

April 16, 1996

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, Illinois 60515

SUBJECT: CORRECTION TO AMENDMENTS (TAC NOS. M94212, M94213, M94214 AND M94215)

Dear Mr. Farrar:

On April 4, 1996, the U.S. Nuclear Regulatory Commission issued Amendment Nos. 81 and 81 to Facility Operating License Nos. NPF-37 and NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment Nos. 73 and 73 to Facility Operating License Nos. NPF-77 and NPF-72 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments were in response to your application dated December 6, 1995, as supplemented on February 27, 1996 and March 28, 1996.

Inaccuracies have been discovered on Technical Specification pages 1-3 and 3/4 6-12, and in the Notice of Partial Denial of Amendment to Facility Operating Licenses and Opportunity for a Hearing. Enclosed please find a copy of the corrected pages. We apologize for any inconvenience that these errors may have caused.

Sincerely,

Original signed by:

George F. Dick, Jr., Sr. Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455
STN 50-456, STN 50-457

Enclosures: As stated

cc w/encls: See next page

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DEFINITIONS

E - AVERAGE DISINTEGRATION ENERGY

1.12 E shall be the average (weighted in proportion to the concentration of each radionuclide in the sample) of the sum of the average beta and gamma energies per disintegration (MeV/d) for the radionuclides in the sample.

ENGINEERED SAFETY FEATURES RESPONSE TIME

1.13 The ENGINEERED SAFETY FEATURES (ESF) RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its ESF Actuation Setpoint at the channel sensor until the ESF equipment is capable of performing its safety function (i.e., the valves travel to their required positions, pump discharge pressures reach their required values, etc.). Times shall include diesel generator starting and sequence loading delays where applicable.

FREQUENCY NOTATION

1.14 The FREQUENCY NOTATION specified for the performance of Surveillance Requirements shall correspond to the intervals defined in Table 1.1.

IDENTIFIED LEAKAGE

1.15 IDENTIFIED LEAKAGE shall be:

- a. Leakage (except CONTROLLED LEAKAGE) into closed systems, such as pump seal or valve packing leaks that are captured and conducted to a sump or collecting tank, or
- b. Leakage into the containment atmosphere from sources that are both specifically located and known either not to interfere with the operation of Leakage Detection Systems or not to be PRESSURE BOUNDARY LEAKAGE, or
- c. Reactor Coolant System leakage through a steam generator to the Secondary Coolant System.

L_a

1.15.a The maximum allowable primary containment leakage rate, L_a, shall be 0.10% of the primary containment air weight per day at the calculated peak containment pressure (Pa).

MASTER RELAY TEST

1.16 A MASTER RELAY TEST shall be the energization of each master relay and verification of OPERABILITY of each relay. The MASTER RELAY TEST shall include a continuity check of each associated slave relay.

MEMBER(S) OF THE PUBLIC

1.17 MEMBER(S) OF THE PUBLIC shall include all persons who are not occupationally associated with the plant. This category does not include employees of the licensee, its contractors or vendors and persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational, or other purposes not associated with the plant.

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.6.1.7.1 Each 48-inch containment purge supply and exhaust isolation valve(s) shall be verified closed and power removed at least once per 31 days.

4.6.1.7.2 Each 8-inch containment purge supply and exhaust isolation valve shall be verified to be positioned in accordance with Specification 3.6.1.7b at least once per 31 days.

4.6.1.7.3 At least once per 6 months on a STAGGERED TEST BASIS, the inboard and outboard valves with resilient material seals in each closed 48-inch containment purge supply and exhaust penetration shall be demonstrated OPERABLE by verifying that the measured leakage rate is less than $0.05 L_a$ when pressurized to at least P_a , 44.4 psig.

4.6.1.7.4 At least once per 3 months, each 8-inch containment purge supply and exhaust isolation valve with resilient material seals shall be demonstrated OPERABLE by verifying that the measured leakage rate is less than $0.01 L_a$ when pressurized to at least P_a , 44.4 psig.

DEFINITIONS

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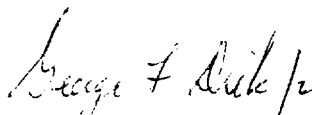
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CORRECTION TO NOTICE OF PARTIAL DENIAL
OF AMENDMENT TO FACILITY OPERATING
LICENSES AND OPPORTUNITY FOR A HEARING

On April 10, 1996, the FEDERAL REGISTER published a Notice of Partial Denial of Amendment to Facility Operating Licenses and Opportunity for a Hearing. On page 15983, under Commonwealth Edison Company, Docket Nos. STN 50-454, STN 50-455, STN 50-456 and STN 50-457, first paragraph, the Facility Operating License Nos. should have read NPF-37, NPF-66, NPF-72 and NPF-77.

Dated at Rockville, Maryland, this 16th day of April 1996.

FOR THE NUCLEAR REGULATORY COMMISSION



George F. Dick, Jr., Sr. Project Manager
Project Directorate III-2
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation