

February 12, 1997

Ms. Irene M. Johnson, ~~ing~~ Manager  
Nuclear Regulatory Services  
Commonwealth Edison Company  
Executive Towers West III  
1400 Opus Place, Suite 500  
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M96427, M96428, M96429 AND M96430)

Dear Ms. Johnson:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 85 to Facility Operating License No. NPF-37 and Amendment No. 85 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 77 to Facility Operating License No. NPF-72 and Amendment No. 77 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated August 2, 1996.

The amendments eliminate License Condition 2.C.(16) from Facility Operating License NPF-37; License Condition 2.C.(5) from Facility Operating License NPF-66; License Condition 2.C.(6) from Facility Operating License NPF-72 and License Condition 2.C.(5) from Facility Operating License NPF-77 that require the licensee to conduct additional corrosion testing of sleeved steam generator tubes.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

George F. Dick, Jr., Senior Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,  
STN 50-456 and STN 50-457

- Enclosures:
1. Amendment No. 85 to NPF-37
  2. Amendment No. 85 to NPF-66
  3. Amendment No. 77 to NPF-72
  4. Amendment No. 77 to NPF-77
  5. Safety Evaluation

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\*concurrence provided by SE dated 12/5/96  
\*\*see previous concurrence

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P PDR

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

February 12, 1997

Ms. Irene M. Johnson, Acting Manager  
Nuclear Regulatory Services  
Commonwealth Edison Company  
Executive Towers West III  
1400 Opus Place, Suite 500  
Downers Grove, IL 60515

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M96427, M96428, M96429 AND  
M96430)

Dear Ms. Johnson:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 85 to Facility Operating License No. NPF-37 and Amendment No. 85 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 77 to Facility Operating License No. NPF-72 and Amendment No. 77 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated August 2, 1996.

The amendments eliminate License Condition 2.C.(16) from Facility Operating License NPF-37; License Condition 2.C.(5) from Facility Operating License NPF-66; License Condition 2.C.(6) from Facility Operating License NPF-72 and License Condition 2.C.(5) from Facility Operating License NPF-77 that require the licensee to conduct additional corrosion testing of sleeved steam generator tubes.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "George F. Dick, Jr.".

George F. Dick, Jr., Senior Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,  
STN 50-456 and STN 50-457

Enclosures: 1. Amendment No. 85 to NPF-37  
2. Amendment No. 85 to NPF-66  
3. Amendment No. 77 to NPF-72  
4. Amendment No. 77 to NPF-77  
5. Safety Evaluation

cc w/encl: see next page

I. Johnson  
Commonwealth Edison Company

cc:

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Byron/Braidwood Power Stations

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Office of Nuclear Facility Safety  
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Braidwood Resident Inspectors Office  
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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85  
License No. NPF-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 2, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to paragraph 2.C.(2) and by deleting paragraph 2.C.(16)\* of Facility Operating License No. NPF-37 and is hereby amended to read as follows:

---

\*Page 7 is attached, for convenience, for the composite license to reflect this change.

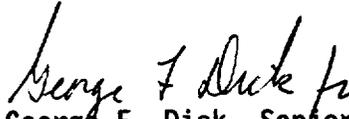
(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 85 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(16) Deleted.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



George F. Dick, Senior Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment:  
License page 7

Date of Issuance: February 12, 1997

(16) Deleted.

- D. The facility requires exemptions from certain requirements of Appendices A, E and J to 10 CFR Part 50. These include (a) an exemption from the requirements of Paragraph III.D.2(b)(ii) of Appendix J, the testing of containment air locks at times when containment integrity is not required (Section 6.2.6 of the SER), (b) an exemption from GDC-2 of Appendix A, the requirement that structures, systems and components important to safety be designed to withstand the effects of natural phenomena such as earthquakes (Section 3.10 of SSER #5), (c) an exemption from GDC-13 and GDC-17 of Appendix A, the requirement that instrumentation be provided to monitor variables and systems over their anticipated ranges, and the requirement that provisions be included to minimize the probability of losing electric power (Section 9.5.4.1 of SSER #5), (d) an exemption from GDC-19 of Appendix A, the requirement that the control room have adequate radiation protection to permit access and occupancy under accident conditions (Section 6.5.1 of SSER #6), and (e) an exemption from the requirement of Section IV.F of Appendix E that a full participation emergency planning exercise be conducted within one year before issuance of the first operating license for full power and prior to operation above 5% of rated power (Section 13.3 of SSER #6). These exemptions are authorized by law and will not endanger life or property or the



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 85  
License No. NPF-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 2, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to paragraph 2.C.(2) and by deleting paragraph 2.C.(5)\* of Facility Operating License No. NPF-66 and is hereby amended to read as follows:

---

\*Page 3a is attached, for convenience, for the composite license to reflect this change.

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 85 and revised by Attachment 2 to NPF-66, and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, dated February 14, 1985, are hereby incorporated into this license. Attachment 2 contains a revision to Appendix A which is hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(5) Deleted.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



George F. Dick Jr., Senior Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment:  
License page 3a

Date of Issuance: February 12, 1997

(3) Initial Test Program

Any changes to the Initial Startup Test Program described in Chapter 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(4) Regulatory Guide 1.97, Revision 2 Compliance

The licensee shall submit by March 1, 1987, a preliminary report describing how the requirements of Regulatory Guide 1.97, Revision 2 have been or will be met. The licensee shall submit by September 1, 1987, the final report and a schedule for implementation (assuming the NRC approves the DCRDR by March 1, 1987).

(5) Deleted.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77  
License No. NPF-72

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 2, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to paragraph 2.C.(2) and by deleting paragraph 2.C.(6)\* of Facility Operating License No. NPF-72 and is hereby amended to read as follows:

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\*Page 4 is attached, for convenience, for the composite license to reflect this change.

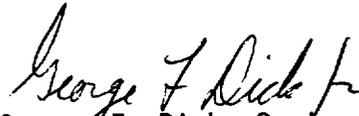
(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 77 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(6) Deleted.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



George F. Dick, Senior Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment:  
License page 4

Date of Issuance: February 12, 1997

(4) Initial Startup Test Program

Any changes to the Initial Test Program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(5) Regulatory Guide 1.97, Revision 2 Compliance

The licensee shall submit the final report and a schedule for implementation within six months of NRC approval of the DCRDR.

(6) Deleted.

- D. The facility requires an exemption from the requirements of Appendix J to 10 CFR Part 50, Paragraph III.D.2(b)(ii), the testing of containment air locks at times when containment integrity is not required (SER Section 6.2.6). This exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. This exemption is hereby granted. The special circumstances regarding this exemption are identified in the referenced section of the safety evaluation report and the supplements thereto. This exemption is granted pursuant to 10 CFR 50.12. With this exemption, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

An exemption was previously granted pursuant to 10 CFR 70.24. The exemption was granted with NRC materials license No. SNM-1938, issued October 8, 1985, and relieved the licensee from the requirement of having a criticality alarm system. Therefore, the licensee is exempted from the criticality alarm system provision of 10 CFR 70.24 so far as this section applies to the storage of fuel assemblies held under this license.

- E. The licensee shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report, as supplemented and amended, and as approved in the SER dated November 1983 and its supplements, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission, only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 77  
License No. NPF-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 2, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to paragraph 2.C.(2) and by deleting paragraph 2.C.(5)\* of Facility Operating License No. NPF-77 and is hereby amended to read as follows:

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\*Page 4 is attached, for convenience, for the composite license to reflect this change.

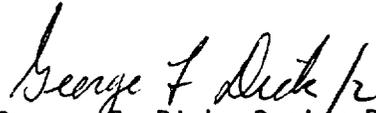
(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 77 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(5) Deleted.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



George F. Dick, Senior Project Manager  
Project Directorate III-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Attachment:  
License page 4

Date of Issuance: February 12, 1997

(4) Initial Startup Test Program

Any changes to the Initial Test Program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(5) Deleted.

- D. The facility requires an exemption from the requirements of Appendix J to 10 CFR Part 50, Paragraph III.D.2(b)(ii), the testing of containment air locks at times when containment integrity is not required (SER Section 6.2.6). This exemption is authorized by law, will not present an undue risk to the public health and safety, and is consistent with the common defense and security. The staff's environmental assessment was published on May 19, 1988 (53 FR 17995). This exemption was granted in the low power license and is continued for the full power license. The special circumstances regarding this exemption are identified in the referenced section of the Safety Evaluation Report and the supplements thereto. This exemption is granted pursuant to 10 CFR 50.12. With this exemption, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

An exemption was previously granted pursuant to 10 CFR 70.24. The exemption was granted with NRC materials license No. SNM-1938, issued October 8, 1985, and relieved the licensee from the requirement of having a criticality alarm system. Therefore, the licensee is exempted from the criticality alarm system provision of 10 CFR 70.24 so far as this section applies to the storage of fuel assemblies held under this license.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-37,  
AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. NPF-66,  
AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. NPF-72,  
AND AMENDMENT NO. 77 TO FACILITY OPERATING LICENSE NO. NPF-77  
COMMONWEALTH EDISON COMPANY  
BYRON STATION, UNIT NOS. 1 AND 2  
BRAIDWOOD STATION, UNIT NOS. 1 AND 2  
DOCKET NOS. STN 50-454, STN 50-455, STN 50-456 AND STN 50-457

1.0 INTRODUCTION

By letter dated August 2, 1996, Commonwealth Edison Company (ComEd, the licensee), submitted a request to amend the Facility Operating Licenses at Byron, Units 1 and 2, and Braidwood Stations, Units 1 and 2. The proposed amendment would eliminate a license condition which requires the licensee to conduct additional corrosion testing of sleeved steam generator tubes. Specifically, the amendments would eliminate License Condition 2.C.(16) from Facility Operating License NPF-37; License Condition 2.C.(5) from Facility Operating License NPF-66; License Condition 2.C.(6) from Facility Operating License NPF-72 and License Condition 2.C.(5) from Facility Operating License NPF-77.

Presently, the license conditions require additional corrosion testing to be conducted to establish the design life of laser welded sleeves in the presence of a crevice. The purpose of the testing is to determine the effects that material microstructure, chemistry, and joint crevices will have on primary water stress corrosion cracking (PWSCC) initiation and growth. The testing is to bound the material condition that exists in Byron and Braidwood steam generators, and include the associated stress intensity values. The testing results are to be reviewed and accepted by the NRC prior to the beginning of Byron Unit 1 Cycle 9, Byron Unit 2 Cycle 8 and Braidwood Cycle 7 for Units 1 and 2.

Until this testing is satisfactorily completed, the licensee also committed to performing an augmented inspection (increased sample size) of the installed sleeves of each unit. This requirement would also be deleted following corrosion testing. However, this requirement was subsequently revised in Amendment Nos. 83 (Byron) and 76 (Braidwood) which approved sleeving by the ABB/CE Tungsten Inert Gas (TIG) welded sleeving process. The current

Technical Specifications (TS) (e.g. TS 4.4.5.2. (for Byron and Braidwood)) require the licensee to perform an inservice inspection of a random sample of a minimum of 20 percent of the installed sleeves, regardless of type. This sampling plan is in accordance with present industry guidelines and will be continued regardless of the deletion of the license condition on corrosion testing.

## 2.0 EVALUATION

On March 4, 1994, the staff issued license amendments 58 (Byron) and 46 (Braidwood) which revised the technical specifications to permit sleeving of steam generator tubes using either Westinghouse laser-welded or B&W kinetically welded sleeves. In the accompanying Safety Evaluation, the staff restated certain commitments made by the licensee that when implemented, would provide assurance that performance of the sleeves would be carefully monitored. The commitments were: (1) amend the licenses to reflect a primary-to-secondary leakage limit of 150 gallons per day; (2) amend the licenses to reflect a more conservative plugging limit in the inservice inspection program; and (3) add a condition to the licenses to conduct additional corrosion testing. On June 3, 1994, the licensee requested amendments to the licenses to incorporate the three commitments; they were approved on January 6, 1995, in amendments 67 and 57 for Byron and Braidwood, respectively. In requiring that accelerated corrosion testing be done, the staff believed that the service life prediction would apply to those components susceptible to stress corrosion cracking (SCC). The service life would be predicted as a result of accelerated corrosion tests. These tests attempted to define a correlation between vendor proprietary laboratory tests of alloy 600 components and in-plant service life (time to cracking in components susceptible to SCC). The resulting deterministic service life predictions would then be used to establish service lifetimes and reinspection intervals for sleeved tubes or other components.

As more information has become available, the staff has concluded that the information obtained from the accelerated corrosion testing is inconclusive in establishing a reliable prediction of steam generator sleeve service life. Consequently, the staff has since rejected this approach and continues to mandate periodic reinspections. The staff now finds the industry developed sampling plan (EPRI NP-6201, "PWR Steam Generator Tube Examination Guidelines," Appendix H, "Performance Demonstration for Eddy Current Examinations") acceptable. The reasons for discontinuing the life prediction approach included unresolved issues such as data scatter and poor predictive results when the method was applied to a certain type of tube plugs.

In previous evaluations of accelerated corrosion test based service life algorithms, the stated staff finding (and current staff conclusion) has been that:

".....accelerated corrosion tests provide a good qualitative assessment of relative service life for various sleeving processes. However, quantitative estimates of service life do not have a high degree of

reliability. For this reason, the staff does not accept deterministic life prediction calculations for tube materials. Periodic inspection (as required by the Technical Specifications) and primary-to-secondary leakage monitoring will identify any premature degradation that may occur in the sleeved joints."

This, or wording similar thereto, may be found in TS amendment Safety Evaluations for welded sleeve installations at several plants including: Waterford Steam Electric Station, Unit 3, Byron and Braidwood Stations (amendments 83 and 76, respectively), and others.

Furthermore, in other welded tube sleeve safety evaluations, the staff has noted previously that over 10 years of operating experience with welded sleeves (both laser welded and TIG welded) has shown the technology to be highly reliable. No operationally induced degradation or leakage has occurred in any welded sleeve, with over 25,000 installed to date. Three or four instances of installation defects have occurred that resulted in inconsequential leaks. All were removed from service.

### 3.0 SUMMARY

The staff finds that the subject license conditions for the respective units comprising the Byron and Braidwood Stations are no longer applicable and may be deleted from the licensee's licenses as requested. No additional corrosion testing of welded tube sleeves as specified under the terms of the subject license conditions is required or warranted.

### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 50340). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Hornseth

Date: February 12, 1997