

October 17, 1990

Docket Nos. STN 50-454
STN 50-455
STN 50-456
and STN 50-457

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, Illinois 60515

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Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 76719, 76720, 76721 AND 76722)

The Commission has issued the enclosed Amendment No. 39 to Facility Operating License No. NPF-37 and Amendment No. 39 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively, and Amendment No. 26 to Facility Operating License No. NPF-72 and Amendment No. 26 to Facility Operating License No. NPF-77 for the Braidwood Station, Unit Nos. 1 and 2, respectively. The amendments are in response to your application dated April 20, 1990.

These amendments approve changes to Technical Specification 3/4.6.3, Containment Isolation Valves, which would delete the requirement for Type C leakage testing for specified steam generator blowdown isolation valves and insert a requirement for the Type C leakage test for the 1/2 SI 8968 safety injection valves.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original Signed By:

Stephen P. Sands, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

9010300067 901017
PDR ADOCK 05000454
P PNU

Enclosures:

1. Amendment No. 39 to NPF-37
2. Amendment No. 39 to NPF-66
3. Amendment No. 26 to NPF-72
4. Amendment No. 26 to NPF-77
5. Safety Evaluation

cc w/enclosures:

See next page

*SEE PREVIOUS CONCURRENCE

OFFICIAL RECORD COPY

DOCUMENT NAME: [AMENDMENT 76719/20/21/22]

Office: *LA/PDIII-2 PM/PDIII-2

Surname: CMoore SSands:ta

Date: 08/24/90 10/17/90

PD/PDIII-2

RBarrett

10/17/90

*OGC-WF1

AHodgdon

08/27/90

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 17, 1990

Docket Nos. STN 50-454
STN 50-455
STN 50-456
and STN 50-457

Mr. Thomas J. Kovach
Nuclear Licensing Manager
Commonwealth Edison Company-Suite 300
OPUS West III
1400 OPUS Place
Downers Grove, Illinois 60515

Dear Mr. Kovach:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. 76719, 76720, 76721 AND 76722)

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These amendments approve changes to Technical Specification 3/4.6.3, Containment Isolation Valves, which would delete the requirement for Type C leakage testing for specified steam generator blowdown isolation valves and insert a requirement for the Type C leakage test for the 1/2 SI 8968 safety injection valves.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Stephen P. Sands".

Stephen P. Sands, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 39 to NPF-37
2. Amendment No. 39 to NPF-66
3. Amendment No. 26 to NPF-72
4. Amendment No. 26 to NPF-77
5. Safety Evaluation

cc w/enclosures:
See next page

Mr. Thomas J. Kovach
Commonwealth Edison Company

Byron/Braidwood Power Station
Unit Nos. 1 and 2

cc:

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Westinghouse Electric Corporation
Energy Systems Business Unit
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Byron, Illinois 61010

Illinois Department of
Nuclear Safety
Office of Nuclear Facility Safety
1035 Outer Park Drive
Springfield, Illinois 62704



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39
License No. NPF-37

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-37 is hereby amended to read as follows:

9010300081 901017
PDR ADOCK 05000454
P PNU

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 39 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 17, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 39
License No. NPF-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 39 and revised by Attachment 2 to NPF-66, and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, dated February 14, 1985, are hereby incorporated into this license. Attachment 2 contains a revision to Appendix A which is hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 17, 1990

ATTACHMENT TO LICENSE AMENDMENT NOS. 39 AND 39

FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66

DOCKET NOS. STN 50-454 AND STN 50-455

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Corresponding overleaf pages (indicated by an asterisk) are provided to maintain document completeness.

<u>Remove Pages</u>	<u>Insert Pages</u>
*3/4 6-17	*3/4 6-17
3/4 6-18	3/4 6-18
*3/4 6-23	*3/4 6-23
3/4 6-24	3/4 6-24

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Phase "A" Isolation test signal, each Phase "A" isolation valve actuates to its isolation position;
- b. Verifying that on a Phase "B" Isolation test signal, each Phase "B" isolation valve actuates to its isolation position; and
- c. Verifying that on a Containment Vent Isolation test signal, each purge and exhaust isolation valve actuates to its isolation position.

4.6.3.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

TABLE 3.6-1
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
1. <u>Phase "A" Isolation</u>			
28	CV8100	RCP Seal Water Return	10
28	CV8112	RCP Seal Water Return	10
41	CV8152	RCS Letdown	10
41	CV8160	RCS Letdown	10
5	WØ020A	Chilled Water	50
5	WØ056A	Chilled Water	50
6	WØ006A	Chilled Water	50
8	WØ020B	Chilled Water	50
8	WØ056B	Chilled Water	50
10	WØ006B	Chilled Water	50
22	CC9437B*	Excess Ltdn HX Return	10
48	CC9437A*	Excess Ltdn HX Supply	10
34	FP010*	Fire Protection	12
39	IA065	Instrument Air	15
39	IA066	Instrument Air	15
13	ØG079	Hydrogen Recombiner	60
13	ØG080	Hydrogen Recombiner	60
13	ØG082	Hydrogen Recombiner	60
13	ØG084	Hydrogen Recombiner	60
23	ØG081	Hydrogen Recombiner	60
23	ØG085	Hydrogen Recombiner	60
69	ØG057A	Hydrogen Recombiner	60
69	ØG083	Hydrogen Recombiner	60
56	SA032	Service Air	4.5
56	SA033	Service Air	4.5
80	SD002C*	Steam Generator Blowdown	7.5
80	SD005B*	Steam Generator Blowdown	3.0
81	SD002D*	Steam Generator Blowdown	7.5
82	SD002A*	Steam Generator Blowdown	7.5
82	SD005A*	Steam Generator Blowdown	3.0
83	SD002B*	Steam Generator Blowdown	7.5
88	SD002E*	Steam Generator Blowdown	7.5
88	SD005C*	Steam Generator Blowdown	3.0
89	SD002F*	Steam Generator Blowdown	7.5
90	SD002G*	Steam Generator Blowdown	7.5
90	SD005D*	Steam Generator Blowdown	3.0
91	SD002H*	Steam Generator Blowdown	7.5

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
9. <u>Manual</u> (Continued)			
99	FW0150*,#	Feedwater	N.A.
100	FW015A*,#	Feedwater	N.A.
101	FW015B*,#	Feedwater	N.A.
102	FW015C*,#	Feedwater	N.A.
10. <u>Check</u>			
28	CV8113	RCP Seal Water Return	N.A.
37	CV8348*	RCS Loop Fill	N.A.
6	W0007A	Chilled Water	N.A.
10	W0007B	Chilled Water	N.A.
21	CC9534	RCP Mtr Brng Return	N.A.
24	CC9518	RCP Thermal Barrier Return	N.A.
25	CC9486	RCP Cooling Wtr Supply	N.A.
1	CS008A	Containment Spray	N.A.
16	CS008B	Containment Spray	N.A.
39	IA091	Instrument Air	N.A.
30	WM191	Make-Up Demin	N.A.
52	PR032	Process Radiation	N.A.
AL	PR002G	Process Radiation	N.A.
AL	PR002H	Process Radiation	N.A.
12	PS231A	Hydrogen Monitor	N.A.
31	PS231B	Hydrogen Monitor	N.A.
27	RY8047	PRT Nitrogen	N.A.
44	RY8046	PRT Make-Up	N.A.
26	SI8815*	Safety Injection	N.A.
50	SI8818A*	Safety Injection	N.A.
50	SI8818D*	Safety Injection	N.A.
51	SI8818B*	Safety Injection	N.A.
51	SI8818C*	Safety Injection	N.A.
59	SI8905A*	Safety Injection	N.A.
59	SI8905D*	Safety Injection	N.A.
60	SI8819A*	Safety Injection	N.A.
60	SI8819B*	Safety Injection	N.A.

TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
10. <u>Check (Continued)</u>			
60	SI8819C*	Safety Injection	N.A.
60	SI8819D*	Safety Injection	N.A.
66	SI8841A*	Safety Injection	N.A.
66	SI8841B*	Safety Injection	N.A.
73	SI8905B*	Safety Injection	N.A.
73	SI8905C*	Safety Injection	N.A.
55	SI8968	Safety Injection	N.A.
34	FP345*	Fire Protection	N.A.
33	CV8368A*	RCP Seal Injection	N.A.
33	CV8368D*	RCP Seal Injection	N.A.
53	CV8368B*	RCP Seal Injection	N.A.
53	CV8368C*	RCP Seal Injection	N.A.
11. <u>S/G Safeties/PORVs</u>			
77	MS013D*	Main Steam	N.A.
77	MS014D*	Main Steam	N.A.
77	MS015D*	Main Steam	N.A.
77	MS016D*	Main Steam	N.A.
77	MS017D*	Main Steam	N.A.
78	MS013A*	Main Steam	N.A.
78	MS014A*	Main Steam	N.A.
78	MS015A*	Main Steam	N.A.
78	MS016A*	Main Steam	N.A.
78	MS017A*	Main Steam	N.A.
85	MS013B*	Main Steam	N.A.
85	MS014B*	Main Steam	N.A.
85	MS015B*	Main Steam	N.A.
85	MS016B*	Main Steam	N.A.
85	MS017B*	Main Steam	N.A.
86	MS013C*	Main Steam	N.A.
86	MS014C*	Main Steam	N.A.
86	MS015C*	Main Steam	N.A.
86	MS016C*	Main Steam	N.A.
86	MS017C*	Main Steam	N.A.
77	MS018D*	Main Steam	20
78	MS018A*	Main Steam	20
85	MS018B*	Main Steam	20
86	MS018C*	Main Steam	20

*Not subject to Type C leakage tests.

**Proper valve operation will be demonstrated by verifying that the valve strokes to its required position.

#May be opened on an intermittent basis under administrative control.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 26
License No. NPF-72

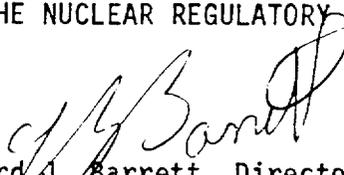
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 26 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 17, 1990



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 26
License No. NPF-77

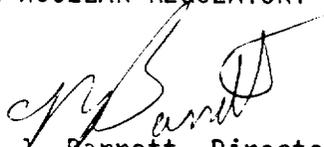
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated April 20, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 26 and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-72, dated July 2, 1987, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard J. Barrett, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 17, 1990

ATTACHMENT TO LICENSE AMENDMENT NOS. 26 AND 26
FACILITY OPERATING LICENSE NOS. NPF-72 AND NPF-77
DOCKET NOS. STN 50-456 AND STN 50-457

Replace the following pages of the Appendix "A" Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change. Corresponding overleaf pages (indicated by an asterisk) are provided to maintain document completeness.

<u>Remove Pages</u>	<u>Insert Pages</u>
*3/4 6-17	*3/4 6-17
3/4 6-18	3/4 6-18
*3/4 6-23	*3/4 6-23
3/4 6-24	3/4 6-24

SURVEILLANCE REQUIREMENTS (Continued)

4.6.3.2 Each isolation valve specified in Table 3.6-1 shall be demonstrated OPERABLE during the COLD SHUTDOWN or REFUELING MODE at least once per 18 months by:

- a. Verifying that on a Phase "A" Isolation test signal, each Phase "A" isolation valve actuates to its isolation position;
- b. Verifying that on a Phase "B" Isolation test signal, each Phase "B" isolation valve actuates to its isolation position; and
- c. Verifying that on a Containment Vent Isolation test signal, each purge and exhaust isolation valve actuates to its isolation position.

4.6.3.3 The isolation time of each power operated or automatic valve of Table 3.6-1 shall be determined to be within its limit when tested pursuant to Specification 4.0.5.

TABLE 3.6-1

CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
1. <u>Phase "A" Isolation</u>			
28	CV8100	RCP Seal Water Return	10
28	CV8112	RCP Seal Water Return	10
41	CV8152	RCS Letdown	10
41	CV8160	RCS Letdown	10
5	WØ020A	Chilled Water	50
5	WØ056A	Chilled Water	50
6	WØ006A	Chilled Water	50
8	WØ020B	Chilled Water	50
8	WØ056B	Chilled Water	50
10	WØ006B	Chilled Water	50
22	CC9437B*	Excess Ltdn HX Return	10
48	CC9437A*	Excess Ltdn HX Supply	10
34	FP010*	Fire Protection	12
39	IA065	Instrument Air	15
39	IA066	Instrument Air	15
13	ØG079	Hydrogen Recombiner	60
13	ØG080	Hydrogen Recombiner	60
13	ØG082	Hydrogen Recombiner	60
13	ØG084	Hydrogen Recombiner	60
23	ØG081	Hydrogen Recombiner	60
23	ØG085	Hydrogen Recombiner	60
69	ØG057A	Hydrogen Recombiner	60
69	ØG083	Hydrogen Recombiner	60
56	SA032	Service Air	4.5
56	SA033	Service Air	4.5
80	SD002C*	Steam Generator Blowdown	7.5
80	SD005B*	Steam Generator Blowdown	3.0
81	SD002D*	Steam Generator Blowdown	7.5
82	SD002A*	Steam Generator Blowdown	7.5
82	SD005A*	Steam Generator Blowdown	3.0
83	SD002B*	Steam Generator Blowdown	7.5
88	SD002E*	Steam Generator Blowdown	7.5
88	SD005C*	Steam Generator Blowdown	3.0
89	SD002F*	Steam Generator Blowdown	7.5
90	SD002G*	Steam Generator Blowdown	7.5
90	SD005D*	Steam Generator Blowdown	3.0
91	SD002H*	Steam Generator Blowdown	7.5

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
9. <u>Manual</u> (Continued)			
99	FW015D*,#	Feedwater	N. A.
100	FW015A*,#	Feedwater	N. A.
101	FW015B*,#	Feedwater	N. A.
102	FW015C*,#	Feedwater	N. A.
10. <u>Check</u>			
28	CV8113	RCP Seal Water Return	N. A.
37	CV8348*	RCS Loop Fill	N. A.
6	WØ007A	Chilled Water	N. A.
10	WØ007B	Chilled Water	N. A.
21	CC9534	RCP Mtr Brng Return	N. A.
24	CC9518	RCP Thermal Barrier Return	N. A.
25	CC9486	RCP Cooling Wtr Supply	N. A.
1	CS008A	Containment Spray	N. A.
16	CS008B	Containment Spray	N. A.
39	IA091	Instrument Air	N. A.
30	WM191	Make-Up Demin	N. A.
52	PR032	Process Radiation	N. A.
AL	PR002G	Process Radiation	N. A.
AL	PR002H	Process Radiation	N. A.
12	PS231A	Hydrogen Monitor	N. A.
31	PS231B	Hydrogen Monitor	N. A.
27	RY8047	PRT Nitrogen	N. A.
44	RY8046	PRT Make-Up	N. A.
26	SI8815*	Safety Injection	N. A.
50	SI8818A*	Safety Injection	N. A.
50	SI8818D*	Safety Injection	N. A.
51	SI8818B*	Safety Injection	N. A.
51	SI8818C*	Safety Injection	N. A.
59	SI8905A*	Safety Injection	N. A.
59	SI8905D*	Safety Injection	N. A.
60	SI8819A*	Safety Injection	N. A.
60	SI8819B*	Safety Injection	N. A.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
10. <u>Check (Continued)</u>			
60	SI8819C*	Safety Injection	N.A.
60	SI8819D*	Safety Injection	N.A.
66	SI8841A*	Safety Injection	N.A.
66	SI8841B*	Safety Injection	N.A.
73	SI8905B*	Safety Injection	N.A.
73	SI8905C*	Safety Injection	N.A.
55	SI8968	Safety Injection	N.A.
34	FP345*	Fire Protection	N.A.
33	CV8368A*	RCP Seal Injection	N.A.
33	CV8368D*	RCP Seal Injection	N.A.
53	CV8368B*	RCP Seal Injection	N.A.
53	CV8368C*	RCP Seal Injection	N.A.
11. <u>S/G Safeties/PORVs</u>			
77	MS013D*	Main Steam	N.A.
77	MS014D*	Main Steam	N.A.
77	MS015D*	Main Steam	N.A.
77	MS016D*	Main Steam	N.A.
77	MS017D*	Main Steam	N.A.
78	MS013A*	Main Steam	N.A.
78	MS014A*	Main Steam	N.A.
78	MS015A*	Main Steam	N.A.
78	MS016A*	Main Steam	N.A.
78	MS017A*	Main Steam	N.A.
85	MS013B*	Main Steam	N.A.
85	MS014B*	Main Steam	N.A.
85	MS015B*	Main Steam	N.A.
85	MS016B*	Main Steam	N.A.
85	MS017B*	Main Steam	N.A.
86	MS013C*	Main Steam	N.A.
86	MS014C*	Main Steam	N.A.
86	MS015C*	Main Steam	N.A.
86	MS016C*	Main Steam	N.A.
86	MS017C*	Main Steam	N.A.
77	MS018D*	Main Steam	20
78	MS018A*	Main Steam	20
85	MS018B*	Main Steam	20
86	MS018C*	Main Steam	20

*Not subject to Type C leakage tests.

**Proper valve operation will be demonstrated by verifying that the valve strokes to its required position.

#May be opened on an intermittent basis under administrative control.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-37,
AMENDMENT NO. 39 TO FACILITY OPERATING LICENSE NO. NPF-66,
AMENDMENT NO. 26 TO FACILITY OPERATING LICENSE NO. NPF-72,
AND AMENDMENT NO. 26 TO FACILITY OPERATING LICENSE NO. NPF-77
COMMONWEALTH EDISON COMPANY
BYRON STATION, UNIT NOS. 1 AND 2
BRAIDWOOD STATION, UNIT NOS. 1 AND 2
DOCKET NOS. STN 50-454, STN 50-455, STN 50-456 AND STN 50-457

1.0 INTRODUCTION

By letters dated April 20, 1990 and September 6, 1990, Commonwealth Edison Company (CECo) (the licensee) proposed an amendment to the Technical Specifications for both Byron Station Unit Nos. 1 and 2 and Braidwood Station Unit Nos. 1 and 2. The proposed amendment revises TS 3/4.6.3, Containment Isolation Valves, to delete the requirement for Type C leakage rate testing for specific Steam Generator (SG) blowdown isolation valves (while continuing to leakage rate test under the Inservice Testing (IST) program) and to add a requirement for Type C leakage rate testing of the 1/2 SI 8968 safety injection valves.

The staff's review of the licensee's proposal and of the licensee's bases for the proposed changes is given below.

2.0 EVALUATION

The steam generator (SG) Blowdown lines transfer secondary water to the SG blowdown system (SD) for cleanup during plant operation. The SG blowdown lines (and the attached secondary side of the SG and main steam system) are neither a part of the reactor coolant system (RCS) pressure boundary nor do they open directly to the containment atmosphere under post-LOCA conditions. In accordance with General Design Criterion (GDC) 57, the steam generator shell and the lines attached to it (such as the main steam line, feedwater line, and the SG blowdown lines) constitute a closed system inside containment, and it is one of the containment isolation barriers for each associated containment penetration. The second redundant barrier at each penetration is a valve, such as the main steam or main feedwater isolation valves and the SG blowdown valves. However, the main steam and main feedwater isolation valves are not required to be locally leakage rate

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tested (Type C tested) in accordance with Section II.H.4 of Appendix J to 10 CFR Part 50. It should be noted that the integrity of the closed systems inside containment are tested during periodic Type A (integrated leakage rate) testing. By letter dated September 6, 1990, the licensee described its commitment to leakage rate test the subject valves as part of the IST program, using water or air as the test medium in accordance with ASME Section XI. The staff considers leakage rate testing of these valves to be a necessary part of periodic verification of the valves' capability to fulfill their safety function. Therefore, the testing committed to by the licensee is acceptable.

The staff has previously allowed other plants to discontinue Type C testing of the SG blowdown valves (for example, Comanche Peak and Seabrook) on the following basis: the secondary system is a closed system inside containment and is not postulated to rupture during an accident. It will preclude the containment atmosphere from reaching the associated isolation valves; therefore, the valves will not be relied on to limit containment leakage.

In light of the above discussion and the licensee's commitment to leakage rate testing of the SG blowdown valves under the IST program, the staff finds the licensee's proposal to discontinue Type C testing of the SG blowdown valves acceptable.

It should be noted that the integrity of the closed systems inside containment will be tested during periodic Type A (integrated leakage rate) testing. Therefore, the staff finds the licensee's proposal to discontinue Type C testing of the SG blowdown valves acceptable. However, this does not necessarily relieve the SG blowdown valves from the testing requirements of the Inservice Testing (IST) program under ASME Section XI and 10 CFR 50.55a. The licensee's submittal stated that the valves perform other safety functions besides limiting containment leakage, such as conserving SG secondary side water mass for heat sink during an accident. These safety functions should be considered when determining testing provisions under the IST program and whether the testing should include leakage rate testing. However, the IST program is outside the scope of this review, which only addresses Appendix J requirements.

The second request for addition of a requirement that would mandate Type C leakage rate testing for the 1/2 SI 8968 safety injection valves is conservative and is therefore acceptable to the staff.

3.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change to a requirement with respect to the installation or use of the facility components located within the restricted areas as defined in 10 CFR 20 or a change to a surveillance requirement. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding.

Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4.0 CONCLUSION

Since the SG blowdown valves will not be relied upon to perform a containment isolation function, the staff finds the licensee's proposal to discontinue the Type C testing on SG blowdown valves currently required by TS 3/4.6.3 to be acceptable. Further, the staff finds that the addition of a requirement to Type C test the 1/2 SI 8968 safety injection valves is also acceptable.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: J. Pulsipher

Dated: October 17, 1990