less restrictive changes involve the relocation of entire specifications, which contain surveillance requirements for structures, systems, components or variables that do not meet the criteria for inclusion in the TSs. Relocated changes are those current TS requirements which do not satisfy or fall within any of the four criteria specified in the Commission's policy statement and may be relocated to appropriate licensee-controlled documents.

The licensee's application of the screening criteria is described in that portion of their December 13, 1996, application titled, "Application of Selection Criteria to the Byron/Braidwood Technical Specifications." The affected structures, systems components or variables are not assumed to be initiators of analyzed events and are not assumed to mitigate accident or transient events. The requirements and surveillances for these affected structures, systems, components or variables will be relocated from the TS to administratively controlled documents such as the Updated Final Safety Analysis Report (UFSAR), the TS Bases, the Technical Requirements Manual (TRM), the Selected Licensee Commitments or plant procedures. Changes made to these documents will be made pursuant to 10 CFR 50.59 or other appropriate control mechanisms. In addition, the affected structures, systems, components or

variables are addressed in existing surveillance procedures which are also subject to 10 CFR 50.59. These proposed changes will not impose or eliminate any requirements.

Before issuance of the proposed license amendments, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

By November 30, 1998, the licensee may file a request for a hearing with respect to issuance of the amendments to the subject facility operating licenses and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the: for Byron, the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010; for Braidwood, the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be

affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendments under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which

satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemaking and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60603, attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).

If a request for a hearing is received, the Commission's staff may issue the amendment after it completes its technical review and prior to the completion of any required hearing if it publishes a further notice for public comment of its proposed finding of no significant hazards consideration in accordance with 10 CFR 50.91 and 50.92.

For further details with respect to this action, see the application for amendments dated December 13, 1996, as supplemented by letters dated February 24, September 2, October 10, October 28 and December 8, 1997 and January 27, January 29, February 6, February 13, February 24, February 26, April 13, April 16, June 1, June 2, July 2, July 8, July 30, July 31, August 11, August 12, September 21, September 25, October 1, October 2, October 5 and October 15, 1998, which are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street NW., Washington, DC. and at the local public document room located at: for Byron, at the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010; and for Braidwood, at the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481.

Dated at Rockville, Maryland, this 23^d day of October 1998.

FOR THE NUCLEAR REGULATORY COMMISSION



Ramin R. Assa, Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation