

October 30, 1

Docket Nos. STN 50-454, STN 50-455
and STN 50-456, STN 50-457

Mr. L. D. Butterfield, Jr.
Nuclear Licensing Manager
Commonwealth Edison Company
Post Office Box 767
Chicago, Illinois 60690

Dear Mr. Butterfield:

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The Commission has issued the enclosed Amendment No. 11 to Facility Operating License No. NPF-37 and Amendment No. 11 to Facility Operating License No. NPF-66 for the Byron Station, Unit Nos. 1 and 2, respectively and Amendment No. 1 to Facility Operating License No. NPF-72 for Braidwood Station, Unit No. 1. The amendment also applies to Braidwood Station, Unit No. 2, although Unit No. 2 does not have an operating license. Braidwood Station, Unit Nos. 1 and 2, have common Technical Specifications. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated August 7, 1987.

These amendments approve changes to the Technical Specifications that allow a one-time extension for diesel generator surveillance testing interval to 31 months from 18 months.

In addition, these amendments correct an oversight made in Amendment No. 9 for Byron Station, issued July 23, 1987. In that amendment, the title of the "Supervisor of the Offsite Review and Investigative Function" was changed to "Superintendent of the Offsite Review and Investigative Function". However, not all the references to this title were changed in Amendment No. 9. Thus, these amendments for Byron correct this oversight.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

Stephen Sands, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Leonard N. Olshan, Project Manager
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Enclosures:

- | | |
|-------------------------------|------------------------------|
| 1. Amendment No. 11 to NPF-37 | 3. Amendment No. 1 to NPF-72 |
| 2. Amendment No. 11 to NPF-66 | 4. Safety Evaluation |

cc: w/enclosures
See next page

PDIII-2:LA *LA*
LLuther:bj
09/30/87

PDIII-2:PM *PM*
LOlshan
9/30/87

PDIII-2 *JPS*
SSands
10/30/87
PDIII-2:PD *PD*
Dwyler
10/30/87

OGC-Bethesda *AB*
RBachmans
10/19/87

PSB
JCraig
10/31/87

Mr. L. D. Butterfield, Jr.
Commonwealth Edison Company

Byron/Braidwood

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Mr. L. D. Butterfield, Jr.
Commonwealth Edison Company

- 2 - Byron/Braidwood

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-454

BYRON STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. NPF-37

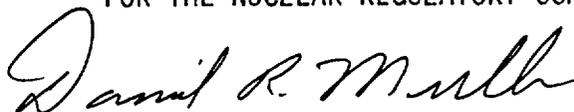
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 7, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-37 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 11 and the Environment Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 30, 1987



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-455

BYRON STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 11
License No. NPF-66

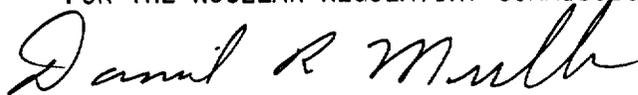
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 7, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 11 and revised by Attachment 2 to NPF-60, and the Environmental Protection Plan contained in Appendix B, both of which were attached to License No. NPF-37, dated February 14, 1985, are hereby incorporated into this license. Attachment 2 contains a revision to Appendix A which is hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 30, 1987

ATTACHMENT TO LICENSE AMENDMENT NOS. 11 AND 11
FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66
DOCKET NOS. STN-50-454 AND STN 50-455

Revise Appendix A as follows:

Remove Pages

3/4 8-3*
3/4 8-4
6-7
6-8*
6-9*
6-10
6-11*
6-12
6-13
6-14

Insert Pages

3/4 8-3*
3/4 8-4
6-7
6-8*
6-9*
6-10
6-11*
6-12
6-13
6-14

*Overleaf pages added for convenience

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS

4.8.1.1.1 Each of the above required independent circuits between the offsite transmission network and the Onsite Class 1E Distribution System shall be:

- a. Determined OPERABLE at least once per 7 days by verifying correct breaker alignments, indicated power availability, and
- b. Demonstrated OPERABLE at least once per 18 months during shutdown by transferring manually unit power supply from the normal circuit to the alternate circuit.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

- a. In accordance with the frequency specified in Table 4.8-1 on a STAGGERED TEST BASIS by:
 - 1) Verifying the fuel level in the day tank,
 - 2) Verifying the fuel level in the fuel storage tank,
 - 3) Verifying the fuel transfer pump starts and transfers fuel from the storage system to the day tank,
 - 4) Verifying the diesel starts from ambient condition and accelerates to at least 600 rpm in less than or equal to 10 seconds.* The generator voltage and frequency shall be 4160 ± 420 volts and 60 ± 1.2 Hz within 10 seconds* after the start signal. The diesel generator shall be started for this test by using one of the following signals:
 - a) Manual, or
 - b) Simulated loss of ESF bus voltage by itself, or
 - c) Simulated loss of ESF bus voltage in conjunction with an ESF actuation test signal, or
 - d) An ESF actuation test signal by itself.
 - 5) Verifying the generator is synchronized, loaded to greater than or equal to 5500 kW in less than or equal to 60 seconds*, operates with a load greater than or equal to 5500 kW for at least 60 minutes, and
 - 6) Verifying the diesel generator is aligned to provide standby power to the associated ESF busses.
- b. At least once per 31 days and after each operation of the diesel where the period of operation was greater than or equal to 1 hour by checking for and removing accumulated water from the day tanks;

*The diesel generator start (10 sec) from ambient conditions shall be performed at least once per 184 days in these surveillance tests. All other engine starts for the purpose of this surveillance testing may be preceded by an engine pre-lube period and/or other warmup procedures recommended by the manufacturer so that mechanical stress and wear on the diesel engine is minimized.

6.5 REVIEW INVESTIGATION AND AUDIT (Continued)

OFFSITE

6.5.1 The Superintendent of the Offsite Review and Investigative Function shall be appointed by the Manager of Nuclear Safety responsible for nuclear activities. The audit function shall be the responsibility of the Manager of Quality Assurance and shall be independent of operations.

a. Offsite Review and Investigative Function

The Superintendent of the Offsite Review and Investigative Function shall: (1) provide directions for the review and investigative function and appoint a senior participant to provide appropriate direction, (2) select each participant for this function, (3) select a complement of more than one participant who collectively possess background and qualifications in the subject matter under review to provide comprehensive interdisciplinary review coverage under this function, (4) independently review and approve the findings and recommendations developed by personnel performing the review and investigative function, (5) approve and report in a timely manner all findings of non-compliance with NRC requirements to the Station Manager, Assistant Vice President and General Manager - Nuclear Stations, Manager of Quality Assurance, and the Vice President - Nuclear Operations. During periods when the Supervisor of Offsite Review and Investigative Function is unavailable, he shall designate this responsibility to an established alternate, who satisfies the formal training and experience for the Superintendent of the Offsite Review and Investigative Function. The responsibilities of the personnel performing this function are stated below. The Offsite Review and Investigative Function shall review:

- 1) The safety evaluations for: (1) changes to procedures, equipment, or systems as described in the safety analysis report, and (2) tests or experiments completed under the provision of 10 CFR 50.59 to verify that such actions did not constitute an unreviewed safety question. Proposed changes to the Quality Assurance Program description shall be reviewed and approved by the Manager of Quality Assurance;
- 2) Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in 10 CFR 50.59;
- 3) Proposed tests or experiments which involve an unreviewed safety question as defined in 10 CFR 50.59;
- 4) Proposed changes in Technical Specifications or this Operating License;

ADMINISTRATIVE CONTROLS

OFFSITE (Continued)

- 5) The Facility Emergency Plan and implementing procedures at least once per 12 months;
- 6) The Facility Security Plan and implementing procedures at least once per 12 months;
- 7) Onsite and offsite reviews;
- 8) The Facility Fire Protection programmatic controls including the implementing procedures at least once per 24 months by qualified QA personnel;
- 9) The fire protection equipment and program implementation at least once per 12 months utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- 10) The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months;
- 11) The OFFSITE DOSE CALCULATION MANUAL and implementing procedures at least once per 24 months;
- 12) The PROCESS CONTROL PROGRAM and implementing procedures for solidification of radioactive wastes at least once per 24 months; and
- 13) The performance of activities required by the Company Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

Report all findings of noncompliance with NRC requirements and recommendations and results of each audit to the Station Manager, Manager of Nuclear Safety, the Assistant Vice President and General Manager - Nuclear Stations, Manager of Quality Assurance, the Vice Chairman, and the Vice President - Nuclear Operations.

c. Authority

The Manager of Quality Assurance and the Manager of Nuclear Safety report to the Chairman and President. Either the Manager of Quality Assurance or the Manager of Nuclear Safety has the authority to order unit shutdown or request any other action which he deems necessary to avoid unsafe plant conditions.

OFFSITE (Continued)

- h) Instrumentation and control,
 - i) Metallurgy, and
 - j) Any other appropriate disciplines required by unique characteristics of the facility.
- 2) Individuals performing the Review and Investigative Function shall possess a minimum formal training and experience as listed below for each discipline.
- a) Nuclear Power Plant Technology
Engineering graduate or equivalent with 5 years experience in the nuclear power field design and/or operation.
 - b) Reactor Operations
Engineering graduate or equivalent with 5 years experience in nuclear power plant operations.
 - c) Utility Operations
Engineering graduate or equivalent with at least 5 years of experience in utility operation and/or engineering.
 - d) Power Plant Design
Engineering graduate or equivalent with at least 5 years of experience in power plant design and/or operation.
 - e) Reactor Engineering
Engineering graduate or equivalent. In addition, at least 5 years of experience in nuclear plant engineering, operation, and/or graduate work in nuclear engineering or equivalent in reactor physics is required.
 - f) Radiological Safety
Engineering graduate or equivalent with at least 5 years of experience in radiation control and safety.
 - g) Reactor Safety Analysis
Engineering graduate or equivalent with at least 5 years of experience in nuclear engineering.

ADMINISTRATIVE CONTROLS

ONSITE (Continued)

- 3) Review of all proposed changes to the Technical Specifications;
- 4) Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety;
- 5) Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Assistant Vice President and General Manager - Nuclear Stations and to the Superintendent of the Offsite Review and Investigative Function;
- 6) Review of all REPORTABLE EVENTS;
- 7) Performance of special reviews and investigations and reports thereon as requested by the Superintendent of the Offsite Review and Investigative Function;
- 8) Review of the Station Security Plan and implementing procedures and submittal of recommended changes to the Assistant Vice President and General Manager - Nuclear Stations;
- 9) Review of the Emergency Plan and station implementing procedures and submittal of recommended changes to the Assistant Vice President and General Manager - Nuclear Stations;
- 10) Review of Unit operations to detect potential hazards to nuclear safety;
- 11) Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the Assistant Vice President and General Manager - Nuclear Stations and the Superintendent of the Offsite Review and Investigative Function; and
- 12) Review of changes to the PROCESS CONTROL PROGRAM, the OFFSITE DOSE CALCULATION MANUAL, and the Radwaste Treatment Systems.
- 13) Review of the Fire Protection Program and implementing instructions and submittal of recommended changes to the Offsite Review and Investigative Function.

c. Authority

The Technical Staff Supervisor is responsible to the Station Manager and shall make recommendations in a timely manner in all areas of review, investigation, and quality control phases of plant maintenance, operation, and administrative procedures relating to facility operations and shall have the authority to request the action necessary to ensure compliance with rules, regulations, and procedures when in his opinion such action is necessary. The Station Manager shall follow such recommendations or select a course



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. NPF-72

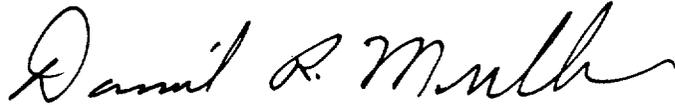
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Commonwealth Edison Company (the licensee) dated August 7, 1987 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 1 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Daniel R. Muller, Director
Project Directorate III-2
Division of Reactor Projects - III,
IV, V and Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 30, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 1
FACILITY OPERATING LICENSE NO. NPF-72
DOCKET NO. STN-50-456

Revise Appendix A as follows:

Remove Pages

3/4 8-3*
3/4 8-4

Insert Pages

3/4 8-3*
3/4 8-4

*Overleaf pages added for convenience

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- c. At least once per 31 days by checking for and removing accumulated water from the fuel oil storage tanks;
- d. By sampling new fuel oil in accordance with ASTM-D4057 prior to addition to storage tanks and:
 - 1) By verifying in accordance with the tests specified in ASTM-D975-81 prior to addition to the storage tanks that the sample has:
 - a) An API Gravity of within 0.3 degrees at 60°F, or a specific gravity of within 0.0016 at 60°F, when compared to the supplier's certificate, or an absolute specific gravity at 60°F of greater than or equal to 0.83 but less than or equal to 0.89, or an API gravity of greater than or equal to 27 degrees but less than or equal to 39 degrees;
 - b) A kinematic viscosity at 40°C of greater than or equal to 1.9 centistokes, but less than or equal to 4.1 centistokes, if the gravity was not determined by comparison with the supplier's certification;
 - c) A flash point equal to or greater than 125°F; and
 - d) A clear and bright appearance with proper color when tested in accordance with ASTM-D4176-82.
 - 2) By verifying within 30 days of obtaining the sample that the other properties specified in Table 1 of ASTM-D975-81 are met when tested in accordance with ASTM-D975-81 except that the analysis for sulfur may be performed in accordance with ASTM-D1552-79 or ASTM-D2622-82.
- e. At least once every 31 days by obtaining a sample of fuel oil from the storage tank, in accordance with ASTM-D2276-78, and verifying that total particulate contamination is less than 10 mg/liter when checked in accordance with ASTM-D2276-78, Method A.
- f. At least once per 18 months,* during shutdown, by:
 - 1) Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service,
 - 2) Verifying the generator capability to reject a load of greater than or equal to 1034 kW while maintaining voltage at 4160 ± 420 volts and frequency at 60 ± 4.5 Hz, (transient state), 60 ± 1.2 Hz (steady state).

*The specified 18 month interval may be extended to 31 months for Cycle 1 only.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-37,
AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO. NPF-66
AND AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE NO. NPF-72
COMMONWEALTH EDISON COMPANY
BYRON STATION, UNITS 1 AND 2
BRAIDWOOD STATION, UNIT 1
DOCKET NOS. STN 50-454 , STN-50-455 AND STN 50-456
TAC. NOS. 65952, 65953 AND 65954

INTRODUCTION

By application dated August 7, 1987 Commonwealth Edison Company (the licensee) requested amendments to the Technical Specification for Byron Station, Units 1 and 2, and Braidwood Station, Units 1 and 2. The proposed changes extend the diesel generator surveillance from 18 months to 31 months for the first cycle of Byron Station, Unit 2, and Braidwood Station, Units 1 and 2.

EVALUATION

The August 7, 1987 application proposed a footnote to Technical Specification 4.8.1.1.2f that would waive the 18-month interval for diesel generator surveillance testing, provided the testing is performed during the refueling outage. (The proposal applies to Byron 2 and Braidwood 1 and 2. Byron 1 has completed its first refueling outage). The licensee made this proposal after receiving agreement from the diesel generator manufacturer that a one-time extension of the surveillance interval is acceptable. The manufacturer based its acceptance on the licensee's preventive maintenance program and on the starting and running of the diesels for one hour fully loaded every 30 days. The manufacturer agreed to a 13-month extension which would allow a total of 31 months (18 + 13) for the first surveillance interval. The staff noted that unforeseen delays during the first cycle could extend the first refueling outage to beyond the 31 months accepted by manufacturer. Thus, the staff proposed, and the licensee agreed, that the footnote be changed to allow the extension to 31 months, rather than to the first refueling outage.

The proposed amendment involves the frequency of surveillance tests to prove diesel generator availability. Previously evaluated accidents which involve a loss of offsite power are the only accidents which depend on diesel generator availability. The probability of other accidents concurrent with a loss of offsite power are not affected by this diesel generator surveillance frequency.

The consequences of previously evaluated accidents would not be significantly increased because the diesel generator availability would not be significantly decreased. The previously established preventive maintenance programs at each station are sufficient to detect up to 95% of the potential failure modes that would be detectable during the performance of the deferred surveillances. The remaining 5% of the potential failure modes are mostly related to a mechanical wear type of failure. Because the operation time of the diesel generators is small, these kinds of failures have a small probability of occurring. This information provides confidence that the diesels will be capable of performing their intended function, resulting in no significant increase in the consequences of an accident previously evaluated.

The change does not add or modify any existing equipment, nor introduce a new mode of plant operation. The operability of the diesel generators will continue to be verified by performing the other related Technical Specification required surveillances, which remain unchanged. As such, the possibility of a new or different kind of accident from any previously evaluated is not created.

Deferral of the surveillance will not significantly increase the possibility of undetected degradation of the diesel generators because they are operated infrequently for short periods of time. Since the actual operation time of the diesel generator is small, the probability of a diesel generator failure due to mechanical wear is small. Commonwealth Edison Company reviewed the failure history for the diesel generator and did not find any trends identifying abnormal failures. The limiting conditions for operation and other surveillance requirements to verify operability are unchanged and will remain in effect. The monthly and quarterly diesel generator surveillances will continue to be performed during the surveillance interval extension, as well as the previously indicated preventive maintenance programs. As such, the margin of safety is not reduced.

ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of the facilities components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on

such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: L. Olshan

Dated: October 30, 1987