

December 12, 1997

Ms. Irene Johnson, Acting Manager  
Nuclear Regulatory Services  
Commonwealth Edison Company  
Executive Towers West III  
1400 Opus Place, Suite 500  
Downers Grove, IL 60515

SUBJECT: CONTAINMENT VESSEL STRUCTURAL INTEGRITY - BRAIDWOOD  
STATION, UNITS 1 AND 2; AND BYRON STATION, UNITS 1 AND 2 (TAC NOS.  
M99174, M99175, M99176 AND M99177)

Dear Ms. Johnson:

The Commission has forwarded the enclosed "Notice of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination and Opportunity for a Hearing" to the Office of the Federal Register for publication.

This notice relates to your submittal of June 17, 1997, regarding Commonwealth Edison Company's request to amend the Technical Specifications (TS) related to "Containment Vessel Structural Integrity," for Byron Nuclear Power Station, Units 1 and 2, and Braidwood Nuclear Power Station, Units 1 and 2, to incorporate the requirements of 10 CFR 50.55a(b)(2)(vi) and 10 CFR 50.55a(b)(2)(ix) and to update your existing Containment Vessel Structural Integrity Programs to meet the requirements found in Subsection IWL of the 1992 Edition, 1992 Addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) Section XI. The proposed submittal also incorporates Regulatory Guide 1.35.1, 1990, "Determination Prestressing Forces for Inspection of Prestressed Concrete Containmentment."

Sincerely,

Orig. signed by

George F. Dick, Jr., Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457,  
STN 50-454, STN 50-455

Enclosure: Notice



cc w/encl: see next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

December 12, 1997

Ms. Irene Johnson, Acting Manager  
Nuclear Regulatory Services  
Commonwealth Edison Company  
Executive Towers West III  
1400 Opus Place, Suite 500  
Downers Grove, IL 60515

SUBJECT: CONTAINMENT VESSEL STRUCTURAL INTEGRITY - BRAIDWOOD STATION,  
UNITS 1 AND 2; AND BYRON STATION, UNITS 1 AND 2 (TAC NOS. M99174,  
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Sincerely,

A handwritten signature in cursive script that reads "George F. Dick, Jr.".

George F. Dick, Jr., Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-456, STN 50-457,  
STN 50-454, STN 50-455

Enclosure: Notice

cc w/encl: see next page

**I. Johnson  
Commonwealth Edison Company**

**cc:**

**Mr. William P. Poirier, Director  
Westinghouse Electric Corporation  
Energy Systems Business Unit  
Post Office Box 355, Bay 236 West  
Pittsburgh, Pennsylvania 15230**

**Joseph Gallo  
Gallo & Ross  
1250 Eye St., N.W., Suite 302  
Washington, DC 20005**

**Michael I. Miller, Esquire  
Sidley and Austin  
One First National Plaza  
Chicago, Illinois 60603**

**Howard A. Learner  
Environmental law and Policy  
Center of the Midwest  
203 North LaSalle Street  
Suite 1390  
Chicago, Illinois 60601**

**U.S. Nuclear Regulatory Commission  
Byron Resident Inspectors Office  
4448 North German Church Road  
Byron, Illinois 61010-9750**

**Regional Administrator, Region III  
U.S. Nuclear Regulatory Commission  
801 Warrenville Road  
Lisle, Illinois 60532-4351**

**Ms. Lorraine Creek  
Rt. 1, Box 182  
Manteno, Illinois 60950**

**Chairman, Ogle County Board  
Post Office Box 357  
Oregon, Illinois 61061**

**Mrs. Phillip B. Johnson  
1907 Stratford Lane  
Rockford, Illinois 61107**

**Byron/Braidwood Power Stations**

**George L. Edgar  
Morgan, Lewis and Bochiuss  
1800 M Street, N.W.  
Washington, DC 20036**

**Attorney General  
500 South Second Street  
Springfield, Illinois 62701**

**Illinois Department of Nuclear Safety  
Office of Nuclear Facility Safety  
1035 Outer Park Drive  
Springfield, Illinois 62704**

**Commonwealth Edison Company  
Byron Station Manager  
4450 North German Church Road  
Byron, Illinois 61010**

**Commonwealth Edison Company  
Site Vice President - Byron  
4450 N. German Church Road  
Byron, Illinois 61010**

**U.S. Nuclear Regulatory Commission  
Braidwood Resident Inspectors Office  
Rural Route #1, Box 79  
Braceville, Illinois 60407**

**Mr. Ron Stephens  
Illinois Emergency Services  
and Disaster Agency  
110 East Adams Street  
Springfield, Illinois 62706**

**Chairman  
Will County Board of Supervisors  
Will County Board Courthouse  
Joliet, Illinois 60434**

**Commonwealth Edison Company  
Braidwood Station Manager  
Rt. 1, Box 84  
Braceville, Illinois 60407**

**Ms. Bridget Little Rorem  
Appleseed Coordinator  
117 North Linden Street  
Essex, Illinois 60935**

**Document Control Desk-Licensing  
Commonwealth Edison Company  
1400 Opus Place, Suite 400  
Downers Grove, Illinois 60515**

**Commonwealth Edison Company  
Site Vice President - Braidwood  
RR 1, Box 84  
Braceville, IL 60407**

**UNITED STATES NUCLEAR REGULATORY COMMISSION****COMMONWEALTH EDISON COMPANY****DOCKET NOS. STN 50-454, STN 50-455, STN 50-456 AND STN 50-457****NOTICE OF CONSIDERATION OF ISSUANCE OF AMENDMENT TO  
FACILITY OPERATING LICENSE, PROPOSED NO SIGNIFICANT HAZARDS  
CONSIDERATION DETERMINATION, AND OPPORTUNITY FOR A HEARING**

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License Nos. NPF-37, NPF-66, NPF-72 and NPF-77 issued to Commonwealth Edison Company (the licensee) for operation of Byron Station, Units 1 and 2, located in Ogle County, Illinois and Braidwood Station, Units 1 and 2, located in Will County, Illinois.

The amendment would amend the Technical Specifications (TS) related to "Containment Vessel Structural Integrity," to incorporate the requirements of 10 CFR 50.55a(b)(2)(vi) and 10 CFR 50.55a(b)(2)(ix) and update the existing Containment Vessel Structural Integrity Programs to meet the requirements found in Subsection IWL of the 1992 Edition, 1992 Addenda of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code) Section XI. The proposed amendment would also incorporate Regulatory Guide 1.35.1, 1990, "Determination Prestressing Forces for Inspection of Prestressed Concrete Containment."

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment

would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes revise the surveillance requirements for containment reinforced concrete and unbonded post-tensioning systems inservice examinations as required by 10 CFR 50.55a(b)(2)(vi) and 10 CFR 50.55a(b)(2)(ix). The revised requirements affect the inservice inspection program designed to detect structural degradation of the containment reinforced concrete and unbonded post-tensioning systems program and do not affect the function of the containment reinforced concrete and the unbonded post-tensioning system components. The reinforced concrete and the unbonded post-tensioning system are passive components whose failure modes could not act as accident initiators or precursors.

The proposed changes do not impact any accident initiators or analyzed events or assumed mitigation of accident or transient events. They do not involve the addition or removal of any equipment , or any design changes to the facility. Therefore, this proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes do not involve a modification to the physical configuration of the plant (i.e., no new equipment will be installed) or change in the methods governing normal plant operation. The proposed changes will not impose any new or different requirements or introduce a new accident initiator or precursor or malfunction mechanism. The proposed changes provide an NRC-approved ASME Code inspection/testing methodology to assure age-related degradation of the containment structure will not go undetected. The function of the containment reinforced concrete and the unbonded post-tentionining system components are not altered by this change. Additionally, there is no change in the types or increase in the amounts of any effluent that may be released offsite; and there is no increase in individual or cumulative occupational radiation exposure. Therefore, the possibility of a new or different kind of accident from any previously evaluated has not been created.

3. **The proposed change does not involve a significant reduction in a margin of safety.**

**The proposed changes revise the surveillance requirements for containment reinforced concrete and unbonded post-tensioning systems inservice examinations and tests contained in the referenced TS as required by 10 CFR 50.55a(b)(2)(vi) and 10 CFR 50.55a(b)(2)(ix). The proposed changes do not affect the ability of containment to mitigate design basis accidents, and, therefore, do not result in a reduction in the margin of safety.**

**The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.**

**The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.**

**Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission take this action, it will publish in the FEDERAL REGISTER a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.**

**Written comments may be submitted by mail to the Chief, Rules and Directives Branch, Division of Administrative Services, Office of Administration, U.S. Nuclear Regulatory**

Commission, Washington, DC 20555-0001, and should cite the publication date and page number of this FEDERAL REGISTER notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC. The filing of requests for hearing and petitions for leave to intervene is discussed below.

By January 20, 1997, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at: for Byron, located at the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010; for Braidwood, the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.

As required by 10 CFR 2.714, a petition for leave to intervene shall set forth with particularity the interest of the petitioner in the proceeding, and how that interest may be



affected by the results of the proceeding. The petition should specifically explain the reasons why intervention should be permitted with particular reference to the following factors: (1) the nature of the petitioner's right under the Act to be made party to the proceeding; (2) the nature and extent of the petitioner's property, financial, or other interest in the proceeding; and (3) the possible effect of any order which may be entered in the proceeding on the petitioner's interest. The petition should also identify the specific aspect(s) of the subject matter of the proceeding as to which petitioner wishes to intervene. Any person who has filed a petition for leave to intervene or who has been admitted as a party may amend the petition without requesting leave of the Board up to 15 days prior to the first prehearing conference scheduled in the proceeding, but such an amended petition must satisfy the specificity requirements described above.

Not later than 15 days prior to the first prehearing conference scheduled in the proceeding, a petitioner shall file a supplement to the petition to intervene which must include a list of the contentions which are sought to be litigated in the matter. Each contention must consist of a specific statement of the issue of law or fact to be raised or controverted. In addition, the petitioner shall provide a brief explanation of the bases of the contention and a concise statement of the alleged facts or expert opinion which support the contention and on which the petitioner intends to rely in proving the contention at the hearing. The petitioner must also provide references to those specific sources and documents of which the petitioner is aware and on which the petitioner intends to rely to establish those facts or expert opinion. Petitioner must provide sufficient information to show that a genuine dispute exists with the applicant on a material issue of law or fact. Contentions shall be limited to matters within the scope of the amendment under consideration. The contention must be one which, if proven, would entitle the petitioner to relief. A petitioner who fails to file such a supplement which

satisfies these requirements with respect to at least one contention will not be permitted to participate as a party.

Those permitted to intervene become parties to the proceeding, subject to any limitations in the order granting leave to intervene, and have the opportunity to participate fully in the conduct of the hearing, including the opportunity to present evidence and cross-examine witnesses.

If a hearing is requested, the Commission will make a final determination on the issue of no significant hazards consideration. The final determination will serve to decide when the hearing is held.

If the final determination is that the amendment request involves no significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

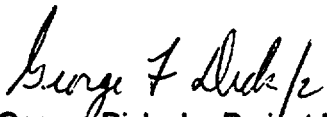
A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, Attention: Rulemakings and Adjudications Staff, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date. A copy of the petition should also be sent to the Office of the General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to Michael I. Miller, Esquire; Sidley and Austin, One First National Plaza, Chicago, Illinois 60603, attorney for the licensee.

**Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for hearing will not be entertained absent a determination by the Commission, the presiding officer or the presiding Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of the factors specified in 10 CFR 2.714(a)(1)(i)-(v) and 2.714(d).**

**For further details with respect to this action, see the application for amendment dated June 17, 1997, which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document rooms: for Byron, located at the Byron Public Library District, 109 N. Franklin, P.O. Box 434, Byron, Illinois 61010; for Braidwood, the Wilmington Public Library, 201 S. Kankakee Street, Wilmington, Illinois 60481.**

**Dated at Rockville, Maryland, this 12 day of December, 1997.**

**FOR THE NUCLEAR REGULATORY COMMISSION**

  
**George Dick, Jr., Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation**