

January 19, 1995

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, Illinois 60515

SUBJECT: CORRECTION OF AMENDMENT (TAC NOS. M90575 AND M90576)

Dear Mr. Farrar:

On October 21, 1994, the U.S. Nuclear Regulatory Commission issued Amendment No. 65 to Facility Operating License No. NPF-37 and Amendment No. 65 to Facility License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively.

Revised Technical Specification Basis page B 3/4 1-3 was incomplete. Please replace the previously issued page with that enclosed.

We regret any inconvenience this error may have created.

Sincerely,

original signed by:

George F. Dick, Jr., Project Manager
Project Directorate III-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455

Enclosure: Corrected page B 3/4 1-3

cc w/encl: see next page

DISTRIBUTION:

Docket File	PUBLIC
PDIII-2 R/F	J. Roe
R. Capra	C. Moore
G. Dick	OGC
D. Hagan	G. Hill (4)
L. Miller, RIII	C. Grimes
ACRS (4)	OPA
OC/LFDCB	H. Richings

DOCUMENT NAME: BY90575.LTR

NRC FILE CENTER COPY

To receive a copy of this document, indicate in the box: "C" = Copy without enclosures "E" = Copy with enclosures "N" = No copy

OFFICE	LA:PDIII-2	PM:PDIII-2	D:PDIII-2				
NAME	C. Moore	G. Dick	R. Capra				
DATE	01/10/95	01/10/95	01/19/95				

9501240154 950119
PDR ADOCK 05000454
P PDR

OFFICIAL RECORD COPY

[Handwritten signature]

[Handwritten mark]

January 19, 1995

Mr. D. L. Farrar
Manager, Nuclear Regulatory Services
Commonwealth Edison Company
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, Illinois 60515

SUBJECT: CORRECTION OF AMENDMENT (TAC NOS. M90575 AND M90576)

Dear Mr. Farrar:

On October 21, 1994, the U.S. Nuclear Regulatory Commission issued Amendment No. 65 to Facility Operating License No. NPF-37 and Amendment No. 65 to Facility License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively.

Revised Technical Specification Basis page B 3/4 1-3 was incomplete. Please replace the previously issued page with that enclosed.

We regret any inconvenience this error may have created.

Sincerely,

original signed by:

George F. Dick, Jr., Project Manager
Project Directorate III-2
Division of Reactor Projects III/IV
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455

Enclosure: Corrected page B 3/4 1-3

cc w/encl: see next page

DISTRIBUTION:

Docket File	PUBLIC
PDIII-2 R/F	J. Roe
R. Capra	C. Moore
G. Dick	OGC
D. Hagan	G. Hill (4)
L. Miller, RIII	C. Grimes
ACRS (4)	OPA
OC/LFDCB	H. Richings

DOCUMENT NAME: BY90575.LTR

To receive a copy of this document, indicate in the box: "C" = Copy without enclosures "E" = Copy with enclosures "N" = No copy

OFFICE	LA:PDIII-2	PM:PDIII-2	D:PDIII-2				
NAME	C. Moore	G. Dick	R. Capra				
DATE	01/12/95	01/18/95	01/19/95				

OFFICIAL RECORD COPY

D. L. Farrar
Commonwealth Edison Company

Byron Station
Unit Nos. 1 and 2

cc:

Mr. William P. Poirier, Director
Westinghouse Electric Corporation
Energy Systems Business Unit
Post Office Box 355, Bay 236 West
Pittsburgh, Pennsylvania 15230

Ms. Lorraine Creek
Rt. 1, Box 182
Manteno, Illinois 60950

Joseph Gallo
Gallo & Ross
1250 Eye St., N.W.
Suite 302
Washington, DC 20005

Chairman, Ogle County Board
Post Office Box 357
Oregon, Illinois 61061

Michael I. Miller, Esquire
Sidley and Austin
One First National Plaza
Chicago, Illinois 60690

Mrs. Phillip B. Johnson
1907 Stratford Lane
Rockford, Illinois 61107

Howard A. Learner
Environmental Law and Policy
Center of the Midwest
203 North LaSalle Street
Suite 1390
Chicago, Illinois 60601

Attorney General
500 South Second Street
Springfield, Illinois 62701

EIS Review Coordinator
U.S. Environmental Protection Agency
77 W. Jackson Blvd.
Chicago, Illinois 60604-3590

U.S. Nuclear Regulatory Commission
Byron Resident Inspectors Office
4448 North German Church Road
Byron, Illinois 61010-9750

Illinois Department of
Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, Illinois 60532-4351

Commonwealth Edison Company
Byron Station Manager
4450 North German Church Road
Byron, Illinois 61010

Kenneth Graesser, Site Vice President
Byron Station
Commonwealth Edison Station
4450 N. German Church Road
Byron, Illinois 61010

REACTIVITY CONTROL SYSTEMS

BASES

BORATION SYSTEMS (Continued)

With the RCS temperature below 350°F, one Boron Injection System is acceptable without single failure consideration on the basis of the stable reactivity condition of the reactor and the additional restrictions prohibiting CORE ALTERATIONS and positive reactivity changes in the event the single Boron Injection System becomes inoperable.

The limitation for a maximum of one centrifugal charging pump to be OPERABLE and the Surveillance Requirement to verify all charging pumps except the required OPERABLE pump to be inoperable below 330°F provides assurance that a mass addition pressure transient can be relieved by the operation of a single PORV or an RHR Suction relief valve.

The boron capability required below 200°F is sufficient to provide a SHUTDOWN MARGIN of 1% $\Delta k/k$ after xenon decay and cooldown from 200°F to 140°F. This condition requires either 740 (2,652*) gallons of 7000-ppm borated water from the boric acid storage tanks or 2264 (11,840*) gallons of 2300-ppm (2000-ppm*) borated water from the refueling water storage tank (RWST). A Boric Acid Storage System level of 7% ensures there is a volume of greater than or equal to 740 (2652*) gallons available. An RWST level of 9% ensures there is a volume of greater than or equal to 38,740 gallons available.

The contained water volume limits include allowance for water not available because of discharge line location and other physical characteristics.

The limits on contained water volume and boron concentration of the RWST also ensure a pH value of between 8.0 and 11.0 for the solution recirculated within containment after a LOCA. This pH band minimizes the evolution of iodine and minimizes the effect of chloride and caustic stress corrosion on mechanical systems and components.

The OPERABILITY of one Boron Injection System during REFUELING ensures that this system is available for reactivity control while in MODE 6.

The OPERABILITY of the automatic Boron Dilution Protection System ensures adequate capability for negative reactivity insertion to prevent a transient caused by the uncontrolled dilution of the RCS in MODES 3, 4, and 5. The functioning of the system precludes the necessity of operator action to prevent further dilution by terminating flow to the charging pump(s) from possible unborated water sources and initiating flow from the RWST. The most restrictive condition occurs shortly after beginning of life when the critical boron concentration is highest, and a 205 gpm dilution flowrate provides the maximum positive reactivity addition rate. One reactor coolant pump in operation with all reactor coolant loop stop isolation valves open reduces the reactivity addition rate by mixing the dilution through all four reactor coolant loops. A minimum count rate of ten counts per second minimizes the impact of the uncertainties associated with the source range nuclear instrumentation. In the analysis of this accident, a minimum SHUTDOWN MARGIN of 1.3 $\Delta k/k$ is required to control the reactivity transient. Actions taken by the microprocessor if the neutron count rate is doubled will prevent return to criticality in these MODES.

*Not applicable to Unit 1. Applicable to Unit 2 until completion of cycle 5.

9501240159 950119
PDR ADOCK 05000454
P PDR